

Bibliography on Irradiated Fuel Shipping
and its Shipping Casks

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日本原子力研究所

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Summary

Present problems of the irradiated fuel shipping and its shipping cask, especially from the standpoint of its design and manufacture, are described in this report.

About 400 literatures of foreign and our countries, which were gathered for this work, are divided into 12 groups according to their own fields, and listed chronologically. In this bibliography, literatures published after 1959 are mainly collected, considering efforts and development works on these problems done by the IAEA and many countries in these several years.

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使用済み燃料の輸送と輸送容器に関する文献集

要 旨

使用済み燃料の輸送問題についての最近の傾向を展望し、同輸送容器の設計上、製作上の問題点を紹介した。あわせて、この調査にあたって収集した約 400 の内外の文献を、その主題別に分け、年代順に掲載した。なお、本文献集にはここ数年間これらの問題に関してなされた国際原子力機関や多くの国の開発を考えて、主に 1959 年以降の文献を選択した。

1966 年 10 月

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1. Introduction

The shipment of irradiated fuels from nuclear reactors is becoming important both in Japan and other countries day after day, as the number of nuclear stations increases. Many countries hope to establish world-wide regulations on the irradiated fuel shipping and also at the same time to determine the methods of testing and evaluating the integrity of irradiated fuel shipping casks, so that there may be no problems on this shipping, even internationally.

In these four or five years, the IAEA has made its efforts to establish the regulations on the irradiated fuel shipping by holding the international panel meetings many times, and it is expected new regulations will be proposed in autumn, 1966.

Many development works on the irradiated fuel shipping and its shipping casks, including fire experiments of the casks, cask drop tests and survey works, have extensively been made in the U.S.A., U.K., etc.^{(92), (201), (374), (375), (376), (377), (378), etc.}

In the mean time, a great attention is now also given to this problem in Japan; the research committee on the irradiated fuel shipping cask was organized in the Japan Society of Mechanical Engineers last winter, the irradiated fuels from the JRR-2 (Japan Research Reactor-2) were shipped to Idaho Chemical Processing Plant, U.S.A. last August and the Government hopes to make regulations on this matter in the special committee referring to the regulations of the IAEA and U. S. A.

In view of the above situations, we have made a survey of the irradiated fuel shipping casks from the standpoint of their design and manufacturing, and collected the literatures on the research and survey works of the irradiated fuel shipping.

2. Present status on the regulations on irradiated fuel shipping^{(5.1), (5.2), (5.11)}

As this problem is very important from the standpoint of public and personnel safety, the IAEA issued the "Regulations for the Safe Transport of Radioactive Materials" in 1960; however the IAEA would like to make it more perfect and asked many experts of the member states to review this regulations on the basis of their actual shipping experiences and research and development works made since 1960. After having many meetings and discussions on this problem, the IAEA revised the above regulations in 1964,⁽⁵⁴⁾ to great extents. This IAEA's revised regulations made clear the safety standard, shipping cask design criteria and cask testing methods, although not much clear on such items as the model scale used for evaluating the cask integrity, shipping approval standards internationally, and so on,

The IAEA, therefore, had an international meeting on "Technical Safety Assessment of a Proposed Movement of a Loaded Irradiated Fuel Flask"^{(35), (36)} in April, 1964 and its first international panel meeting on the "Design and Testing of Packaging for Large Radioactive Sources with Special Reference to Irradiated Fuel Casks"^{(46), (47), (48)} in March, 1965,—both of which were held in Vienna—, to establish the regulations on the irradiated fuel shipping. With the results of the above panel meeting and the subsequent opinions by the member states experts, the draft regulations were proposed.^{(53), (54)} The second international panel meeting⁽⁶⁴⁾ was then held in February, 1966 and the new revised regulations are expected to be issued in this fall.

In the U. S. A., 10-CFR-Part 72⁽¹⁴⁾ "Protection against Radiation in the Shipment of Irradiated

Fuel Elements" was proposed in September, 1961 and 10-CFR-Part 71⁽²⁹⁾ "Shipment of Special Nuclear Material" in February, 1963; but both were revised (Draft)^{(44), (45)} in November, 1964. Taking into consideration the IAEA's work mentioned above and also the developments and actual experiences on the radioactive material shipment new 10-CFR-Part 71⁽⁶¹⁾ "Transport of Licensed Material" was proposed in December, 1965, which is very similar to the IAEA's regulations and revised in August 1966.^(61-a)

These regulations were made based on the shipping experiences of the radioactive materials and many development works which have been made in the U. S. A., U. K. and so on. Many countries are going to make or establish their own regulations on the irradiated fuel shipping and its shipping casks, following the IAEA's regulations.

As the irradiated fuel contains not only fissile materials but also radioactive substances, "Public Safety" and "Personnel Safety" are strictly required in designing its shipping cask and the shipment. The regulations in this connection are and have been made on the basis of this criteria, and the irradiated fuel shipping cask should be designed and manufactured under the conditions of containment of the fissile materials, type B packaging standard and the shipment of the large radioactive sources, according to the IAEA's regulations.

Each requirement in the regulations has its own significance which should be considered in the designing. For instance, "Hypothetical Accident Conditions" or "Test Procedures for Cask Integrity Evaluation" is derived by considering possible automobile collision accidents accompanied by a fire. It is reasonable to consider that the above hypothetical accidents do not represent any accident, and that if the cask could withstand the above accidents it would give good assurance for the cask performances in case of accidents, as written in 10-CFR-Part 71.⁽⁶¹⁾

However, it should be noted that the design conditions, such as shipping route, shipping method, etc., must be studied in each case including accidents, prior to the design of the cask. Another evaluation might be requested for the fire in a ship, and the requirements for the cask design may be influenced by the shipping route, if the route is restricted.

3. Problems in designing irradiated fuel shipping casks

The difficult problems in designing an irradiated shipping cask are the hypothetical accident conditions: these are "9m Drop", "Puncture Test" and "30 min. Fire". The development works on the irradiated shipping cask, therefore, are mainly concentrated on the solving of these items.

3.1 Problems in the heat removal design

3.1.1 Decay heat^{(5.3), (5.5)}

When making heat removal calculations for the irradiated fuel shipping cask, one must first estimate the decay heat from the irradiated fuels to be shipped. The decay heat is dependant upon the following,

- (a) irradiated period or time of the fuel in the reactor,
- (b) number of fissions per unit volume per unit time in the reactor,
- (c) cooling period or time of the fuel after removal from the reactor.

The cooling time of the irradiated fuel is usually taken to be about 90 to 120 days, from

the standpoint of the reprocessing and its shipment. Some experiments and measurements^{(233), (263)} were made on the decay heat of irradiated fuels, and its estimation could be made by the equations such as (1),⁽⁴⁰¹⁾ (2),⁽²³³⁾ etc., or by means of the work of the Oak Ridge National Laboratory, U. S. A.

$$P_s/P_0 = 5.7 \times 10^{-2} [T_s^{-0.2} - (T_0 + T_s)^{-0.2}] \quad \text{..... (1)}$$

$$P_s/P_0 = 0.1 \{ (T_s + 10)^{-0.2} - 0.87 (T_s + 2 \times 10^7)^{-0.2} \} - 0.1 \{ (T_s + T_0 + 10)^{-0.2} - 0.87 (T_s + T_0 + 2 \times 10^7)^{-0.2} \} \quad \text{..... (2)}$$

where P_s : decay heat

P_0 : thermal output of the reactor

T_s : cooling time (sec)

T_0 : irradiated time in the reactor (sec)

The heat removal calculation for an irradiated fuel shipping cask is usually done by assuming that all the decay heat of the irradiated fuel is generated within the fuel. When the cooling time of the irradiated fuel is 90 to 120 days, about half the decay heat is from the gamma photons, which means that some of the decay heat is produced directly in the shielding. If the heat generation in the shielding by the gamma photons can be estimated or calculated quantitatively, more irradiated fuels may be sent in one shipping cask, or the evaluation of its accidents may be easier, especially in the case of the coolant loss.

3.1.2 Atmospheric conditions

The IAEA regulations specify the following ;

- (a) -40°C and 70°C shall be considered,
- (b) the temperature of the accessible surfaces of the package shall not exceed 50°C in the shade when fully loaded, assuming still, ambient air at 38°C ; and other ambient air conditions which correspond to environmental conditions of transport may be assumed upon approval by the competent authority ; and if the package is transported as a full load the limit of 50°C above shall be raised to 82°C .

However, the temperatures in the ship and atmosphere in the shipping route should be examined carefully, and also the solar heat along the route must be considered ; these may affect the cask design temperature.

Many experiments on the solar heat have been done in the world⁽⁴⁰²⁾, and the solar constant is taken about $1.94 \text{ cal/cm}^2 \text{ min}$. As some of this energy may be absorbed in the air, about 0.9 of the total may be suggested to take.⁽²⁵⁹⁾

3.1.3 Fire

The irradiated fuel shipping cask is required to withstand the fire accidents during its shipment. The standard one hour fire designated in the ASTM, E 119-61 was at first considered for this accident evaluation. This standard one hour fire, however, is mainly stipulated for the fire in the building, and the probability of such a fire accident is probably larger in the case of vehicle collisions.

On the basis of the above evaluation principle, the IAEA took the open fire testing method ($30 \text{ min.}, 800^\circ$), in which the following positions of the cask tested are suggested by considering the open fire characteristics, temperature distribution in the fire, the emissivity of the fire, etc.

- (a) The package shall be supported so that the bottom of the package is 1 m above the bottom of the tank which will hold the fuel for the fire ;
- (b) The fire shall be such that all the sides of the package are exposed to a luminous flame not less than 0.7 m and not more than 2 m thick.

The burning rate of the fuel pool and the flame height were measured in U.S.S.R.⁽⁹²⁾ ; the former would be about 4 mm/sec when the diameter of the fuel pool was more than 1 m and the latter about 1.7 times as much as the diameter of the fuel pool which was over 2 m. Rough estimations could be made on the fire exposure time of the shipping cask using the results of such tests. In the case of vehicle collisions, however, the fuel may not flow out all at once and some fuel will also soak into the earth, which makes difficult to evaluate this accident precisely. The temperature of the flame may exceed 800°C in some parts, but it would be reasonable to make an evaluation tests of the shipping cask under the condition of being completely surrounded by the 800°C of flame, taking into account such fire phenomena in the vehicle collision accident.⁽⁵⁶⁾

As the shipping cask is usually made of lead with steel lining. It would be important to know how the lead would behave in the case of the fire from the standpoint of the personnel radiation safety. Experiments were performed on this problem and there occurred cavities in the lead shield after exposed to the fire in some case^{(247), (253)} (Fig. 1). It was found that this could be

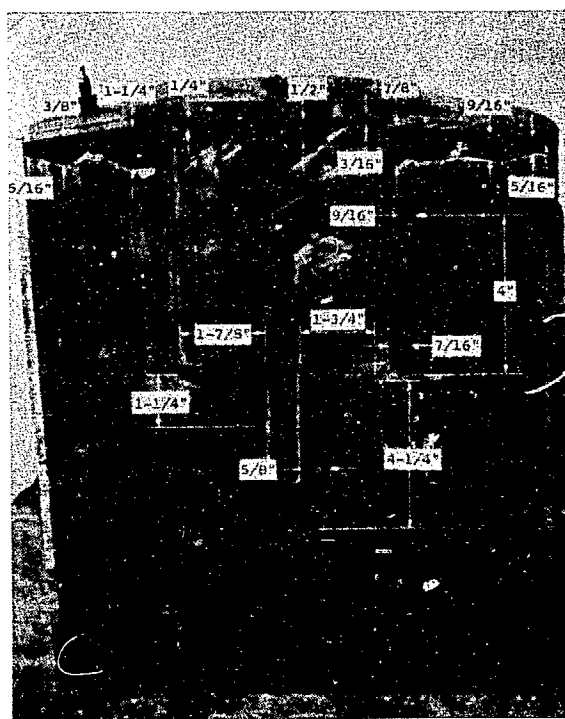


Fig. 1⁽²⁴⁷⁾ Cutway of cask after being exposed by fire

prevented by surrounding the cask with wood,^{(248), (374)} however, this method is not considered satisfactory because of the heat removal of the irradiated fuel. Some suggestions and cask mock-ups were made including those in which depleted uranium metal or concrete with depleted uranium dioxide⁽¹¹⁰⁾ was used instead of the lead, or some gaps were provided around the cask body for fire protection.

