

JAERI-M
85-101

NEANDC (J) 114/U
INDC (JPN) 100/L

**EVALUATION OF NEUTRON NUCLEAR DATA
OF NATURAL NICKEL AND ITS ISOTOPES FOR
JENDL-2**

July 1985

Yasuyuki KIKUCHI and Nobuo SEKINE*

日本原子力研究所
Japan Atomic Energy Research Institute

JAERI-M レポートは、日本原子力研究所が不定期に公刊している研究報告書です。

入手の問合せは、日本原子力研究所技術情報部情報資料課（〒319-11茨城県那珂郡東海村）
あて、お申しこしください。なお、このほかに財団法人原子力弘済会資料センター（〒319-11茨城
県那珂郡東海村日本原子力研究所内）で複写による実費頒布をおこなっております。

JAERI-M reports are issued irregularly.

Inquiries about availability of the reports should be addressed to Information Division, Department
of Technical Information, Japan Atomic Energy Research Institute, Tokai-mura, Naka-gun,
Ibaraki-ken 319-11, Japan.

© Japan Atomic Energy Research Institute, 1985

編集兼発行 日本原子力研究所
印 刷 日立高速印刷株式会社

Evaluation of Neutron Nuclear Data
of Natural Nickel and its Isotopes for JENDL-2

Yasuyuki KIKUCHI⁺ and Nobuo SEKINE^{*}

Department of Physics

Tokai Research Establishment, JAERI

(Received June 21, 1985)

Neutron nuclear data of natural nickel and its isotopes have been evaluated. Evaluated are the total, elastic and inelastic scattering, capture, ($n,2n$), ($n,3n$), (n,p), (n,α), ($n,n'p$) and ($n,n'\alpha$) reaction cross sections, the resonance parameters, the angular and energy distributions of secondary neutrons in the energy range from 10^{-5} eV to 20 MeV. The evaluation has been made on the basis of recently measured data with the aid of the spherical optical model and statistical model. The results of the benchmark tests of JENDL-1 have been also taken into consideration. Special care has been taken on the background cross sections in the resonance region, the remaining resonance structure in the unresolved resonance region up to a few MeV, and grouping of the inelastic scattering levels in the natural nickel file. The problems left for future work are also discussed. The results of the present evaluation were adopted in JENDL-2.

Keywords: Natural Nickel, Isotopic Nickel, Evaluation, JENDL-2,

Background Cross Section, Resonance Structure,
Level Grouping, Threshold Reactions

+ Office of Planning

* Research student from Faculty of Science, Tohoku University.

Present Address: Hitachi Engineering Co., Ltd, Hitachi, Ibaraki 316

J E N D L - 2 のための天然および同位体ニッケルの中性子核データ評価

日本原子力研究所東海研究所物理部

菊池 康之⁺・関根 信雄^{*}

(1985年6月21日受理)

天然および同位体ニッケルの中性子核データの評価を行った。評価した量は 10^{-5} eV から 20 MeV にわたる全断面積、弾性および非弾性散乱、捕獲、(n, 2n), (n, 3n), (n, p), (n, α), (n, n' p), (n, n' α) 反応の各断面積、共鳴パラメータ、二次中性子の角度及びエネルギー分布である。評価は球型光学模型や統計模型を利用しつつ、最近の実験データに基づいて行った。J E N D L - 1 のベンチマークテストの結果も考慮に入れた。特に注意を払った点は、共鳴領域のバックグラウンド断面積、数 MeV 以下の非分離共鳴領域の共鳴構造、天然ニッケルファイルの非弾性散乱レベルのグループ化である。将来に残された問題も議論した。今回の評価結果は J E N D L - 2 に採用されている。

+ 企画室

* 核融合特研生（東北大学理学部），現所属：日立エンジニアリング（株）

Contents

1. Introduction	1
2. Isotopic Abundances, Masses and Q-values	2
3. Thermal Cross Sections	3
4. Resonance Region	3
4.1 Resolved Resonance Parameters	3
4.2 Background Cross Sections	4
4.3 Resonance Integral	4
5. Cross Sections above Resonance Region	4
5.1 Total Cross Section and Optical Potential	4
5.2 ($n,2n$), ($n,3n$), ($n,n'\alpha$), ($n,n'p$), (n,p) and (n,α) Reaction Cross Sections	5
5.3 Capture Cross Section	8
5.4 Elastic and Inelastic Scattering Cross Sections	8
6. Other Quantities	10
6.1 Angular Distributions of Emitted Neutrons	10
6.2 Energy Distributions of Emitted Neutrons	10
7. Discussion	11
7.1 Direct and Semi-direct Reactions	11
7.2 Threshold Reactions	12
8. Conclusion	13
Acknowledgment	14
References	15
Appendix	177

