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Author(s)	Shimada Asako, Omori Hiroyuki, Kameo Yutaka		
Citation	Journal of Radioanalytical and Nuclear Chemistry,		
	314(2),p.1361-1365		
Text Version	Accepted Manuscript		
URL	https://jopss.jaea.go.jp/search/servlet/search?5060393		
DOI	https://doi.org/10.1007/s10967-017-5440-6		
	This is a post-peer-review, pre-copyedit version of an article		
Dialet	published in Journal of Radioanalytical and Nuclear Chemistry.		
Right	The final authenticated version is available online at:		
	https://doi.org/10.1007/s10967-017-5440-6		



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4	Names of the authors:
5	Asako Shimada, Hiroyuki Ohmori, Yutaka Kameo
6	Title: Dr., Mr., Dr.
7	Affiliation(s) and address(es) of the author(s):
8	Japan Atomic Energy Agency
9	2-4 Shirakata, Tokai-mura, Naka-gun, Ibaraki, 319-1195 Japan
10	E-mail address of the corresponding author:
11	shimada.asako@jaea.go.jp

# Development of determination method of <sup>93</sup>Mo content in metal waste generated at the Japan Power Demonstration Reactor

Asako Shimada<sup>1</sup>, Hiroyuki Ohmori<sup>1</sup>, and Yutaka Kameo<sup>1</sup>

- <sup>17</sup> <sup>1</sup>Department of Decommissioning and Waste Management, Japan Atomic Energy
- 18 Agency, 2-4 Shirakata, Tokai-mura, Naka-gun, Ibaraki 319-1195, Japan

## 19 Abstract

16

A separation method of Mo from Nb, Zr, and the matrix elements of rubble waste was modified to determine the content of <sup>93</sup>Mo in metal waste. A separation scheme to treat 1 g of metal waste was established by optimizing the amount of ascorbic acid, the rinsing solution, and repeating of the procedure. A thin-layer source was prepared using direct drop deposition and evaporation to measure <sup>93</sup>Mo content. Finally, <sup>93</sup>Mo content in the metal waste generated at the Japan Power Demonstration Reactor was analyzed using the developed method.

## 27 Keywords

<sup>93</sup>Mo, TEVA resin, low-level radioactive waste, metal waste, Japan Power Demonstration
Reactor

## 30 Introduction

The international Atomic Energy Agency proposed the application of scaling factors (SF) method, which is established by a correlation between the difficult to measure and easy to measure nuclides, for waste characterization in nuclear power plants [1]. Accordingly, the applicability of the SF method is being studied for the metal waste generated at the research

35 reactors in the Japan Atomic Energy Agency. Therefore, samples were obtained from the 36 waste generated in the Japan Power Demonstration Reactor (JPDR), and important 37 nuclides for the safety assessment have been analyzed [2-6].

38 Molybdenum-93 (with half-life 4000 y) is one of the difficult to measure nuclide, and is mainly produced by the neutron irradiation of <sup>92</sup>Mo contained in metal structural materials. 39 Molybdenum-93 decays to <sup>93m</sup>Nb (with half-life 16.13 y) through an electron capture 40 41 reaction, followed by the emission of a specific X-ray of Nb (the intensities of L (2.17 keV), 42 KA2 (16.521 keV), KA1 (16.6151 keV), and KB (18.6 keV) are 6.0, 21, 40, and 12%, respectively) [7]. The daughter nuclide, <sup>93m</sup>Nb, decays to stable Nb via an IT decay, and 43 44 this process also emits a specific X-ray of Nb (the intensities of L, KA2, KA1, and KB are 45 2.82, 3.24, 6.2, and 1.84%, respectively) [8]. Since <sup>93m</sup>Nb is produced by not only the EC decay of  $^{93}$ Mo but also the  $\beta^-$  decay of  $^{93}$ Zr. Mo has to be separated from Zr and Nb to 46 detarmine <sup>93</sup>Mo using the X-rays. Moreover, the X-ray energy of <sup>93</sup>Mo is sufficiently low 47 48 to be absorbed by the sample matrix and/or glass vial; further, the chemical separation of 49 Mo from the matrix elements and the preparation of thin-layer source are indispensable.