It is also important the gasket which would show good behaviours at the time of a fire or in the high temperature atmosphere.

3.1.4 Coolant loss

In this case, heat removal of the decay heat of the irradiated fuel is very important for safety. Since atmospheric air is the only coolant available in the cask (in this accident), the decay heat of the irradiated fuels is mainly dissipated by the radiation and one could not evaluate this heat removal precisely, due to the gamma heating in the shield, etc. Many experiments and measurements are thus necessary to establish the evaluation methods and to obtain the data for the evaluation. Some dried-type shipping casks (using no liquid coolant) are now used in the U. S. A.^{(153), (192)}

3.2 Problems in the shielding design of the cask^{(5.3), (5.6)}

The numbers of gamma photons emitted from the irradiated fuel decreases day by day after removal from the reactor; which may be calculated by separating them into several energy groups.^{(269), (270), (403), (404)} In the shielding calculation, however, the gamma photons of less than 0.8 Mev energy are rather insignificant, and one has to consider those with long half lives and high energies.⁽¹¹⁵⁾ These nuclides would be found in the literatures.^{(181), (403), (405)}

The comparative data of the shielding design values and their actual measured ones are much appreciated by the designers as well as manufacturers; but there are available few data on this problem.

3.3 Problems in the cask construction or structure^{(5.2), (5.3), (5.4)}

3.3.1 Acceleration and vibration received under normal shipment

It is necessary to have the data on the accelerations (Fig. 2) and vibrations suffering during

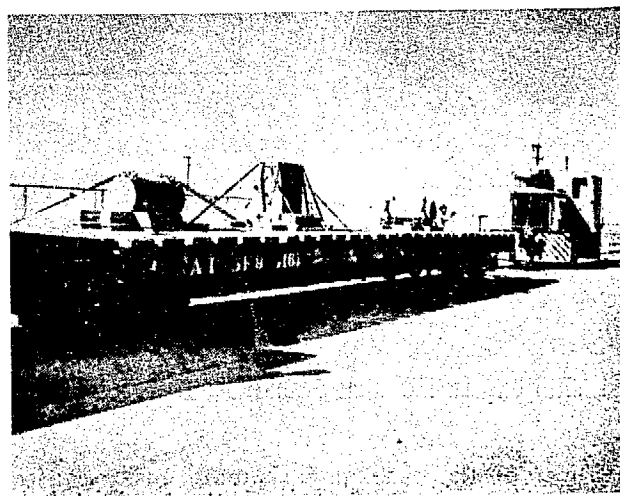


Fig. 2 Shunting test

the cask shipment, these are much affected by the nature of the route, including the road conditions and the method of transportation. In Table-1⁽⁹²⁾ are shown the results of studies made in the U. K. for this problem, which also has made studies on the testing methods, such as the

"Bump Test", "Vibration Test", referring to the testing methods of packages employed in the U. K. army.⁽⁹²⁾

TABLE 1⁽⁹²⁾ Maximum accelerations of shocks in ordinary shipping conditions

Road	3 g	(anchored package suspension on rough road)
Rail	2 g	(rail joint)
	(10 g)	(shunting)
Sea	3 g	(slamming in rough weather)

3.3.2 Penetration check

The IAEA specifies the penetration check in its regulations, in which the cask is dropped from the height of 1 m to the target (15 cm \pm 0.5 cm in diameter, 20 cm long). This testing is adopted by considering that the cask might collide with a protruding objects, such as the angle, etc., during its shipment, or that an automobile has a collision accident followed by the gasoline fire, which may cause the leak of lead out of the cask if the cask should be penetrated.

The Franklin Institute⁽²⁰⁷⁾⁽²²²⁾⁽²¹⁹⁾ has studied this problem systematically, using specimens (Fig. 3) composed of lead and steel liners just like the wall of an actual shipping cask and

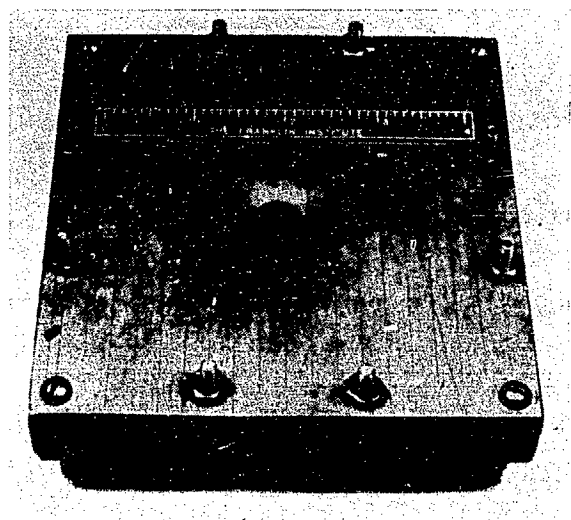


Fig. 3⁽²²²⁾⁽²¹⁹⁾ Penetration test specimen

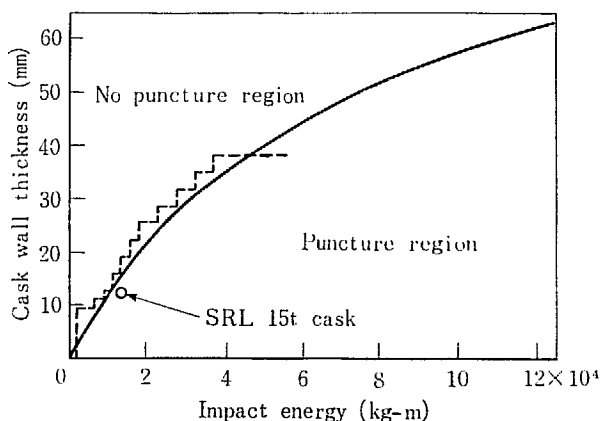


Fig. 4⁽²²²⁾⁽²¹⁹⁾ Puncture design curve
dotted line : proposed value specified in 10-CFR-Part
72 draft (1964)

also analyzing the actual shipping cask testing data. They obtained the following results or conclusions.

- (a) The outer steel liner of the cask may be more easily penetrated by the target with the flat surface than by the one with the spherical surface.⁽²⁰⁶⁾
- (b) The ordinary steel may be more easily penetrated than the high-tension steel.
- (c) Penetration limit may be roughly guessed from Fig. 4.⁽²²²⁾⁽²¹⁹⁾

The depth of penetration is found not to be proportional to the scale factor, however, the penetration limit through the wall could be estimated rather precisely by the sample or specimen testing method developed in the Franklin Institute. This specimen testing method may, therefore,

be considered useful for this purpose.

The usual shipping casks are weak against this penetration as was shown by the Savannah River Laboratory's experiments.⁽²¹⁵⁾ Then a multilayer shipping cask (steel-lead-steel—steel)⁽²²²⁾ was proposed for preventing the leak of lead out of the cask in the collision accident accompanied by a fire. This multilayer shielding cask was shown to be stronger than ordinary casks (steel-lead-steel) in the penetration.

3.3.3 Shock and drop tests

The IAEA also specifies the 9 m drop test in the regulations for evaluating the cask integrity, considering such accidents during the shipment as the vehicle collision, derailment of the train car, falling down from the bank or crane, etc. The height of 9 m was taken by considering the height of the crane was loaded on the ship, the speed of the vehicle (50 km/hr), and so on. It is very difficult to evaluate this accident precisely only by analysis and calculation, and moreover the full scale testing would be very expensive. Therefore, the model drop testing is now attracting much interest in the world. Recently many irradiated fuel shipping casks are shipped with the crush bars around the cask (Fig. 5), so that the crush bars may be broken and the cask be protected when the cask received a big shock in the accident.



Fig. 5 JRR-2 irradiated fuel shipping cask

The shock or deceleration of the vehicle collision depends on the type of collision and kind of vehicle. This accident was measured experimentally in the U. S. A., and one example is shown in Fig. 6.⁽³⁰⁷⁾

The drop tests of the shipping cask have been made extensively in the U. S. A., at Savannah River Laboratory,⁽²¹⁵⁾ (15 ton cask, Fig. 7), Oak Ridge National Laboratory (four 1.4 ton casks and two 6 ton casks), Franklin Institute,⁽²¹⁹⁾ Hanford Laboratory, etc. The general conclusions and tendencies are as follows :

- (a) in the drop test of the lead ball, the strain is proportional to the impact velocity and is not influenced by the ball size.
- (b) the deformation is proportional to the scale factor,

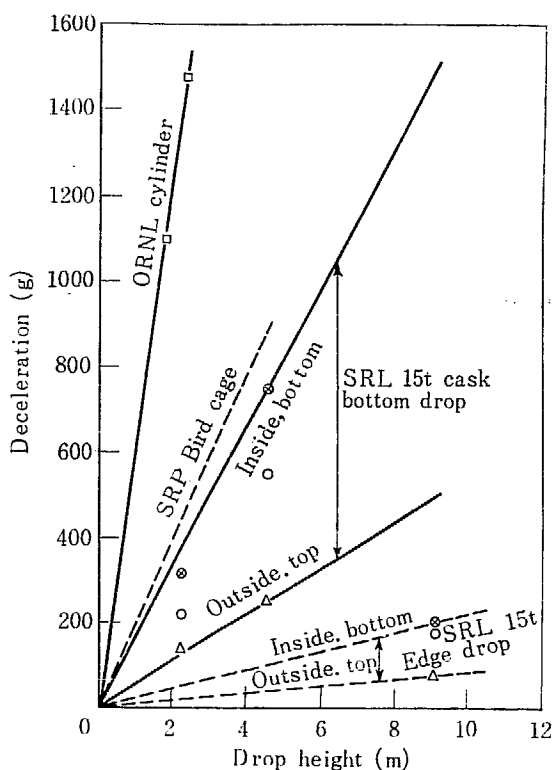


Fig. 9⁽²¹⁵⁾ Comparison of decelerations recorded in drop test

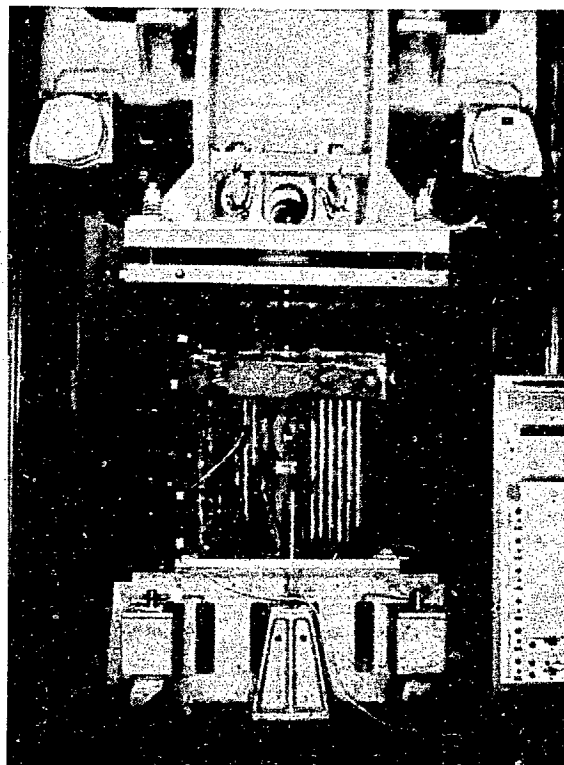


Fig. 10⁽²¹⁵⁾ Compression test of 15 ton DuPont cask

shown in Fig. 10, and it is found that the cask is weaker when the compression is applied diagonally.⁽²¹⁵⁾