目 次

1. 序 論	1
2. 同位体存在比, 質量およびQ-値	2
3. 熱中性子断面積	3
4. 共鳴領域	3
4.1 分離共鳴パラメータ	3
4.2 バックグラウンド断面積	4
4.3 共鳴積分	4
5. 共鳴領域より上の断面積	4
5.1 全断面積と光学模型ポテンシャル	4
5.2 $(n, 2n)$, $(n, 3n)$, $(n, n'\alpha)$, $(n, n'p)$, (n, p) および (n, α) 反応断面積	5
5.3 捕獲断面積	8
5.4 弹性および非弾性散乱断面積	8
6. その他の物理量	10
6.1 放出中性子の角度分布	10
6.2 放出中性子のエネルギー分布	10
7. 議 論	11
7.1 直接および半直接過程反応	11
7.2 しきい反応	12
8. 結 論	13
謝 辞	14
参考文献	15
附 錄	177

1. Introduction

Neutron nuclear data of nickel are much required, because nickel is a main component of stainless steel and some threshold reactions of its isotopes are important for neutron dosimetry. Various evaluations have so far been made¹⁻⁸⁾. Every large evaluated nuclear data library contains the data of nickel. In spite of this, there still remain considerable discrepancies among the evaluated data.

For the first version of Japanese Evaluated Nuclear Data Library (JENDL-1)⁹⁾, a new evaluation of nickel data was made by Kikuchi et al.¹⁰⁾ in 1974. It was pointed out, however, through various benchmark tests^{11,12)} of JENDL-1 that the following drawbacks existed in the evaluated data of structural materials including nickel:

- 1) The total and elastic scattering cross sections are overestimated considerably in the energy region above resolved resonances up to several MeV. This causes considerable underestimate of the diffusion coefficients.
- 2) As to the inelastic scattering to the discrete levels, the natural nickel file contains only a few levels of main isotopes (⁵⁸Ni and ⁶⁰Ni) and many low lying levels of a minor odd-mass isotope (⁶¹Ni).
- 3) As to natural nickel, the capture cross section is overestimated above several hundred keV and the inelastic scattering cross section is underestimated.
- 4) As to the threshold reactions, the evaluation is rather rough except for some important ones.

Considering such a situation, a complete reevaluation of structural material nuclear data was planned for JENDL-2 in 1976. Completion of the entire compilation for JENDL-2 was scheduled at the end of 1981. At the early stage of compilation, however, the highest priority was put to

evaluation of the most important nuclides for fast reactors: ^{235}U , ^{238}U , ^{239}Pu , ^{240}Pu , ^{241}Pu , Cr, Fe and Ni, whose cross sections mainly determine characteristics of a typical fast reactor. This decision was made responding to an urgent request to use JENDL-2 for analyses in the JUPITOR project¹³⁾, joint USA-Japan mock-up experiments of large fast reactors using ZPPR facility.

The evaluation of the eight nuclides was completed in November 1979. Since then a combined library, called JENDL-2B library, consisting of JENDL-2 for the eight nuclides and of JENDL-1 for the others has been widely used for fast reactor analyses in Japan. Various benchmark tests have also been made and satisfactory results have been obtained^{14,15)}.

As to nickel, only the resonance parameters, cross sections and angular distributions (files 2,3 and 4 in ENDF/B format) of natural nickel were supplied to JENDL-2B with the data of ENDF/B-IV for the energy distributions (file 5). After that the data of isotopes were evaluated. The energy distributions of natural nickel were also evaluated and the file 5 was replaced by the new data. Final data were released in December 1982. As the evaluation of natural nickel was thus made before the evaluation of its isotopes, some inconsistencies remain inevitably between the natural nickel data and its isotope data.