50 Recently, a chemical separation method of Mo from Zr, Nb, and the matrix elements of 51 rubble sample using a tetra valent actinide resin (TEVA resin) was developed in our 52 laboratory to analyze a small amount of rubble samples from Fukushima Daiichi Nuclear 53 Power Station [9]. In order to use this method for the analysis of the metal waste from the 54 JPDR, it should be modified because the sample amount is large (approximately 0.2 g of 55 Fe) and the metal waste contains a large amount of Fe, which interferes with the extraction 56 of Mo on the TEVA resin. In the present study, the amount of ascorbic acid of the sample 57 solution, rinse solution, and duplication of the separation were elucidated to optimize the separation of Mo. Finally, <sup>93</sup>Mo in the metal waste from JPDR was analyzed. 58

## 59 **Experimental**

#### 60 Materials

61 TEVA resin was purchased from Eichrom Technologies, USA (particle size 100-150 μm).

62 Standard materials of stainless steel (SUS304, JSS 651-10) and carbon steel (CS, JSS 050-

63 2) were purchased from the Japan Iron and Steel Federation. Hydrochloric acid (35-37%)

64 and nitric acid (60-62%) for the analysis of poisonous metal and perchloric acid (70%) and

65 hydrofluoric acid (47%) of analytical grade were purchased from Wako Pure Chemical

66 Industries, Ltd., Japan.

#### 67 Dissolution of metal samples and preparation of sample solutions

68 Grained SUS304 and CS were used in the experiments to optimize the separation method. 69 One gram of SUS304 or CS was placed in a beaker, and 15 mL of HCl and 5 mL of HNO3 70 were added; subsequently, the beaker was heated on a hot plate. The addition of these acids 71 and subsequent heating were repeated several times. Subsequently, the concentrated 72 solution was transferred into a PTFE beaker, and 10 mL of HF and 20 mL of HNO<sub>3</sub> were 73 added to evaporate Si. This process was also repeated several times. After the evaporation 74 of the solution, the residue was treated with 10 mL of HClO<sub>4</sub> and 20 mL of HNO<sub>3</sub> to 75 eliminate F completely. The residue was dissolved in HCl solution and filtered. Finally, 76 the solution was adjusted to obtain a 4 M HCl/0.1 M HF solution. Sample solutions to load 77 the TEVA resin column were prepared by adding ascorbic acid and 0.5 mg of Mo into the 78 digested solutions containing 0.5 g of SUS304 or CS. The digestion method of metal waste 79 generated at the JPDR was described in ref [2]. The metal surface was digested by using 80 acid.

#### 81 Modification of the separation scheme

82 The basic separation scheme is shown in Fig. 1. The sample solution consisted of 0.5 g of 83 SUS304 or CS, 0.1 M HF, 4 M HCl, and ascorbic acid. The concentration of ascorbic acid 84 in the sample solution was changed from 0.3 M to 1 M. The TEVA resin column (5 mL) 85 was conditioned with 25 mL of a solution whose composition is the same as the sample 86 solution but without SUS304 or CS. Subsequently, the sample solution was loaded into the 87 TEVA resin column and rinsed with the solution containing 0.1 M HF, 4 M HCl and the 88 same concentration of ascorbic acid as the sample (hereafter referred as "rinse1"). 89 Subsequently, the column was rinsed with a solution containing 0.1 M HF and 4 M HCl



92 Fig 1 Basic chromatographic scheme to optimize the separation of Mo from Fe

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94 (hereafter referred to as "rinse2"). The remaining Fe was rinsed with "rinse3" solution. 95 Where, 0.01 M HNO<sub>3</sub>, 0.1 M HF, and 2 M HF were examined as rinse3 solution to decrease 96 the elution of Mo. Finally, the extracted Mo was recovered with 1 M HNO<sub>3</sub> and the total 97 volume was adjusted to 50 mL. Moreover, the procedure was repeated for the selected 98 rinse3 (0.1 M HF) to improve the total Mo recovery. The concentration of Mo in the Mo 99 recovery fractions was measured using an inductively coupled plasma atomic emission 100 spectrometer (ICP-AES) system (Shimadzu ICPS-7500 sequential plasma emission 101 spectrometer). The percentage of recovery was calculated using the following formula:

102 Recovery percentage = 
$$\frac{Metal \ amount \ in \ the \ fraction \ \times \ 100}{Metal \ amount \ in \ the \ sample \ solution}$$

103 Loss of Mo during the preparation of thin-layer source

The loss of Mo during direct drop deposition was evaluated because some Mo compounds
have low boiling points. A standard solution containing 0.05 mg of Mo was placed on a Ta

106 disk and evaporated on a hot plate until it was dry. Subsequently, the deposit was dissolved

107 with 10 mL of 1 M HCl to measure the concentration of Mo using the ICP-AES.