3. 4 Water decomposition

The pressure within the cask (wet type) may increase due to the water decomposition caused by radioactivity of the irradiated fuel and chemical reaction between the water and cladding materials. This problem is serious when the magnox fuel is shipped and the pressure in the cask may sometimes increase at the rate of as much as 0.2 to 0.4 kg/cm²/day.⁽¹¹⁵⁾

Therefore, when the magnox fuel of the Latina was shipped from Italy to the U. K., the refrigerators were provided in the ship so that the water temperature of the cask might be maintained about 10°C during the shipment.⁽¹²⁰⁾

4 Miscellaneous

Many other items should be studied including statistical analysis^{(291), (295), (5, 7)} of the accidents, filters, cask standardization from the standpoint of receiving facilities in the reprocessing plants, fission products release at cladding failure, and so on.

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5. Bibliography

5. 1 Regulations, Standards, Panel Papers and so on

1. **Handbook of federal regulations applying to transportation of radioactive materials.** United States Atomic Energy Commission, Division of Construction and Supply, Traffic Management Section. May 1958. v, 48 p.
2. **[10 CFR] Part 71—regulations to protect against accidental conditions of criticality in the shipment of special nuclear material.** 10-7-58. 23 p.
3. 放射性物質車両運搬規則（昭和33年5月20日運輸省令第16号）核原料物質、核燃料物質及び原子炉の規制に関する法律、原子力損害の賠償に関する法律、原子力損害賠償補償契約に関する法律、関係法令集。科学技術庁原子力局、昭和38年7月。p. 177~182.
4. **New rules for transporting radioactive materials.** A. SHTAN' and N. LESHCHINSKII. 1959. 1s. AEC-TR-4227; Атомная энергия, том 7, p. 399 (1959).
5. **Interpretation of regulations governing the shipment of radioactive material.** CARL W. BUCKLAND, JR. *A.M.A Archives of Industrial Health*, vol. 19, p. 33~43 (1959)
6. **Federal regulations applying to the transportation of radioactive materials.** P. 3. 42~3. 72 of Radiation hygiene handkook. New York, McGraw-Hill, 1959.
7. **Recommendations of the International Commission on Radiological Protection, report of Committee II on permissible dose for internal radiation (1959).** New York, Pergamon, 1959. xxxii, 233 p.
8. **Coast guard regulations.** Title 46, code of federal regulations. 11-15-60. 44 p.
9. **The legal aspects of atomic waste disposal and transport of radioactive materials.** R. H. BURNS. P. 187~201 of Atomic energy waste; its nature, use and disposal. New York, Interscience, 1961.
10. **Regulations for the safe transport of radioactive materials. Notes on certain aspects of the regulations.** Vienna, IAEA, 1961. 105 p. Safety series no. 7
11. Правила перевозки радиоактивных веществ. Москва, Госатомиздат, 1961. 64 p.
Regulations for the transport of radioactive materials. Moskva, State Atomic Energy Publishing House, 1961. 64 p.
12. **Regulation for the safe transport of radioactive materials.** Vienna, IAEA, 1961. 72 p. Safety series, no. 6
13. **Federal and state regulation of transportation of atomic materials.** ARVIN E. UPTON and CHARLES A. EHREN, JR. *Atomic Energy Law Journal*, vol. 3, no. 2, p. 123~157 (1961)
14. **[10 CFR] Proposed part 72—protection against radiation in the shipment of irradiated fuel elements.** 10-12-61. 19 p. Atomic Energy Law Report
15. **Interstate Commerce Commission regulations. Title 49, code of federal regulations. Parts 71-78—explosives and other dangerous articles.** 12-13-61. 28 p.
16. **Basic safety standards for radiation protection.** Vienna, IAEA, 1962. 57 p. Safety series, no. 9.
17. **Atomic energy regulation by states.** W. L. HARWELL, E. J. GLENNON and J. L. SMITH. *Nuclear Safety*, vol. 4, no. 2, p. 14~26 (1962)
18. **Regulatory control; a review of national and international regulations for the transport of active materials.** *Nuclear Engineering*, vol. 7, p. 268

~273 (1962)

19. **United States Atomic Energy Commission contract for chemical processing and conversion services.** Dec. 1962. 32 p.
20. **Development of interstate commerce commission regulations.** THURBER C. GEORGE. P. 45~54 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
21. **Interagency Committee work on U. S. transportation regulations applicable to radioactive materials.** LESTER R. ROGERS. P. 55~87 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
22. **U. S. Atomic Energy Commission regulations pertaining to fissile materials.** ROBERT F. BARKER. P. 88~98 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
23. **Development of transportation regulations in Canada.** W. A. MARTIN. P. 99~102 of Summary report of AEC Symposium on Packaging and regulatory Standards for Shipping Radioactive Material. TID-7651
24. **Development of transport regulations in the United Kingdom.** ROY GIBSON and W. de L. M. MESSENGER. P. 103~125 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
25. **Methods and problem in evaluating and approving casks using safety standards.** ALEXANDER E. AIKENS, JR. P. 126~131 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
26. **Canadian technical standards for design and testing of irradiated fuel shipping containers.** [1963] 13 p. for IAEA Panel, 15~16 March 1965
27. **The 1963 revision of the IAEA transport regulations.** R. GIBSON. *Nuclear Engineering*, vol. 8, p. 195~197 (1963)
28. **Standards and tests for radioactive-materials shipping casks.** K. W. HAFF and L. B. SHAPPERT. *Nuclear Safety*, vol. 4, no. 4, p. 18~21 (1963)
29. **[10 CFR] Proposed part 71 shipment of special nuclear material.** (Published in 28 Federal Register, March 5, 1963) 9 p. Atomic Energy Law Report
30. **[10 CFR] Proposed part 70—licensing of special nuclear material.** 3-14-63. 45 p.
31. **First draft. Provisional revised text of the International Atomic Energy Agency's regulations for the safe transport of radioactive materials.** June 1963. vii, 71 p. (WP/33)
32. **Transportation information for the shipment of irradiated fuel elements.** Materials Branch, Division of International Affairs, USAEC. Nov. 1963. 20 p.
33. **American standard recommended practice for design and operation of shipping containers for irradiated solid fuel from nuclear reactors.** American Institute of Chemical Engineers. Dec. 1963. 16 p.
34. **Revision of the regulations for the safe transport of radioactive materials; draft of the revised regulations.** International Atomic Energy Agency, Board of Governors. Feb. 1964. 1 vol. (various ings.) (GOV/966/Add. 1) (GOV/966/Add. 1/Mod. 1) (Gov/966/Add. 2)
35. **The technical safety assessment of a proposed movement of a loaded irradiated fuel flask; draft paper prepared for a meeting of experts from selected Member States to be held at the Agency's Headquarters in Vienna, 27~30 April 1964.** Vienna, Division of Health, Safety and Waste Disposal, I.A.E.A. 11 p.
36. **Report on the technical safety assessment of a proposed movement of a loaded irradiated fuel Flask (held a by IAEA, 27~30 April 1964.)** Y. KAWASHIMA, Y. ISHIDA
37. 一欠番一

38. [10 CFR] Part 70—special nuclear material. 5-14-64. 19 p.
39. Transport regulations: the need for compatibility. I. G. K. WILLIAMS. *Atom*, no. 94, p. 191~195 (Aug. 1964)
40. Atomic energy legislation in the United Kingdom with particular reference to transport. I. G. K. WILLIAMS. P. 25~30 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug. 1964. AHSB(A)-R-8
41. Transport regulation, (a), general principles underlying regulations governing the transport of radioactive materials. J. B. W. ARMSTRONG. P. 47~51 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug. 1964. AHSB(A)-R-8
42. Transport regulations, (b), the development of international and national transport regulations. R. GIBSON. P. 53~58 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug. 1964. AHSB(A)-R-8
43. Postal regulations. Title 39, code of federal regulations. 8-7-64. 2 p.
44. [10 CFR 71] Draft criteria for the transport of special nuclear material. Nov. 1964. 25 p. (For discussion purpose only).
45. [10 CFR 72] Draft criteria for the transport of irradiated special nuclear material. Nov. 1964. 13 p. (For discussion purposes only).
46. Comments on the "first preliminary draft of the working paper for the panel for the development of a set of standards for the design and testing of irradiated fuel casks in respect of the transport of those casks". Jan. 1965. 21 p. (Panel members only).
47. Working paper for the participants of the panel (panel on the design and testing of packaging for large radioactive sources with special reference to irradiated fuel casks (15~26 March 1965)). International Atomic Energy Agency. Jan. 1965. 22 p. (PL 145/1) For discussion only.
48. Note on the transportation of irradiated fuel. Commissariat a l'Energie Atomique. Feb. 1965. 24 p. (for IAEA Panel, 15~26 March 1965)
49. Federal aviation regulations. Title 14—code of federal regulations. Part 103—transportation of dangerous articles and magnetized materials. 1965. 5 p.
50. Handbook of federal regulations applying to transportation of radioactive materials. United States Atomic Energy Commission, Division of Construction, Traffic Management Branch, Washington, D. C. 1965. vi, 88 p.
51. Interstate commerce commission regulations and permits for the safe transportation of explosives. NOYES, H. E. 1965. 1s. (5 p.) LA-DC-7469
52. Regulations for the safe transport of radioactive materials. 1964 rev. ed. Vienna, International Atomic Energy Agency, 1965. 104 p. (IAEA, Safety series, no. 6) STI/PUB/97
- 52-a. Regulations for the safe transport of radioactive materials 1964 revised edition. (Safety Series No. 6). Modification No. 1. International Atomic Energy Agency, 10 p., April, 1966
53. Draft. Proposals in respect of the requirements for packages containing large radioactive sources, with special reference to irradiated fuel. May 1965. 13 p. IAEA. (PL 145/MS/2) (Not for publication)
54. Draft. Proposed code of practice (design guide) on the design, fabrication and testing of packaging for large radioactive sources, with special reference to irradiated fuel. May 1965. 8 p. (PL 145/MS/3) Not for publication) IAEA.
55. Regulatory packaging standards. BARKER, R. F., HOPKINS, D. R. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 45~54.
56. Packaging standards. APPLETON, G. J., SERVANT, J. Y. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. SC-RR-65-98, p. 32~44. or Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61

(1965), p. 50~57.