The essence of the present evaluation was already published¹⁶⁾. Hence this report is intended to provide more complete information for users of JENDL-2. Many figures and tables are provided but less discussion on the evaluation method is given in this report. The status of the presently evaluated quantities are given in Table 1.

2. Isotopic Abundances, Masses and Q-values

The isotopic abundances were taken from the recommendation by

evaluation of the most important nuclides for fast reactors: ^{235}U , ^{238}U , ^{239}Pu , ^{240}Pu , ^{241}Pu , Cr, Fe and Ni, whose cross sections mainly determine characteristics of a typical fast reactor. This decision was made responding to an urgent request to use JENDL-2 for analyses in the JUPITOR project¹³⁾, joint USA-Japan mock-up experiments of large fast reactors using ZPPR facility.

The evaluation of the eight nuclides was completed in November 1979. Since then a combined library, called JENDL-2B library, consisting of JENDL-2 for the eight nuclides and of JENDL-1 for the others has been widely used for fast reactor analyses in Japan. Various benchmark tests have also been made and satisfactory results have been obtained^{14,15)}.

As to nickel, only the resonance parameters, cross sections and angular distributions (files 2,3 and 4 in ENDF/B format) of natural nickel were supplied to JENDL-2B with the data of ENDF/B-IV for the energy distributions (file 5). After that the data of isotopes were evaluated. The energy distributions of natural nickel were also evaluated and the file 5 was replaced by the new data. Final data were released in December 1982. As the evaluation of natural nickel was thus made before the evaluation of its isotopes, some inconsistencies remain inevitably between the natural nickel data and its isotope data.

The essence of the present evaluation was already published¹⁶⁾. Hence this report is intended to provide more complete information for users of JENDL-2. Many figures and tables are provided but less discussion on the evaluation method is given in this report. The status of the presently evaluated quantities are given in Table 1.

2. Isotopic Abundances, Masses and Q-values

The isotopic abundances were taken from the recommendation by

Moxon³⁾. The exact masses and the Q-values of considered threshold reactions were taken from the compilation of Wapstra and Boss¹⁷⁾. They are given in Table 2.

3. Thermal Cross Sections

The data recommended in BNL-325, 3rd edition¹⁸⁾ were adopted as the 2200m/s values. They are given in Table 3 with the calculated values from the present resonance parameters.

4. Resonance Region

4.1 Resolved Resonance Parameters

Resonance parameters were evaluated for each isotope on the basis of various measurements listed in Table 4 by taking account of the other evaluation such as ENDF/B-IV, BNL-325, 3rd edition¹⁸⁾ and Moxon's work³⁾. For levels whose radiation widths Γ_γ are not known, $\Gamma_\gamma = 2$ eV was assumed for s-wave resonances and $\Gamma_\gamma = 1$ eV for p-wave ones of all the isotopes according to the recommendation by Moxon³⁾. The values of effective scattering radius were mainly taken from BNL-325, 3rd edition¹⁸⁾.

In order to reproduce the adopted thermal cross sections, the following adjustment was made: Two negative resonances were added for ⁵⁸Ni and a negative resonance for each of ⁶⁰Ni and ⁶¹Ni. Parameters of the first positive resonance at 4.6 keV were adjusted for ⁶²Ni. As for natural nickel, a negative resonance was added to ⁵⁸Ni at -28.5 keV without considering the thermal cross section data of each isotope. Hence there remains the inconsistency in treatment of the negative or lowest positive resonances and in the thermal cross sections between the natural nickel and isotopic nickel data. Table 5 gives the status of resonance parameters together with the values of the effective scattering radius. Tables 6-10 gives the presently adopted resonance parameters

Moxon³⁾. The exact masses and the Q-values of considered threshold reactions were taken from the compilation of Wapstra and Boss¹⁷⁾. They are given in Table 2.

3. Thermal Cross Sections

The data recommended in BNL-325, 3rd edition¹⁸⁾ were adopted as the 2200m/s values. They are given in Table 3 with the calculated values from the present resonance parameters.