108 Measurement of <sup>93</sup>Mo in the metal waste generated at the JPDR

109 Stable Mo (0.5 mL of 1000 ppm Mo solution in 0.4 mol HCl-0.2 mol HNO<sub>3</sub>) was added to 110 the dissolved solution of the metal waste generated at the JPDR (corresponding amount of 111 the metal waste was 1 g, for dissolved amounts of the waste was  $0.2\pm0.3$  g, 0.05-1 g), and 112 the solution was adjusted as 25 mL of 4 M HCl-0.1 M HF-0.6 M ascorbic acid to load into 113 the conditioned TEVA resin (5 mL). After rinsing with 4 M HCl-0.1 M HF-0.6 M ascorbic 114 acid (rinse1) and 4 M HCl-0.1 M HF (rinse2), the remaining Fe was rinsed with 0.1 M HF 115 (rinse3), and Mo was recovered with 1 M HNO<sub>3</sub> (Mo). Since a part of Mo was eluted in 116 the rinse3 fraction, the fraction was adjusted to 4 M HCl/0.1 M HF/0.6 M ascorbic acid 117 solution to repeat the chromatographic separation process. The Mo recovery fractions of 118 the first and second operations were combined and evaporated until they were dry. The 119 residue was dissolved in 3 M HNO<sub>3</sub> (0.5 mL). A part of the dissolved solution was diluted 120 to determine the concentration of Mo using the ICP-AES. The percentage of recovery of 121 Mo was calculated using the concentration. A thin-layer source was prepared according to ref [3]. X-rays of <sup>93</sup>Mo were detected using a low-energy photon detector (GLP-36360/13-122 123 P, Seiko EG&G).

124 **Results and discussion** 

#### 125 *Modification of the separation scheme*

126 In this study, the sample solution contained 0.5 g of SUS304 or CS. These amounts are 127 equivalent to approximately 6.6 mM and 7.5 mM of Fe, respectively; whereas, 25 mL of 128 0.3 M ascorbic acid solution contains 7.5 mM of ascorbic acid. In theory, 2 mole equivalent 129 of Fe is reduced by 1 mole equivalent of ascorbic acid. Therefore, 0.3, 0.6, and 1 M ascorbic 130 acid are equivalent to 2, 4, and 6.7 times the amount of required ascorbic acid to reduce Fe 131 in the sample solution, respectively. Table 1 presents the percentage of recovery of Mo in 132 the recovery fraction of Mo when the concentration of ascorbic acid in the sample solution 133 was changed from 0.3 to 1 M. When the concentration of ascorbic acid was 0.3 M, the 134 percentage of recovery was low (6% and 11% for SUS304 and CS, respectively). However, 135 the percentage of recovery was more than 50% for both SUS304 and CS when the

136 concentration of ascorbic acid was more than 0.6 M. An increase in the concentration of

137 ascorbic acid from 0.6 to 1 M did not cause an increase in the percentage of recovery.

138 Moreover, the dissolution of ascorbic acid is a time-consuming process. Therefore, the

139 concentration of ascorbic acid was optimized to 0.6 M.

140	Table1 The percentage of recovery of Mo in the recovery fraction of Mo.				
	Concentration of ascorbic acid	SUS304	CS		
	0.3 M	6%	11%		
	0.6 M	62%	53%		
	1 M	67%	50%		
		1 (77 17 2 2 4 1)	1 1 (00) 1 1 ( 110)		

<sup>141</sup> The sample consisted of 0.5 g of stainless steel (SUS304) or carbon steel (CS), 4 M HCl,

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The elution behavior of Mo in each fraction is summarized in Fig. 2, wherein 0.6 M ascorbic acid-4 M HCl-0.1 M HCl solution containing 0.5 g of SUS304 or CS was loaded into the TEVA resin. The subsequent rinse with rinse2, i.e., 4 M HCl-0.1 M HF, was skipped in this evaluation.



Fig.2 Elution behavior of Mo in each fraction

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<sup>0.1</sup> M HF, and ascorbic acid. The concentration of ascorbic acid was varied from 0.3 to 1M.