57. **Special problems from the viewpoint of bridge, tunnel and turnpike authorities.** LEE, P. B. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 57~65.
58. **Problems pertaining to packaging and transportation regulatory standards applicable to shipments of radioactive materials.** NOYES, H. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 66~73 or LA-DC-6866
59. **Regulatory standards for irradiated fuel shipments.** AIKENS, A. E., LINDBERG, B. G. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 651~661.
60. **Practices and procedures for licensing.** AIKENS, A. E. JR. Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 42~49.
61. **10 CFR proposed Part 71—transport of licensed material.** Atomic Energy Law Reports, 20613-

5. 2 Irradiated Fuel Shipping

65. **Shipping of radio-isotopes.** KARL Z. MORGAN. *Journal of Applied Physics*, vol. 19, no. 7, p. 593~598 (1948)
66. **Transportation of large quantities of radioactive materials.** HANSON BLATZ. A. M. A. *Archives of Industrial Health*, vol. 14, p. 1~6 (1956)
67. **Comments on the transportation of irradiated fuel and radioactive wastes for M. Louis Armand, EURATOM group.** F. L. CULLER. 1957. 1s. CF-57-5-24
68. **Factors affecting the transport of radioactivity by water.** FRANKLIN B. BARKER. *Journal American Water Works Association*, vol. 50, p. 603~612 (1958)
69. **Guide to shipment of U-235 enriched uranium**

20635. Dec. 30, 1965.

- 61-a. **10 CFR Part 71—Packaging of radioactive material for transport.** Atomic Energy Law Reports, 20609-20627. August 21, 1966 (effective)
62. **Analyse comparative des règlements du transport des matières radioactives en vigueur dans les pays-membres de la Communauté Européenne de l'Energie Atomique par rapport au règlement de l'Agence Internationale de l'Energie Atomique.** Da Novi, P. 1966. 242 p. EUR-2747. f
63. **On the international panel of "Design and Testing of Packaging for Large Radioactive Sources with Special Reference to Irradiated Fuel Casks" (1st Japanese)** NUKAZUKA. *Journal of Atomic Energy Society of Japan*. Vol. 8. No. 4. P. 50~52 (1966)
64. **Pre-Final draft standard and requirements for the design, testing and shipment of large radioactive source packages with special reference to irradiated fuel.** Joint Meeting on the Design and Testing of Packaging for Large Radioactive Sources, with Special Reference to Irradiated Fuel, held in 14-18. Feb. 1966. Revised PL-145-2. Restricted
- materials. H. F. HENRY. June 1959. vii, 75 p. TID-7019
70. **Army gas-cooled reactor systems program. Transportability studies ML-1 nuclear power plant.** J. W. BLAKLEY, J. S. ALCORN, L. G. DEL VALLE, P. W. HEALY, D. H. MORAN and W. S. SCOTT. Apr. 1960. viii, 62, 207 p. IDO-28555
71. **Le transport des matières radioactives.** YVES DUVAUX. *Energie Nucléaire*, vol. 2, no. 5, p. 295~301 (1960)
72. **A laboratory study to define conditions for loading strontium-90 on Decalso for HAPO-1A cask shipment.** F. M. SMITH. 1961. 2s. HW-71574
73. **Haftungsrechtliche Fragen bei der Beförderung von Kernbrennstoffen.** G. H. SCHEUTEN. *Atomwirt-*

- schaft*, Jg. 6, Nr. 1, p. 82~84 (1961)
74. Die Beförderung von Kernbrennstoffen und Brennelementen. H. BRÜCHER und H. ESCHNAUER. *Atomwirtschaft*, Jg. 6, Nr. 9, p. 431~435 (1961)
 75. Present trends in fuel transportation. Eurochemic technical report no. 109. F. MARCUS. June 1961. 1s. (27 p.) NP-12393
 76. Le transport des matières radioactives. YVES DUVAUX. 1961. 7 p. CEA-1869
 77. Transporting irradiated fuel elements. *Reactor Fuel Processing*, vol. 4, no. 4, p. 1 (1961)
 78. 放射性物質などの運搬・放射線安全取扱手引. 1961年10月. 2 p. JAERI-6007
 79. Transporting irradiated fuel elements. *Reactor Fuel Processing*, vol. 5, no. 1, p. 1~2 (1962)
 80. Experience in the shipment and chemical reprocessing of GETR and VBWR fuel. K. STRATTON. *Transactions of the American Nuclear Society*, vol. 5, no. 2, p. 361 (1962)
 81. Spent fuel transfer, storage and shipment for PL-3. G. C. HAUENSTEIN and D. L. POMEROY. 1962. 2s. APAE-MEMO-310
 82. Radiochemical processing-off-site transportation and ultimate storage problems. J. O. BLOMEKE and L. B. SHAPPERT. 1962. 1s. ORNL-TM-164
 83. Shipping and handling of radioactive materials. P. 4.61—of Bibliography on nuclear reactor fuel reprocessing and waste disposal, vol. 4; hazards and protection. T. F. CONNOLLY. ORNL-2971
 84. Fuel shipping studies. P. 199~201 of Chemical Technology Division annual progress report for period ending June 1962. 3 p. ORNL-3314
 85. Power reactor spent fuel transport services taking shape. *Nucleonics*, vol. 20, no. 10, p. 26~27 (1962)
 86. Etude de transport vers les Etats-Unis des combustibles irradiés dans les réacteurs de puissance Européens, vol. 1. 1962. 42 p. EUR-112. e, f
 87. Etude du transport vers les Etats-Unis des combustibles irradiés dans les réacteurs de puissance Européens, vol. 3. 1962. 510 p. EUR-112. e, f
 88. First radioactive cargo comes into a U.S. port. *Business Week*, No. 1769, P. 24~25 (1963)
 89. Transport of radioactive material and other legal aspects. P. 193~197 of Radiation hazards and protection. London, George Newnes, 1963
 90. Transportation of irradiated fuel. *Reactor Fuel Processing*, vol. 6, no. 1, p. 3~4 (1963)
 91. Shipment of gross quantities of radiostrontium. L. L. ZAHN, C. W. SMITH and R. L. JUNKINS. P. 623~640 of Treatment and storage of high-level radioactive wastes, proceedings of the Symposium on Treatment and Storage of High-Level Radioactive Wastes. Vienna, IAEA, 1963. or (HW-SA-2514)
 92. The transport of radioactive materials. Interim recommendations for the application of environmental tests to the approval of packaging. W. de L. M. MESSENGER and A. FAIRBAIRN. 1963. xi, 117 p. AHSB(S)-R-19
 93. Rapport d'études pour le transport des combustibles irradiés. 1963. 174 p. EUR-185. f, d, e
 94. Report concerning the study of the transportation of irradiated fuel from the SENN reactor at Garigliano (Italy). Special report no. 1. 2s. EURAEC-215
 95. Shipping and storage of radioactive materials; a literature search. THEODORE F. DAVIS. Aug. 1963. 24 p. TID-3552 (Rev. 2)
 96. Fuel shipping studies. p. 233~235 of Chemical Technology Division annual progress report for period ending May 31, 1963. Sept. 1963. 3 p. ORNL-3452
 97. The transportation of highly radioactive materials; a review of current research. J. TRUEMAN THOMPSON. Oct. 1963. 26 p. NYO-9774
 98. Spent Fuel Shipping Cask and Its Shipment (I). S. SAWAI, H. TAKAYANAGI. July 1964. 49 p. JAERI-memo 1628.

99. **The transport of radioactive materials introductory address.** F. W. SPIERS. p. 7~11 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug., 1964. AHSB(A)-R-8
100. **The nature and extent of radioactive traffic in the U.K., (a), isotopes.** R. WEST. p. 31~35 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug., 1964. AHSB(A)-R-8
101. **The nature and extent of radioactive traffic in the U.K., (b), movement of fuel and of fuel materials.** K. J. NORMAN. p. 37~45 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug., 1964. AHSB(A)-R-8
102. **The transport of fissile materials.** J. T. DANIELS. p. 59~62 of Proceedings of the Symposium on the Transport of Radioactive materials. Aug., 1964. AHSB(A)-R-8
103. **Spent Fuel Shipping Cask and Its Shipment (II).** S. SAWAI, H. TAKAYANAGI. Sept. 1964. p. 38. JAERI-memo 1683
104. **Transportation of irradiated fuel.** *Reactor Fuel Processing*, vol. 7, no. 1, p. 1~3 (Winter 1963-1964)
105. **Problems on Spent Fuel.** F. SAKAUCHI *Nuclear Power (Japan)*. Vol. 8, No. 1, P. 3~9.
106. **Problems on Spent Fuel Shipping.** T. OTSUBO, *Nuclear Power (Japan)* vol. 8, no. 1, p. 24~31, 1964)
107. **Survey on Problems of Spent Fuel Shipping.** K. MUROTA. *Nuclear Power (Japan)*, vol. 8, No. 1, p. 32~44, 1964
108. **Problems relating to the transportation of MTR-type irradiated fuels.** R. SCALLIET. p. 195~200 of EURATOM scientific and technical activities. 1964. EUR-1850. e
109. **Certaines questions d'assurance portant sur le transport par mer de combustibles nucléaires irradiés et non irradiés.** 1964. 102 p. EUR-1639. f
110. **Experience in the transport of irradiated fuel.** N. F. C. BISHOP, J. C. EGAN and W. W. MARSHALL. May 1964. 14 p. A/CONF. 28/P-184
111. **Transport of irradiated nuclear fuel on the European continent.** F. R. MARCUS and J. ASYEE. May 1964. 18 p. A/CONF. 28/P-422
112. **Shipment of a large liquid source-a case history.** F. A. ERNEST and R. MAHER. Dec. 1964. 1s. (13 p.) CONF-641201-2
113. **Preparation and transportation of piqua fuel elements. Summary report.** NEFF, K. E. Oct. 1964. 1s. (32 p.) NAA-SR-Memo-10529
114. **Transportation of irradiated fuel.** *Reactor Fuel Processing*, vol. 8, no. 1, p. 2-3 (Winter 1964-1965)
115. **Experience in the shipment of Yankee fuel assemblies for post-irradiation examination.** B. JAMES, D. D. PAYNE and H. E. WALCHLI. Feb. 1965. x, 60, 90 p. WCAP-6062
116. **A survey of research in the transportation of radioactive materials.** THOMPSON, J. T. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 17-31.
117. **Aspects of transportation of radioactive materials.** GEORGE, T. C. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 74-78.
118. **Shipment of a large liquid source-a case history.** Ernest, F. A. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. SC-RR-65-98, p. 545-566. or Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 63-73.
119. **Transport of irradiated fuel from nuclear power stations in the United Kingdom.** AINSWORTH, S. I. J. *Brit. Nucl. Ener. Soc.*, vol. 4, no. 4, p. 299-306 (1965).
120. **Transport of irradiated fuel from Latina to Windscale.** KAVANAGH, M. T. *et al. Nuclear Engineering* Vol. 11, No. 117 p. 117-122, Feb., 1966