4. Resonance Region

4.1 Resolved Resonance Parameters

Resonance parameters were evaluated for each isotope on the basis of various measurements listed in Table 4 by taking account of the other evaluation such as ENDF/B-IV, BNL-325, 3rd edition¹⁸⁾ and Moxon's work³⁾. For levels whose radiation widths Γ_γ are not known, $\Gamma_\gamma = 2$ eV was assumed for s-wave resonances and $\Gamma_\gamma = 1$ eV for p-wave ones of all the isotopes according to the recommendation by Moxon³⁾. The values of effective scattering radius were mainly taken from BNL-325, 3rd edition¹⁸⁾.

In order to reproduce the adopted thermal cross sections, the following adjustment was made: Two negative resonances were added for ⁵⁸Ni and a negative resonance for each of ⁶⁰Ni and ⁶¹Ni. Parameters of the first positive resonance at 4.6 keV were adjusted for ⁶²Ni. As in natural nickel, a negative resonance was added to ⁵⁸Ni at -28.5 keV without considering the thermal cross section data of each isotope. Hence there remains the inconsistency in treatment of the negative or lowest positive resonances and in the thermal cross sections between the natural nickel and isotopic nickel data. Table 5 gives the status of resonance parameters together with the values of the effective scattering radius. Tables 6-10 gives the presently adopted resonance parameters

Moxon³⁾. The exact masses and the Q-values of considered threshold reactions were taken from the compilation of Wapstra and Boss¹⁷⁾. They are given in Table 2.

3. Thermal Cross Sections

The data recommended in BNL-325, 3rd edition¹⁸⁾ were adopted as the 2200m/s values. They are given in Table 3 with the calculated values from the present resonance parameters.

4. Resonance Region

4.1 Resolved Resonance Parameters

Resonance parameters were evaluated for each isotope on the basis of various measurements listed in Table 4 by taking account of the other evaluation such as ENDF/B-IV, BNL-325, 3rd edition¹⁸⁾ and Moxon's work³⁾. For levels whose radiation widths Γ_γ are not known, $\Gamma_\gamma = 2$ eV was assumed for s-wave resonances and $\Gamma_\gamma = 1$ eV for p-wave ones of all the isotopes according to the recommendation by Moxon³⁾. The values of effective scattering radius were mainly taken from BNL-325, 3rd edition¹⁸⁾.

In order to reproduce the adopted thermal cross sections, the following adjustment was made: Two negative resonances were added for ⁵⁸Ni and a negative resonance for each of ⁶⁰Ni and ⁶¹Ni. Parameters of the first positive resonance at 4.6 keV were adjusted for ⁶²Ni. As for natural nickel, a negative resonance was added to ⁵⁸Ni at -28.5 keV without considering the thermal cross section data of each isotope. Hence there remains the inconsistency in treatment of the negative or lowest positive resonances and in the thermal cross sections between the natural nickel and isotopic nickel data. Table 5 gives the status of resonance parameters together with the values of the effective scattering radius. Tables 6-10 gives the presently adopted resonance parameters

with the experimentally deduced values as well as various evaluated parameters.

4.2 Background Cross Sections

The resonance region is set up to 600 keV except for ^{61}Ni . The present resonance parameters fail to reproduce the measured total and capture cross sections particularly in the energy region above a few tens of keV. The reason of this disagreement was investigated, and the disagreement was corrected by applying the positive or negative background cross sections. We will detail this problem in Appendix 1. Figure 1 shows the total and capture resonance cross sections of natural nickel with the measured data as well as the data of JENDL-1 and ENDF/B-IV. The resonance cross sections of the isotopic nickels are shown in Figs. 2-6.

4.3 Resonance Integral

The resonance capture integrals (the cut-off energy of 0.5 eV) calculated from the present resonance parameters and the background cross section are compared in Table 11 with the recommended values of BNL-325, 3rd edition¹⁸⁾. They agree with each other for natural nickel and the main isotopes but are discrepant for ^{61}Ni and ^{64}Ni .

5. Cross Sections above Resonance Region

5.1 Total Cross Section and Optical Potential

As was pointed out in benchmark tests of JENDL-1^{11,12)}, the remaining resonance structure in the unresolved resonance region up to a few MeV has an important role for self-shielding effects. In the present evaluation, we traced the resonance structure in high resolution measurements by Cierjacks et al.²⁷⁾ up to 3 MeV for natural nickel. This was made by the eye-guide method with Neutron Data Evaluation

with the experimentally deduced values as well as various evaluated parameters.