The Mo elution during the sample loading and rinsing with 0.6 M ascorbic acid-4 M HCl-152 153 0.1 M HCl solutions is small, 1-5%, and blue-green band was observed on the upper side 154 of the TEVA resin column. This indicates that Mo was extracted on the TEVA resin 155 regardless the sample solution contained a large amount of Fe. However, approximately 156 half of the total amount of Mo was eluted by the 0.01 M HNO<sub>3</sub> solution, and the blue-green 157 band disappeared followed by the elution of red-brown solution. It was reported that most 158 of Fe was eluted during the sample loading and the extracted Fe was eluted with 0.01 M 159 HNO<sub>3</sub> with remaining of the extraction of Mo on the TEVA resin when a rubble sample 160 solution containing 52 µg/mL of Fe and 40 µg/mL of Mo was analyzed [9]. However, a 161 large amount of Fe (approximately 0.37 g for SUS304 and 0.5 g for CS) in the sample 162 solutions elutes Mo together with the extracted Fe. Molybdenum may form a complex with 163 Fe in the sample solution and the formation is facilitated by the high concentration of Fe. 164 If the Mo-Fe complex is extracted on the TEVA resin, the elution of Fe from the TEVA 165 resin column may induce the accompanying elution of Mo.

Analogous column separation experiments were performed with different rinse 3 solutions, 0.01 M HNO<sub>3</sub>, 2 M HF, and 0.1 M HF. Table 2 presents the percentages of recovery of Mo in the Mo recovery fraction. This result suggests that 0.1 M HF slightly suppressed the elution of Mo with the extracted Fe. Moreover, the eluate with 0.1 M HF was adjusted to 0.6 M ascorbic acid-4 M HCl-0.1 M HF solution to load the TEVA resin column again. It was confirmed that more than 80% of Mo was recovered via duplication of the column separation.

/4	composition of rinses was change	ged		
	Composition of Rinse 3	SUS304	CS	
	0.01 M HNO <sub>3</sub>	62%	53%	
	2 M HF	19%	29%	
	0.1 M HF (first tern)	58%	81%	
	0.1 M HF (second tern)	26%	23%	
	0.1 M HF (total)	84%	104%	

173 Table2 The percentage of recovery of Mo in the Mo recovery fraction when the 174 composition of rinse3 was changed

- 176 Finally, the following conditions were selected to analyze <sup>93</sup>Mo content in the metal waste
- 177 of JPDR: 0.6 M ascorbic acid-4 M HCl-0.1 M HF for the sample solution, 0.1 M HF for
- 178 rinse 3 solution, and the eluate of rinse3 solution was adjusted to 0.6 M ascorbic acid-4 M
- 179 HCl-0.1 M HF to repeat the column separation procedure.
- 180 Loss of Mo during the preparation the thin-layer source
- 181 The direct drop deposition method was originally developed to prepare a thin-layer source 182 for the measurement of  $\alpha$ -ray [10]. Although tetraethylene glycol was used as a spreading 183 agent, it was not used in this case because the self-absorption of X-rays is considered to be 184 smaller than that of  $\alpha$ -rays. It was confirmed that the percentage of recovery of Mo deposit 185 was approximately 100%; in other words, the loss of Mo during the deposition was 186 negligible.

## 187 Measurement of <sup>93</sup>Mo in the metal waste from JPDR

- The analytical results of <sup>93</sup>Mo content in the metal waste of JPDR are summarized in Table 3. A high percentage of recovery (88±8%, 72-105%) was obtained in the analysis. All the <sup>93</sup>Mo concentrations were less than the detection limit of  $2 \times 10^{-2}$  Bq/g. It was shown that <sup>93</sup>Mo concentration in the metal waste of JPDR is at least 500 times less than that of the exclusion value of <sup>93</sup>Mo, i.e., 10 Bq/g [11].
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Sample	Sample No.	Percentage	<sup>93</sup> Mo
		of recovery	concentration
		(%)	(Bq/g)[5]
Pipe from feed-water system	M-926-1-2	82	$<2 \times 10^{-2}$
Reactor building	NY-926-557-2	105	$< 1 \times 10^{-2}$
Pipe from forced circulation system (exit)	NY-926-558-1	72	$<2 \times 10^{-2}$
Pipe from forced circulation system (exit)	NY-926-558-2	86	$<2 \times 10^{-2}$
Pipe from purification demineralizer system	NY-927-1030-1	95	$<2 \times 10^{-2}$
Pipe from liquid waste treatment system	NY-927-1047-1	91	$<2 \times 10^{-2}$
Pipe from liquid waste treatment system	NY-927-1050-1	90	$<2 \times 10^{-2}$
Pipe from liquid waste treatment system	NY-927-1050-2	92	$<2 \times 10^{-2}$
Filter from fuel pool	NY-927-976-1	94	$<2 \times 10^{-2}$
Pipe from liquid waste treatment system	NY-927-946-2	84	$<2 \times 10^{-2}$
Pipe from liquid waste treatment system	NY-924-225-1	80	$<2 \times 10^{-2}$
Valve from liquid waste treatment system	NY-922-1730-1	82	$<2 \times 10^{-2}$
Pipe from liquid waste treatment system	NY-927-957-1	97	$<2 \times 10^{-2}$
Pipe from purification demineralizer system	NY-927-932-1	85	$<2 \times 10^{-2}$
Device used for decommissioning	NY-923-1949-1	91	$<2 \times 10^{-2}$
Pipe from feed-water system	NY-91Y-989-1	89	<9×10 <sup>-3</sup>
Pipe from purification demineralizer system	NY-919-372-1	88	$<2 \times 10^{-2}$