121. 使用済燃料輸送に関する諸問題； 使用済燃料輸送問題検討会報告書。 原子力国内事情， 第11巻第5号， 24~32ページ (1966年)。
122. Design of a shield carrier. E. C. BARKER. 1s. AECD-2583
123. Wall transfer unit and transfer carrier for hot cells. C. W. ANGEL and F. RING, JR. *Nucleonics*, vol. 11, no. 9, p. 69 (1953)
124. Carrier design of irradiated APPR fuel elements. F. D. MIRALDI and A. D. ROSSIN. 1954. 3s. KT-178
125. Irradiated fuel assembly shipping container; functional requirements. 1s. AECU-3575
126. Controlled venting of storage and shipping containers. G. V. LEMMON. 1955. 1s. SCTM-9-55-12
127. Design of MTR fuel-element-source shipping cask for railway mobile irradiation facility. Progress report. L. E. BROWNELL, J. PATTERSON and S. N. PUROHIT. 1957. 2s. AECU-3624
128. Design criteria for the type M-130 standardized shielded shipping container. B. B. BIGGS. 1958. 1s. KAFL-M-BBB-2
129. 70-ton shipping cask for the Savannah river plant design and fabrication. W. H. PIPER and J. W. LANGHAAR. Jan. 1959. 71 p. DP-357
130. A portable radioactive solution container. BERGENE KAWIN. *International Journal of Applied Radiation and Isotopes*, vol. 5, p. 305-306 (1959)
131. A high intensity cobalt container for industrial radiography. *Metallurgia*, vol. 59, no. 356, p. 311-312 (1959)
132. Metal space frame shipping containers. C. W. HUGHES. p. 215-223 of AEC and Contractor SS Materials Management Meeting, May 25-28, 1959. Nov. 1959. TID-7581
133. Новый контейнер для источников излучения высокой активности. В. Синицын, Н. Лещинский и А. Гусев. *Атомная Энергия*, том 7, p. 399~400 (1959)
- New container for highly radioactive materials.
134. A shielded container for transporting radioactive spent fuel. B. B. BIGGS. 1960. 1s. TID-6395
135. OMRE fuel removal and shipping equipment. P. J. MALLON, D. S. DUNCAN and R. C. NOYES. Apr. 1960. v, 41 p. NAA-SR-3337
136. Fission product shipping cask. Design criteria and hazards evaluation. C. W. SMITH. May 1960. 2s. (48 p.) HW-65268
137. Proposed nylon tiedown system for ML-1. J. S. ALCORN. July 1960. 2s. (51 p.) AGN-TM-379
138. Safer packages for shipping fuel. W. B. LEWIS and R. W. GOIN. *Nucleonics*, vol. 18, no. 7, p. 91 and 93 (1960)
139. Semi-remote connector for fission product casks. JERRY J. CADWELL. 1961. 1s. HW-70316
140. Strontium-90 shipping cask design criteria and hazards evaluation. C. W. SMITH. 1961. 2s. HW-68081
141. Structural analysis and design considerations for shipping containers of highly radioactive materials. ROBERT G. SANFORD. May 1961. 78 p. NYO-9374
142. The packaging, transport and related handling of radioactive materials. F. R. FARMER. Sept. 1961. 13 p. DPR-INF-264
143. Demonstration of unloading technique for the HAPO II-2 strontium filter cask. W. V. DEMIER. 1962. 1s. HW-72295
144. Design and economic study of spent fuel shipping. Quarterly report no. 2, July 1 to September 30, 1961. GERALD FISHER. 1962. 1s. EURAEC-167
145. Type B packages, design criteria. A. GRANGE. *Nuclear Engineering*, vol. 7, p. 274-276 (1962)
146. Radioisotope consignments. *Nuclear Engineering*, vol. 7, p. 277-279 (1962)
147. Containers for the shipment of spent reactor fuel

- elements. E. C. LUSK, H. M. EPSTEIN, R. J. EIBER and J. E. GATES. *Transactions of the American Nuclear Society*, vol. 5, no. 2, p. 361-362 (1962)
148. **Design and economic study of spent fuel shipping.** Quarterly report no. 4, January 1, 1962 to March 31, 1962. GERALD FISHER. Mar. 1962. 1s. EURAEC-292
 149. **Design and economic study of spent fuel shipping.** Stanray Corp., Chicago, Ill. May 1962. 3s. (196 p.) EURAEC-374 (Vol. 2)
 150. **Structural integrity of shipping containers for radioactive materials, part I; study of transport operations and container construction.** H. G. CLARKE, JR. and M. M. REDDI. July 1962. 145 p. NYO-9859
 151. **Fission product casks and containers prototype design, scaling of models for impact testing.** D. W. MCLENEGAN. Oct. 1962. 1s. (14 p.) HW-SA-2807
 152. **HAPO in fission product shipping cask design evaluation report.** C. W. SMITH. Dec. 1962. 3s. (72 p.) HW-65268 (Rev. 1)
 153. **Piqua Elk river reactors spent fuel shipping cask design report.** 1962. 3s. (179 p.) TID-21246
 154. **Design and economic study of spent fuel shipping,** quarterly report, May 1, 1961 to June 30, 1961. GERALD FISHER. 1962. 1s. EURAEC-164
 155. **Problems in designing casks to meet regulatory standards.** JOHN W. LANGHAAR. p. 132-144 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
 156. **Description of a plutonium and enriched uranium shipping container and integrity tests on this container.** C. L. SCHUSKE. Nov. 1962. 1s. (10 p.) RFP-290 See also: TID-7651, p. 187-196
 157. **Description of a plutonium and enriched uranium shipping container and integrity tests on this container.** C. L. SCHUSKE. P. 187-196 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
 158. **Application of external energy absorbers to large fission product shipping casks.** D. W. MCLENEGAN, C. W. SMITH, E. A. RIPPERGER and J. E. BREEN. P. 265-303 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
 159. **The design of shielded containers to IAEA standards.** F. E. DIXON. P. 331-387 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
 160. **A specification of wood and cadmium shielded class I packages proposed for IAEA transport regulations, 1963.** F. ABBEY and R. A. O'SULLIVAN. 1963. 20 p. fig. 1 p. AHSB (S)-R-63
 161. **Analysis of HFIR irradiated fuel element shipping cask.** J. H. EVANS, G. H. LLEWELLYN, T. W. PICKEL and B. W. WIELAND. Jan. 1963. 3s. (161 p.) ORNL-TM-959
 162. **Freight container for radioactive materials.** NACHABAR, HENRY D.; BIGGS, BILLY B.; TARIERE, PAUL J.; GEORGE, KURT O. Translated for Oak Ridge National Lab., Tenn., from Japanese Patent 409, 289. Apr. 1963. 1s. (12 p.) ORNL-TR-600
 163. **New developments in the design of shipping casks for irradiated fuel elements.** E. C. LUSK and J. E. GATES. *Nuclear News*, vol. 6, no. 6, p. 11-14 (1963)
 164. **The influence of packaging on radiation fields from radioactive shipments.** GEORGE COWPER. June 1963. 9 p. fig. 1 p. AECL-1802
 165. **Single element shipping cask for PNPf, OMRE, and EOGR irradiated fuel elements.** C. JONES, D. LEW, K. ADLER and J. MARRON. June 1963. 3s. (87 p.) NAA-SR-MEMO-7844
 166. **The effect of IAEA regulations on the design of shielded containers.** F. E. DIXON. *Atom*, no. 84, p. 315-324 (Oct. 1963)
 167. **Radioisotope shipping container development.** K. W. HAFF. 1964. 1s. (8 p.) ORNL-P-856: CONF-641201-13

168. **Structural integrity of shipping containers for radioactive materials. Part II. Some aspects of fire simulation in shipping container studies.** M. M. REDDI. Feb. 1964. 1s. (21 p.) NYO-2539-1)
169. **Ågesta fuel element casks.** G. JONASSON. (A. B. Atomenergi, Studsvik, Nyköping, Sweden) Feb. 1964. 6 p. TPM-RKS-172
(for IAEA Panel, 15~26 March 1965)

5. 3 Shipping Cask and Its Design

170. **A nuclear safe container for class II shipments of dry uranium compounds.** W. T. MEE and F. G. WELFARE. Apr. 1964. 17 p. Y-1460
171. **Shipping casks for the BR2 and HFR research reactors.** A. de STREEL, J. VAN MULDER, T. de SCHRYNMAKERS de DORMAEL, G. BONNET, C. GARRIC and M. TROUSSON. May, 1964. 15 p. A/CONF.28/P-448
172. **Approval of packaging design.** A. FAIRBAIRN. p. 63~80 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug. 1964. AHSB(A)-R-8
173. **The design and testing of irradiated fuel flasks in relation to the safe transport of loaded flasks.** United Kingdom Atomic Energy Authority. Aug. 1964. 8 p.
174. **Interbuilding fuel transfer coffin for the EBR-II reactor.** G. J. BERNSTEIN, A. A. CHILENSKAS and R. F. MALECHA. Sept. 1964. 21 p. ANL-6934
175. **Plutonium nitrate shipping packages.** F. E. ADCOCK, R. D. GAUTHIER, J. T. BYRNE, D. W. PARK, R. L. DELNAY and C. L. SCHUSKE. Oct. 1964. xii, 138 p. RFP-437
176. **Structural integrity of shipping containers for radioactive materials. Part IV. A discussion of approaches to the vehicle-collision problem.** K. D. DOSHI. Dec. 1964. 1s. (20 p.) NYO-2539-3
177. **Preliminary experimental gas-cooled reactor spent fuel shipping cask design.** L. B. SHAPPERT. Jan. 1965. 1s. (18 p.) ORNL-TM-1001
178. **Some Problems on Spent Fuel Shipping Cask.** S. SAWAI. *Journal of Atomic Energy Society of Japan*. Vol. 7, No. 3, p. 167~168. 1965
179. **Plutonium oxide shipping packages.** ADCOCK, F. E.; BYRNE, J. T.; DELNAY, R. L. Apr. 1965. viii, 90 p. RFP-501
180. 一欠番一
181. **Some problems on design and technical assessment of JRR-2 shipping cask made by Edlow Lead Comp.** S. SAWAI. May 1965. 20 p. JAERI-memo-1952
182. **Progress report on a class II container for low assay uranium compounds.** GUTMAN, T. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 330~356
183. **Class I shipping container for fissile material.** BROWN, C. L. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 377~398 or HW-SA-3995
184. **Description and evaluation of four Pu(NO₃)₄ shipping containers.** DELNAY, R. L. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 423~441
185. **The problem of container interchangeability.** PATTERSON, D. E. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 442~449
186. **A standard series of packagings for loads up to 20,000 curies of cobalt 60 or equivalent.** DIXON, F. E. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 534~542 or AERE-M-1512
187. **Design of plutonium oxide and plutonium nitrate**

shipping containers. ADCOCK, F. E. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 609~619

188. **Pressure venting device for spent fuel element shipping casks.** LUSK, E. C. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 620~624
189. **Special problems in packaging and transportation of radioactive materials.** EDLOW, S. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 625~629
190. **The transport container for irradiated fuel from civil power stations in the UK.** JOHNSTON, K. W. H. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 711~718
191. **Radioisotope shipping container development.** HAFF, K. W. Proceedings of the International

5. 4. Testing, Mainly Structural Testing

196. **Fuel shipping container test. Core 1, seed 1, section 1. Test results T-643717-A.** Duquesne Light Co. 1961. 1s. DLCS-3250101
197. **Sea disposal container test and evaluation; technical report.** June 1961. vii, 132 p. TID-13226
198. **Strength tests of prototype sea disposal containers for radioactive wastes.** JACK, R. POHLMAN and M. M. LEMCOE. Oct. 1961. vi, 53 p. TID-13942
199. **Design verification test program, Hanford designed shipping systems.** L. L. ZAHN and R. L. JUNKINS. 1962. 1s. HW-SA-2655
200. **Non-radioactive testing of the HAPO 1-2 fission product shipping cask.** W. V. DEMIER. Feb. 1962. 1s. (6 p.) HW-71744
201. **Report of working meeting on shipping container**