4.2 Background Cross Sections

The resonance region is set up to 600 keV except for ^{61}Ni . The present resonance parameters fail to reproduce the measured total and capture cross sections particularly in the energy region above a few tens of keV. The reason of this disagreement was investigated, and the disagreement was corrected by applying the positive or negative background cross sections. We will detail this problem in Appendix 1. Figure 1 shows the total and capture resonance cross sections of natural nickel with the measured data as well as the data of JENDL-1 and ENDF/B-IV. The resonance cross sections of the isotopic nickels are shown in Figs. 2-6.

4.3 Resonance Integral

The resonance capture integrals (the cut-off energy of 0.5 eV) calculated from the present resonance parameters and the background cross section are compared in Table 11 with the recommended values of BNL-325, 3rd edition¹⁸⁾. They agree with each other for natural nickel and the main isotopes but are discrepant for ^{61}Ni and ^{64}Ni .

5. Cross Sections above Resonance Region

5.1 Total Cross Section and Optical Potential

As was pointed out in benchmark tests of JENDL-1^{11,12)}, the remaining resonance structure in the unresolved resonance region up to a few MeV has an important role for self-shielding effects. In the present evaluation, we traced the resonance structure in high resolution measurements by Cierjacks et al.²⁷⁾ up to 3 MeV for natural nickel. This was made by the eye-guide method with Neutron Data Evaluation

System(NDES)²⁸⁾. Above 3 MeV, the evaluated data were obtained by smoothing the data of Cierjacks et al. The evaluated total cross section is shown in Fig. 7.

For each isotope, on the other hand, no high resolution data exist, and the evaluation was based on the optical model calculation. The potential parameters were obtained by Kawai²⁹⁾ so as to reproduce the total cross section of natural nickel by taking account of the systematic trends among neighboring nuclides such as Ti, V, Cr, Mn, Fe, Co and Cu. The potential parameters are:

$$\begin{aligned} V &= 51.33 - 0.331 \times E_n \quad (\text{MeV}) \\ W_s &= 8.068 + 0.112 \times E_n \quad (\text{MeV}) \\ V_{so} &= 7.00 \quad (\text{MeV}) \\ r_o &= r_{so} = 1.24 \quad (\text{fm}) \\ r_s &= 1.4 \quad (\text{fm}) \\ a &= a_{so} = 0.541 \quad (\text{fm}) \\ b &= 0.4 \quad (\text{fm}). \end{aligned}$$

This set of parameters is applied to all the isotopes. The calculated total cross sections of ^{58}Ni and ^{60}Ni are compared with the measured data in Figs. 8 and 9, respectively.

5.2 $(n,2n)$, $(n,3n)$, $(n,n'\alpha)$, $(n,n'p)$, (n,p) and (n,α) Reaction Cross Sections

These cross sections were evaluated for each isotope as follows.

$(n,2n)$ reaction

For ^{58}Ni the data of JENDL-1, which were evaluated on the basis of numerous experimental data, were adopted and extended up to 20 MeV. For the other isotopes, on the other hand, experimental data are very scarce. Hence its shape was calculated with the evaporation model based

on Pearlstein's approximation³⁰⁾, and the cross section value was normalized at 15 MeV to the calculated value by Pearlstein with his semi-empirical formula³¹⁾; 510 mb for ^{60}Ni , 870 mb for ^{61}Ni , 910 mb for ^{62}Ni and 1200 mb for ^{64}Ni .

The ($n,2n$) reaction cross sections of natural and isotopic nickels are shown in Figs. 10-15 with the measured data as well as the other evaluated data. The present data of natural nickel agree fairly well with the measured data by Auchampaugh et al.³²⁾, suggesting the reliability of the present evaluation for the other isotopes than ^{58}Ni .

($n,3n$) reaction

The ($n,3n$) reaction channel is closed below 20 MeV for ^{58}Ni and ^{60}Ni , and is nearly closed for ^{61}Ni and ^{62}Ni . Hence we ignored this cross section for these nuclides. For ^{64}Ni , the shape was calculated with Pearlstein's approximation, and the same normalization factor as the ($n,2n$) reaction was applied to the ($n,3n$) reaction cross section. The ($n,3n$) reaction cross section of ^{64}Ni is also shown in Fig. 15.