200	Table 3 The analyt	cal results of	<sup>93</sup> Mo content in	the metal	waste of JPDR
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## 202 **Conclusions**

The existing analytical method of <sup>93</sup>Mo was successfully modified to treat a metal waste 203 sample. The concentration of ascorbic acid was increased from 0.3 M to 0.6 M to ensure 204 the reduction of Fe<sup>3+</sup> to Fe<sup>2+</sup>, and 0.1 M HF was used as a rinse solution for the extracted 205 Fe instead of 0.01 M HNO<sub>3</sub> to suppress the Mo elution with Fe. By repeating the separation 206 procedure for the rinse solution of Fe, the percentage of recovery obtained was more than 207 80%. Finally, the concentrations of <sup>93</sup>Mo in the metal waste of the JPDR were analyzed 208 using the modified method, and it was confirmed that they were lower than the exclusion 209 210 level.

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## 212 **References**

- 213 1. International Atomic Energy Agency, IAEA Nuclear energy series No.NW-T-1.18,
- 214 Determination and use of scalling factors for waste characterization in nuclear power plants,
- 215 Wagramer strasse 5, P.O.Box 100, 1400 Vienna, Austria, 2009
- 216 2. Hoshi A, Tsuji T, Tanaka K, Yasuda M, Watanabe K, Sakai A, Kameo Y, Kogure H,
- 217 Higuchi H, Takahashi K (2011) Analysis of the radioactivity concentration in low-level
- 218 radioactive waste generated from JPDR facilities, JAEA-Data/Code 2011-011
- 219 3. Tanaka K, Yasuda M, Watanabe K, Hoshi A, Tsuji T, Higuchi H (2013) Analysis of the
- 220 radioactivity concentration in low-level radioactive waste generated from JPDR facilities
- 221 part 2, JAEA-Data/Code 2013-008
- 4. Yasuda M, Tanaka K, Watanabe K, Hoshi A, Tsuji T, Kameo Y (2014) Analysis of the
- radioactivity concentration in low-level radioactive waste generated from JPDR facilities
   part 3, JAEA-Data/Code 2014-011
- 5. Ohmori H, Nebashi K, Shimada A, Tanaka K, Yasuda M, Hoshi A, Tsuji T, Ishimori K,
- 226 Kameo Y (2015) Analysis of the radioactivity concentration in low-level radioactive waste
- 227 generated from JPDR facilities part 4, JAEA-Data/Code 2015-029
- 228 6. Tsuji T, Kameo Y, Sakai A, Hoshi A, Takahashi K (2012) Study on the evaluation
- 229 method to determine the radioactivity concentration in low-level radioactive wastes
- 230 generated at JPDR facilities part 1, JAEA-Technology 2012-045
- 231 7. http://www.ndc.jaea.go.jp/NuC/Radout/93MO\_EC.txt
- 232 8. http://wwwndc.jaea.go.jp/NuC/Radout/93NB\_IT.txt
- 233 9. Shimada A, Ozawa M, Yabuki K, Kimiyama K, Sato K, Kameo Y (2014) Development
- of a separation method for molybdenum from zirconium, niobium, and major elements of
- rubble samples, J. Chromatogr. A 1371:163-167
- 236 10. Shinohara N, Kohno N (1989) Rapid preparation of high-resolution sources for alpha-
- ray spectrometry of actinides in spent fuel, Appl. Radiat. Isot. 40(1):41-45

- 238 11. International Atomic Energy Agency, Safety Reports Series, Derivation of Activity
- 239 Concentration Values for Exclusion, Exemption and Clearance, Vienna, 2005