Symposium for Packaging and Transportation of Radioactive Materials. SC-RR-65-98, p. 413~422 or Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 74~77

192. **Design of containers for shipment of radioactive materials.** PETERSON, R. W. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. SC-RR-65-98, p. 450~472 or Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 16~29
193. **Structural analysis of shipping casks. Vol. 1. Analysis of a shipping cask subjected to internal pressure.** SPALLER, A. E.; SHOBE, L. R. Mar. 1966. 2s. (61 p.) ORNL-TM-1312 (Vol. 1)
194. **An improved shipping container for fissile material.** GASKILL, JAMES R., TAYLOR, ROBERT D. June 1966. 26 p. UCRL-14903
195. **Irradiated Fuel Shipping Cask.** S. SAWAI. *Journal of Japan Society of Mechanical Engineers*, Vol. 69, No. 571. p. 1047~1053 (1966)
- testing programs at the Johns Hopkins University, May 2~3, 1962. JAMES M. MORGAN, JR., ARNOLD B. JOSEPH, JOHN D. STEVENSON and DONALD K. JAMISON. Aug. 1962. 154 p. TID-7635
202. **Tests of a proposed uranium container.** J. D. MCLENDON. Nov. 1962. 2s. (27 p.) Y-KB-22
203. **Test of fissile material shipping containers.** JAMES M. HOFFMANN. P. 145~160 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
204. **Tests of a proposed uranium container.** J. D. MCLENDON. P. 161~186 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651

205. **Static and impact tests on a full-scale, irradiated fuel shipping cask.** WAYNE I. THISELL. P. 197-210 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
206. **The cask testing program at the Oak Ridge National Laboratory.** L. B. SHAPPERT. P. 211-237 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
207. **Model impact tests pertaining to shipping containers for radioactive materials.** H. G. CLARKE, JR. and W. E. ONDERKO. P. 238-264 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
208. **Free fall and fire test on a type 28 container in a western red cedar packing case, all up weight 17 Cwt.** F. E. DIXON. P. 388-393 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
209. **Development of safety performed criteria and tests standard for radioisotopes sealed sources and devices.** HITCH, JAMES W. 1963. 1s. (12 p.) CONF-650616-5
210. **Impact testing of shipping containers.** L. B. SHAPPERT. *Transactions of the American Nuclear Society*, vol. 6, no. 1, P. 172 (1963)
211. **Vibration and shock, highway transport between Santa Susana mountains and Edwards air force base.** E. L. GARDNER. Jan. 1963. 1s. (23 p.) NAA-SR-MEMO-8074
212. **Model studies of buffered shipping containers for fission products. Final report.** E. A. RIPPERGER. May 1963. 5s. (111 p.) HW-77963
213. **Drop test of unbuffered 1/4 size model HAPO-IB cask.** C. W. SMITH. June 1963. 1s. (23 p.) HW-77962
214. **Drop tests performed on the Franklin Institute's 1/4 scale model of a 40-ton cask.** L. B. SHAPPERT. Aug. 1963. 2s. (33 p.) ORNL-TM-629
215. **Static and impact tests on 15-ton cask for shipping irradiated fuel.** W. I. THISELL and J. W. LANGHAAR. Aug. 1963. 44, 108 p. DP-843
216. **Isotopes Development Center safety testing program. Progress report, March~April 1963.** R. A. ROBINSON. Sept. 1963. 2s. (28p.) ORNL-TM-630
217. **Impact and thermal tests on transport containers ICC 2-R and ICC 6-J.** HORACE E. NOYES. Dec. 1963. 12 p. LAMS-2983
218. **Results of impact tests performed on 55-gal drum-type birdcages.** L. B. SHAPPERT. Dec. 1964. 57 p. ORNL-3735
219. **Impact resistance of casks.** CLARKE, H. G. JR. Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 30~41. or CONF-641201-16
220. **Impact tests on 20-inch-cube birdcage-type transport container.** (B. E. 1012) HORACE NOYES and DAVID SMITH. Jan. 1965. 15 p. LA-3203-MS
221. **The testing of isotope shipping containers.** HORN L. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 119~132.
222. **Experimental studies of cask impact resistance.** CLARKE, H. G., JR. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 261~282.
223. **Destructive drop tests of a model fuel shipping cask.** SAYWELL, G. L. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-6598, p. 283~301.
224. **Results of a testing program for unirradiated fissile material containers.** SHAPPERT, L. B. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 321-329. or ORNL-P-309

225. **Testing shipping containers for slightly enriched uranium billets, ingots, and fuel cores.** DUNAWAY, D.L. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 357~376
226. **Testig of 3M shipping containers for enriched uranium.** ROWE, W.E., FREDERICKSON, R.H. Proceedings of the International Symposium for Packaging and Transportation of Radiactive Materials. June, 1965. SC-RR-65-98, p. 399~410
227. **Experimental work at AERE on the developmnet of packaging for radioactive materials which will meet the IAEA regulations.** DIXON, F.E., PART-
RIDGE, A.H. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 473~505. or AERE-R-4810
228. **UF₆ container test and development program at the Oak Ridge Gaseous Diffuson Plant.** MALLET, A. J. Proceedings of the Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 567~579, or K-D-1845
229. **Testing of 10-ton capacity uranium hexafluoride shipping containers.** MYERS, J.L., PEDIGO, W.R., HAMER, W. J., SMITH, V.A., BERNSTEIN, S. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965. SC-RR-65-98, p. 580-608
230. **Impact and thermal tests on transport containers ICC 2-R and ICC 6-J.** NOYES, H. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 736-743.
231. **Testing of ten-ton uranium hexafluoride cylinders.** PEDIGO, W.R.; SMITH, V.A.; BERNSTEIN, S.; HAMER, W. J.; MYERS, J.L. Oct. 1965. 33, 24 p. KY-500
232. **Structural integrity of shipping containers for radioactive materials. Part V. An analytical study of longitudinal vehicle-collisions.** DOSHI, K.D. Nov. 1965. 1s. (49 p.) NYO-2539-4

5. 5 Heat Removal and Fire Test

233. **Heat generation in irradiated uranium.** S. UNTERMYER and J. T. WEILLS. Feb., 1952. 20 p. AECD-3454.
234. **Fuel assembly heat evolution in transfer coffin.** D.T. BRAY and W.L. MORRIS. 1952. IDO-16008
235. **Heat transfer from spent fuel and blanket sub-assemblies during transfer from reactor to decay storage facility. Technical memorandum no. 2.** L. L. KINTNER. 1957. 2s. AECU-4058
236. **Permissible fuel heat generation rates for OMRE shipping cask.** R.C. NOYES. Apr. 1958. 1s. (10 p.) NAA-SR-MEMO-2658
237. **Preliminary heat transfer and cooling studies for fuel transfer operations.** J. T. REAM. Oct. 1958. 2s. (33 p.) NAA-SR-MEMO-3034
238. **Decay heat in shipping containers.** J. C. SUDDATH. 1959. 1s. CF-59-11-30
239. **Natural convection cooling system for the fuel handling cask.** J. D. WILDL. 1959. 1s. NAA-SR-MEMO-4178
240. **Surface temperatures of irradiated ORR fuel elements cooled in stagnant air.** J. F. WETT, JR. 15 p. ORNL-2892
241. **Evaluation of M-130 heat transfer test.** JAMES L. NASH. 1960. 2s. KAPL-M-S3G-RES-74
242. **Heat transfer study of cobalt-60 shipping container.** L. B. ADLER. *Transactions of the American Nuclear Society*, vol. 4, no. 2, p. 361 (1961)
243. **Heat transfer study of cobalt-60 shipping container; progress report no. 1.** L. B. ADLER. Mar. 1960. 23 p. BNL-657
244. **Heat transfer study of cobalt-60 shipping container; progress report no. 2.** L. B. ADLER. June 1961. 9 p. fig. 2 p. BNL-684

245. **Fire testing of radioisotope shipping containers.** LEONARD HORN. *Transactions of the American Nuclear Society*, vol. 5, no. 2, p. 294~295 (1962)
246. **Fire testing of radioisotope shipping containers and cobalt 60 teletherapy head.** LEONARD HORN. p. 304~314 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
247. **Fire tests of shipping containers and cobalt-60 teletherapy head.** LEONARD H. HORN. Mar. 1963. xiv, 65 p. COO-274
248. **Fire test of wooden-jacket shield for radioisotope shipping container.** LEONARD H. HORN. Mar. 1964. xi, 33 p. COO-275
249. **An analysis of steady-state heat transfer in a spring-loaded dry shipping cask.** F. T. STETSON. 1s. (18 p.) KAPL-M-6106
250. **Piqua-Elk river reactors spent fuel shipping cask.** Thermal test report. May 1964. 3s. (70 p.) COO-1028-2
251. **Study of plutonium oxide heat generation in a wood moderated container.** J. D. MOSELEY. Dec. 1964. 29 p. RFP-418
252. **Resistance to fire of lead lined casks.** AUPETIT, A. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 107~113.
253. **Fire tests of packagings for radioactive materials.** SOUSSELIER, YVES. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 114~118
254. **Can we model a fire test?** WACHTELL, G. P. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 133~140
255. **Thermal tests on an insulated, large source, cobalt 60 container design No. 0063.** DIXON, F. E., PARTRIDGE, A. H., BROOK, A. J. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 506~533
256. **Fire test of 15-ton shipping container.** WACHTELL, G. P. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 631~650.
257. **Studies on fire resistivities of radioisotope containers.** MORIYA, T., JIN, T. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 723~731
258. **Approximate calculative assessment made relating to wall temperatures and behaviour of cask in fire test.** TEAGUE, H. J., BROOK, A. J. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 732~735
259. **Heat removal from casks.** GLYNN, JAMES C. Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 58~62.
260. **Heat transfer in liquid hydrocarbon fuel fires.** BADER, B. E. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. SC-RR-65-98, p. 79~106 or Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 78~90

5. 6 Shielding, Radioactivity and Health Physics

261. **CPP health physics manual.** American Cyanamid Co. 1952. 4s. IDO-14055
262. **Shielding requirements for shipment of metallurgical samples.** LEE H. BOYER. Feb. 1953. 24 p. IDO-16066
263. **Experimental and theoretical values of the gamma**

- decay dose rate and heating from spent MTR fuel elements. W.C. FRANCIS and L.L. MARSDEN. 1956. 2s. IDO-16247
264. HRT sample carrier shielding calculations. D.M. JOHNSON. 1956. 1s. (8 p.) CF-56-1-122
265. Reactor shielding design manual. THEODORE ROCKWELL, III. New York, McGraw-Hill, 1956. p. 35
266. Shielding of irradiated APPR-1 fuel elements. F. B. FAIRBANKS and D. C. MORSE. 1956. 1s. APAE-MEMO-34
267. Shielding calculations for APPR-1 demineralizer shipping containers. R. C. DEYOUNG. 1957. 1s. APAE-MEMO-98
268. Effective-energy method for spent-fuel shielding. RAMON L. ASHLEY. *Nucleonics*, vol. 16, no. 10, p. 78~81 (1958)
269. Uranium-235 fission-product production as a function of thermal neutron flux, irradiation time, and decay time. I. Atomic concentrations and gross totals. J. O. BLOMEKE and MARY F. TODD. 211 p. ORNL-2127 (Pt. 1) (Vol. 1)
270. Uranium-235 fission-product production as a function of thermal neutron flux, irradiation time, and decay time. I. Atomic concentrations and gross totals. J. O. BLOMEKE and MARY F. TODD. 212~426 p. ORNL-2127 (Pt. 2) (Vol. 2)
271. Gamma damage to ethylene glycol in MTR 90-day cooled fuel element shipment. F. R. KELLER. Feb. 1958. 3 p. IDO-16441
272. Contamination of shipping cask and storage canal water by fuels irradiated in pressurized water reactors; a review of pertinent subjects. L. J. KING. 1960. 1s. CF-60-3-49
273. Methods of calculation for use in the design of shields for power reactors. A. F. AVERY, D. E. BENDALL, J. BUTLER and K. T. SPINNEY. May 1960. 96, 86 p. fig. 11 p. AERE-R-3216
274. Radiation dose received by passengers and crew on planes carrying radioisotope shipments. H. H. ABEE and D. M. DAVIS. 1961. 1s. (11 p.) ORNL-P-745:CONF-803-5
275. Reactor handbook, volume III, part B; shielding. EVERITT P. BLIZARD and LORRAINE S. ABBOTT. New York, Interscience, 1962. p. 28-33.
276. Evaluation of uranium shielding in the transportation of radioisotopes. J. STUART LICHLITER. p. 450-457 of proceedings of the Eighth Conference on Hot Laboratories and Equipment, San Francisco, California, December 13-15, 1960. TID-7599 (Book 2)
277. The classification of radionuclides for transport purposes and the derivation of activity limits in relation to package requirements. K. J. ASPINALL and A. FAIRBAIRN. May 1963. vii, 74 p. AHSB (RP)-R-23
278. Reactor Physics constants July 1963. p. 635-636. ANL-5800 (2d ed.)
279. Radiation and other risks. F. D. SOWBY. p. 89-108 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug., 1964, AHSB (A)-R-8
280. Radioisotope sources and source testing. NIE-MEYER, R. G. 1965. 1s. (17 p.) ORNL-P-1910
281. A summary of accidents and incidents involving radiation in atomic energy activities, June 1945 through December 1955. DANIEL F. HAYES. 1956. 3s. TID-5360
282. A summary of incidents involving radioactive material in atomic energy activities, January—December 1956. DANIEL F. HAYES. Aug. 1957. vi, 25 p. TID-5360 (Suppl.)
283. A summary of transportation incidents in atomic energy activities, 1949~1956. Dec. 1957. v, 26 p. AECU-3613
284. Nuclear and radiation hazards evaluation of SRE fuel processing and storage. J. C. SUDDATH.

5. 7 Hazards and Accidents

1959. 1s. CF-59-5-90
285. **A summary of industrial accidents in USAEC facilities.** Sept. 1959. iv, 30 p. TID-5360 (Suppl. 2)
286. **A compendium of information for use in controlling radiation emergencies; including lecture notes from a training session at Idaho Falls, Idaho, February 12~14, 1958.** ALLEN BRODSKY and G. VICTOR BEARD. Sept. 1960. vii, 100 p. TID-8206 (Rev.)
287. **Guarding against major radiation accidents in a large city.** DANIEL E. LYNCH. *American Industrial Hygiene Association Journal* vol. 22, p. 130~132 (1961)
288. 一欠番—
289. **The consequences of accidental releases during shipments of radioactive cesium and strontium.** E. C. WATSON, R. I. JUNKINS, J. J. FUQUAY and L. L. ZAHN. June 1961. HW-69561 (Rev.) (Del.)
290. **Statistical analysis of the frequency and severity of accidents to potential highway carriers of highly radioactive materials.** F. F. LEIMKUHLE, M. J. KARSON and J. T. THOMPSON. July 1961. 188 p. NYO-9771
291. **A study of the possible consequences and costs of accidents in the transportation of high level radioactive materials.** J. M. MORGAN, JR., J. W. KNAPP and J. T. THOMPSON. Aug. 1961. 116 p. NYO-9772
292. **A summary of industrial accidents in USAEC facilities.** Aug. 1961. iv, 33 p. TID-5360 (Suppl. 3)
293. **The potential hazards of shipping gross quantities of radionuclides.** E. C. WATSON. May 1962. 1s. HW-SA-2518
294. **A summary of incidents involving USAEC shipments of radioactive material, 1957~1961.** D. E. PATTERSON and V. P. DEFATTA. Nov. 1962. ix, 70 p. TID-16764
295. **Types and quantities of materials being shipped and AEC accident experience.** D. E. PATTERSON. p. 3~17 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
296. **Theoretical consequences of accidents.** J. M. MORGAN, JR., J. W. KNAPP and J. T. THOMPSON. p. 18~30 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
297. **A proposed study of rail and highway transport systems during a credible accident.** ROBERT MELDRUM. p. 315~330 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
298. **A summary of incidents involving USAEC shipments of radioactive material, 1962.** D. E. PATTERSON and A. MEHN. Nov. 1963. viii, 28 p. TID-16764 (Suppl. 1)
299. **Accidents in shipping radioactive material.** L. B. SHAPPERT. *Nuclear Safety*, vol. 5, no. 1, p. 123~127 (1963)
300. **A summary of industrial accidents in USAEC facilities, 1961~1962.** Dec. 1963. v, 34 p. TID-5360 (Suppl. 4)
301. **Introduction to radioactivity and associated hazards with particular reference to transport, (a), health physics aspects.** F. MORLEY. p. 13~17 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug., 1964. AHSB(A)-R-8
302. **Transport accidents; emergency organisation.** F. J. NEARY. p. 109~112 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug., 1964. AHSB(A)-R-8
303. **Experimental work at A.E.R.E. on the development of packaging for radioactive materials which will meet the I.A.E.A. regulations.** F. E. DIXON and A. H. PARTRIDGE. Apr. 1965. 12 p. AERE-R-4810
304. **Introduction to radioactivity and associated hazards with particular reference to transport,**

(b), medical aspects. K. P. DUNCAN. p. 19~24 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug., 1964. AHSB(A)-R-8

305. New developments in accident resistant shipping containers for radioactive materials. SISLER, J. A. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 141~186.

306. An analytical approach to study vehicle collisions. DOSHI, K. D. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 243~260.

5. 8. Safety (including Nuclear Safety)

310. Effect of neutron interaction on critical size. J. W. WEBSTER. Nov. 1954. 22 p. IDO-16201

311. Nuclear safety specifications for fuel element manufacturing processes. General Electric Co. 1955. 1s. HW-37952

312. Nuclear safety of vessels in arrays. N. KETZLACH. 1956. 1s. HW-41899

313. Safety of MTR fuel shipping boxes. H. L. McMURRY and B. L. HENSON. 1956. 1s. IDO-16360

314. Nuclear safety of iron-encased fuel elements. N. KETZLACH. 1951. 1s. HW-51423

315. Nuclear safety considerations in the handling and storage of reactor fuels. NORMAN KETZLACH. 15 p. fig. 8 p. Nuclear Engineering and Science Conference sponsored by Engineers Joint Council from March 17 to 21, 1958, at Chicago, Illinois. Preprint 29

316. PRTR fuel element nuclear safety. N. KETZLACH. 1959. 1s. HW-59786

317. Criticality in the HRT transfer vessel. S. JAYE and L. L. BENNETT. 1960. 1s. CF-60-7-81

318. Public safety aspects of radioisotopes distribution. G. R. NEWBERY. Atom (UKAEA), no. 43, p. 19-32 (May 1960)

307. Investigation of low-level, radioactive waste containers in an accident environment. RASTRELLI, L. U., MINOR, JOSEPH E. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 302~320. or CONF-650103-6

308. Dispersion of radioisotopes caused by transport fires. MORIYA, T., JIN, T. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965. SC-RR-65-98, p. 719~722.

309. A summary of incidents involving USAEC shipments of radioactive material, 1963~64. Apr. 1966. viii, 15 p. TID-16764 (Suppl. 2)

319. Nuclear safety guide, 1961. 42 p. TID-7016 (Rev. I)

320. The criticality aspects of transportation of fissile materials. E. R. WOODCOCK and H. C. PAXTON p. 401-434 of Progress in nuclear energy series IV; technology, engineering and safety, vol. 4. Oxford, Pergamon, 1961.

321. Criticality aspects of the transport of fissile materials; the technical basis of the proposed IAEA regulations. R. F. BARKER. p. 433-439 of Criticality control in chemical and metallurgical plant, Karlsruhe symposium, 1961.

322. Presentation of "criticality aspects of the transport of fissile materials". M. C. ERGINSOY. p. 441-455 of Criticality control in chemical and metallurgical plant, Karlsruhe symposium, 1961.

323. Lectures on criticality. E. D. PENDLEBURY, E. R. WOODCOCK, A. F. THOMAS, K. D. B. JOHNSON and C. M. NICHOLLS. 1961. 96 p. AHSE(S)-R-4

324. Nuclear safety criteria in use of RMDF transfer cask. NORMAN KETZLACH. 1961. 1s. NAA-SR-MEMO-6415

325. The development and evaluation of safety performance criteria for sealed radiation sources.

- CHARLES W. TOWNLEY, JOSEPH M. FACKELMANN, CHARLES L. SELANDER, ROBERT A. EWING and DUANE N. SUNDERMAN. Dec. 1961. 76, 12 p. BMI-1559
326. **The interpretation of wood density measurements in the criticality clearance of transport containers.** D. C. DOWSON, T. MURPHY, R. A. O'SULLIVAN and E. R. WOODCOCK. Feb. 1962. 8 p. AHSB (S)-R-36
327. **A safety review of the Oak Ridge critical experiments facility.** Dec. 1962. 2s. (54 p.) ORNL-TM-349
328. **Safety considerations in the transportation of highly radioactive materials; an operations research approach.** FERDINAND F. LEIMKUHLER. p. 31-44 of Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioactive Material. TID-7651
329. **Trucking of radioactive materials: safety vs. economy in highway transport.** FERDINAND F. LEIMKUHLER. June 1963. viii, 156 p. NYO-9773
330. **Safety aspects of spent fuel transport.** JOHN W. LANGHAAR, ALEXANDER E. AIKENS, JR., NELMAR J. RIGSTAD, HAROLD E. WALCHLI and LYLE L. ZAHN, JR. May 1964. 12 p. A/CONF. 28/P-279
331. **The safety testing program at ORNL.** KARL W. HAFF. *Transactions of the American Nuclear Society*, vol. 7, no. 1, p. 208-209 (1964)
332. **Liability and indemnity policy.** B. C. PEATEY. p. 81-87 of Proceedings of the Symposium on the Transport of Radioactive Materials. Aug., 1964, AHSB (A)-R-8
333. **U. S. Atomic Energy Commission AEC manual. Chapter 0529. Safety standards for the preparation of radioactive and fissile material for transportation.** p. 123-131 of Plutonium nitrate shipping packages. RFP-437
334. **Nuclear safety coordination within Oak Ridge Operations Facilities.** JOHNSON, WILEY A.; PRYOR, WILLIAM A. 1965. 1s. (20 p.) CONF-651103-2
335. **Safety aspects of transport of radioactive materials.** ZAHN, L. L. JR.; JUNKINS, R. L. Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 1-6.
336. **The elements of neutron interacting arrays.** NEWLON, C. E. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June, 1965 SC-RR-65-98, p. 663-677.
337. **Criticality safety evaluation of packages for the transport of fissile material.** SMITH, D. R. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials. June 1965, SC-RR-65-98, p. 679-698.

5. 9 Economy and Irradiated Fuel Shipping Cost

338. **Criteria for use in the estimation of the cost of shipping contaminated reactor fuels.** F. T. BINFORD. 1954. 1s. CF-54-5-229
339. **Economics of shipping spent nuclear fuel elements.** CHARLES E. DRYDEN. *Nucleonics*, vol. 14, no. 7, p. 77-80 (1956)
340. **An economic analysis of the domestic and overseas shipment of spent reactor fuel.** L. B. SHAPPERT. June 1963. 1s. (21 p.) ORNL-TM-590
341. **Irradiated fuel transport costs and maintenance costs in chemical reprocessing plant.** YVES SOUSSELIER. May 1964. 13 p. A/CONF. 28/P-64
342. **A computer code (CDC 1604A or IBM 7090) for calculating the cost of Shipping spent reactor fuels as a function of burnup, specific power, cooling time, fuel composition, and other variables.** ROYES SALMON. Aug. 1964. iv. 69 p. ORNL-3648

343. **Economics of irradiated fuel shipment.** SALMON, ROYES. Nuclear engineering, part 14. Chemical engineering progress symposium series, no. 56, vol. 61 (1965), p. 7-15.
344. **Fuel shipping costs.** R. SALMON. p. 133-145 of A comparative evaluation of advanced converters. Jan. 1965. ORNL-3686
345. **Estimation of fuel-shipping costs for nuclear power cost-evaluation purposes.** SALMON, ROYES. Mar. 1966. 2s. (67 p.) ORNL-3943
346. **Shipping cost comparisons for lead-, steel-, and uranium-shielded casks.** SHAPPERT, L. B.; SALMON, ROYES. Mar. 1966. 20 p. ORNL-3918

5. 10 Handling and Irradiated Fuel Storage

347. **Uranium handling for CVR experiments.** R. J. CREAGAN and A. F. HENRY. 1952. 1s. WAPD-RM-145
348. **The handling and dispensing of chemically processed radioactive isotopes.** F. HUDSWELL, G. A. NEATHWAY, B. R. PAYNE, J. A. PAYNE and P. SCARGILL. 1954. 1s. AERE-I/R-1360
349. **Isotopes Division annual report for 1957.** H. E. SEAGREN, E. E. BEAUCHAMP, J. H. GILLETTE, E. LAMB and P. S. BAKER. v, 59 p. ORNL-2492
350. **HRT source handling facilities.** C. A. BURCHSTED and C. L. SEGASER. 1956. 1s. CF-56-11-58
351. **Some criteria for safe storage of U-235 fuel.** C. V. LARRICK. 1956. 1s. KAPL-M-CVL-4 (Del.)
352. **On the construction of some tools and manipulators which are used when working with radioactive substances.** H. B. САМОХВАНОВ. Атомная Энергия, том 3, p. 368-370 (1957)
353. **Tritium handling system.** B. J. MASSEY. 1957. 1s. ONNL-2238
354. **Safe handling of radioisotopes.** Vienna, 1958. 99 p. Safety series, no. 1
355. **Safe handling of radioisotopes health physics addendum.** G. J. APPLETON and P. N. KRISHNAMOORTHY. Vienna, IAEA, 1960. 120 p. Safety series, no. 2
356. **Design and engineering problems of the spent fuel element irradiation ponds at A. E. R. E., Harwell.** J. W. MARK. 1960. 15 p. fig. 8 p. plate. 9 p. AERE-R-3391
357. **Problems de manipulation et de transport.** J. POMAROLA et J. SAVOUYAUD. 1960. 11, 4 p. fig. 1 p. CEA-1434
358. **ICPP facilities for receiving, storage and mechanical treatment.** A. L. AYERS and C. B. LEEK. p. 99-112 of Proceedings of the AEC Symposium for Chemical Processing of Irradiated Fuels from Power, Test, and Research Reactors, Richland, Washington, Oct. 20 and 21, 1959. Jan. 1960. TID-7583
359. **Mechanical handling of Chalk River and PRDC fuel.** J. W. LANGHAAR. p. 174-186 of Proceedings of the AEC Symposium for Chemical Processing of Irradiated Fuels from Power, Test, and Research Reactors, Richland, Washington, Oct. 20 and 21, 1959. Jan. 1960. TID-7583
360. **Preliminary technical manual. Loading, shipping and testing procedures for task 2 isotopic powered thermoelectric generator.** M. J. REILLY. Mar. 1960. 1 vol. (various pagings.) MND-P-2316
361. **The packaging, transport and related handling of radioactive materials.** F. R. FARMER. Vienna, IAEA, 1961. 64 p. Review series, no. 12
362. **Safety considerations in aqueous reprocessing plant operations.** W. G. MORRISON. 1961. 2s. IDO-14550
363. **Recommendations on safety principles and practice, 1962. Safe handling of zirconium metal and zirconium alloys.** 7 p. RSPP-20 (Issue 1)
364. **Recommendations on the safe handling of zirconium metal and zirconium alloys.** G. H. BULMER and A. QUINTON. 1963. 47 p. AHSB

(S)-R-59

365. SRE reactor fuel handling equipment. E. O. DRYER, E. L. BROWN and W. J. FREEDE. Feb. 1963. v, 34 p. NAA-SR-5599

366. New techniques for out-of-plant packaging and internal handling containers. C. R. RIGGS. Aug. 1964. 1s. (23 p.) Y-OA-21

367. JRR-2 spent fuel storage facility (and its handling equipments).

S. SAWAI [and others] Oct. 1964. 55 p. JAERI-memo-1743

368. JRR-2 spent fuel storage facility; present status and its history. S. SAWAI. *Nuclear Engineering (Japan)* vol. 11, no. 6, p. 49-55 (June 1965)

369. Handling and measurement of plutonium shipments. BYRNE, J. T. Oct. 1965. vi, 32 p. RFP-508

5. 11. Bibliography and Symposium

370. Proceedings of the 1958 Atomic Energy Commission and contractor Safety and Fire Protection Conference; held at Atomic Energy Commission, Headquarters Building, Germantown, Maryland, June 24-25, 1958. May 1959. 121 p. TID-7569

371. Bibliography. p. 75-145 of Structural integrity of shipping containers for radioactive materials, part I. NYO-9859

372. Bibliography. p. 146-154 of Report of working meeting on shipping container testing programs at the Johns Hopkins University, May 2-3, 1962. TID-7635

373. Bibliography on transportation of radioactive materials. N. WRIGHT. Sept. 1962. 22 p. TRG-Inf. Ser.-239 (R)

374. Summary report of AEC Symposium on Packaging and Regulatory Standards for Shipping Radioac-

tive Material, held in Germantown, Maryland, December 3-5, 1962. vii, 416 p. TID-7651

375. Symposium on the Transport of Radioactive Materials. Harwell, 9th and 10th September, 1963. Jan. 1964. 101 p. AHSB (A) R-6

376. Proceedings of the Symposium on the Transport of Radioactive Materials, Durley Hall, Bourne-mouth, 13th-15th April, 1964. F. J. NEARY. Aug. 1964. 126 p. AHSB (A)-R-8

377. Proceedings of the International Symposium for Packaging and Transportation of Radioactive Materials, held at Albuquerque, New Mexico, January 12-15, 1965. June 1965. 754 p. SC-RR-65-98

378. Nuclear Engineering, Part 14 Chemical Engineering Progress Symposium Series, No. 56, Vol. 61 1965

5. 12. Miscellaneous

379. Production of iodine-132. W. E. WINSCH, L. G. STANG, JR. and W. D. TUCKER. *Nuclear Engineering*, vol. 8, no. 3, p. 14-18 (1951)

380. Radioisotope production and process development annual report for 1955. A. F. RUPP, E. E. BEAUCHAMP, J. H. GILLETTE and E. J. WITKOWSKI. May 1956. vi, 16 p. ORNL-2064

381. Joint design and welding of Boral. F. J. MALONY. 1957. 1s. SCDC-706

382. Welding builds "transportation casks" for radioac-

tive materials. P. E. WOODWARD. *Welding Journal*, vol. 37, p. 597 (1958)

383. Über die Bemessung strahlensicherer Isotopenbehälter. ROLF PLESCH. *Atompraxis*, vol. 5, p. 49-53 (1959)

384. Television monitor-SRE fuel coffin. D. K. DARLEY and J. D. SMITH. 1959. 1s. NAA-SR-MEMO-3844

385. Nuclear merchant ship reactor; general guided to shipping requirements for activated internals and

- components. W. R. SMITH. 1960. 2s. BAW-1204
386. Determination of the S. S. N. M. content of the Davison Chemical Company, Erwin, Tennessee, Dec. 20, 1960. 1961. 1s. CF-61-1-33
387. 一欠番—
388. 一欠番—
389. Prüfung umschlossener radioaktiver Stoffe auf Dichtigkeit. F. KUHN. *Atompraxis*, vol. 7, p. 16-17 (1961)
390. Chemical effluents technology. p. 158-237 of Research and development activities in the radiological sciences, physical sciences portion January through December, 1961. R. L. JUNKINS and J. E. BROWN. Jan. 1962. HW-73337
391. Transportbehälter für bestrahlte Brennstoffelemente. W. GEIPEL. *Kerntechnik*, Jg. 4, Heft. 9, p. 402-410 (1962)
392. Faßtransporteinrichtungen bei Entaktivierungsanlagen. H. WITTE. *Kerntechnik*, Jg. 4, Heft. 7, p. 288-290 (1962)
393. A basic toxicity classification of radionuclides; report of joint study of a group of consultants to the IAEA. Vienna, IAEA, 1963. 39 p. Technical reports series, no. 15
394. The efficient importation and distribution of radioisotopes. Vienna, IAEA, 1963. 21 p. Technical reports series no. 19
395. Process instrumentation development. p. 47-62 of Instrumentation and Controls Division annual progress report for period ending September 1, 1962. Feb. 1963. ORNL-3378
396. Role of Spent Fuel in Nuclear Power. M. TAKAHASHI, *Nuclear Power (Japan)*. Vol. 8, No. 1. p. 10-15, 1964
397. Spent Fuel and Its Reprocessing Plant. T. SAKAMOTO. *Nuclear Power (Japan)*, Vol. 8, No. 1, p. 16-23, 1964
398. Final report, the cesium-137 power program. May 1964. 1 vol. (various pagings.) SAN-366-1
399. Large-scale fission-product recovery. R. E. TOMLINSON, B. F. JUDSON and L. L. ZAHN. May 1964. 14 p. A/CONF. 28/P-252
400. Reactor evaluation studies. p. 249-255 of Chemical Technology Division annual progress report for period ending May 31, 1964. Oct. 1964. 7 p. ORNL-3627
401. Principles of Nuclear Reactor Engineering. S. GLASSTONE. p. 119, 1956. D. Van Nostrand Comp. Inc.
402. Solar Energy Research. F. DANIEL, *et al.*, 1955, University of Wisconsin Press.
403. Reactor Handbook. "Shielding". Second Edition, 3-Part B, E. P. BLIZARD, *et al.*, p. 27-28, 1962, Interscience Publishers
404. Reactor Shielding Design Manual. Rockwell. p. 35, 1956, McGraw-Hill Book Comp.
405. Nuclear Engineering Handbook. ETHERINGTON, *et al.*, p. 7-16~7-17, 1958, McGraw-Hill Book Comp.