($n,n'\alpha$) reaction

The ($n,n'\alpha$) reaction cross section was evaluated only for ^{58}Ni . Its shape was estimated on the analogy of the ^{65}Cu ($n,n'\alpha$) reaction cross section by considering the difference of Q-values. The $^{65}\text{Cu}(n,n'\alpha)$ reaction was selected because of its numerous experimental data and its similar Z and A values to those of ^{58}Ni , and its shape was obtained by the eye-guide method. The absolute value of ^{58}Ni was obtained by normalizing the curve to the data of Seebeck and Bormann³³⁾ (30 mb at 14 MeV). The ($n,n'\alpha$) reaction cross section of ^{58}Ni is shown in Fig. 16.

(n,n'p) reaction

The shape of the (n,n'p) reaction was estimated by taking account of the evaporation model calculation with the GROGI code³⁴⁾, and the absolute value was obtained by considering various measured data near 14 MeV; 480 mb at 14 MeV for ^{58}Ni , 60 mb at 14 MeV for ^{60}Ni , 13 mb at 14.7 MeV for ^{61}Ni and 6 mb at 14.5 MeV for ^{62}Ni . This cross section was ignored for ^{64}Ni . The (n,n' α) reaction cross sections are shown in Figs. 17-20.

(n,p) reaction

For ^{58}Ni and ^{60}Ni , the data of JENDL-1, which were evaluated on the basis of numerous experimental data, were adopted and extended up to 20 MeV. For the other isotopes, the shape was estimated on the analogy of the ^{60}Ni (n,p) reaction cross section by considering the difference of Q-values. The absolute value was obtained by considering various measured data at 15 MeV; 90 mb for ^{61}Ni , 22 mb for ^{62}Ni and 4.5 mb for ^{64}Ni .

The (n,p) reaction cross sections are shown in Figs. 21-26, with the measured data.

(n, α) reaction

For each isotope, the cross section shape was estimated on the analogy of the ^{59}Co (n, α) reaction cross section by shifting the energy scale corresponding to the Q-value difference. The $^{59}\text{Co}(n,\alpha)$ -reaction was selected, because of its well-known cross section as a reaction for neutron dosimetry and of its similar Z and A values to those of nickel isotopes. The absolute values were obtained by some measured data; 130 mb at 14 MeV for ^{58}Ni and 21 mb at 14.5 MeV for ^{62}Ni , or by the calculated value by Pearlstein³¹⁾ with his semi-empirical formula; 56 mb

at 15 MeV for ^{60}Ni , 34 mb at 15 MeV for ^{61}Ni and 6.9 mb at 15 MeV for ^{64}Ni . The (n,α) reaction cross sections are shown in Figs. 27-32.

5.3 Capture Cross Section

The capture, elastic and inelastic scattering cross sections were calculated with the statistical model code CASTHY³⁵⁾ for each isotope. The $(n,2n)$, $(n,3n)$, $(n,n'p)$, $(n,n'\alpha)$, (n,p) and (n,α) reactions were taken into account as the competing processes. The level fluctuation was considered.

The γ -ray strength functions were obtained so that the calculated capture cross section might reproduce the experimental data of Ernst et al.³⁶⁾ for ^{58}Ni , ^{60}Ni and ^{61}Ni and of Beer and Spencer²⁶⁾ for ^{62}Ni and ^{64}Ni . The obtained γ -ray strength functions are given in Table 12. For natural nickel, the γ -ray strength functions were adjusted so that the calculated capture cross section might reproduce the experimental data of Gayther et al.³⁷⁾; 9.6 mb at 450 keV. This adjustment was made without considering the capture data of each isotope and the obtained γ -ray strength functions are also given in Table 12. Though the γ -ray strength functions used in the evaluation of natural nickel are different from those used for the isotopes as seen in Table 12, the calculated cross sections of natural nickel agree with those constructed from the isotopic nickel data. The Berman³⁸⁾ type giant resonance profile function was adopted. The parameters of ^{60}Ni were used for all the nickel-isotopes and are given in Table 13. The capture cross sections of natural and isotope nickels are shown in Figs. 33-38 with the measured data.

5.4 Elastic and Inelastic Scattering Cross Sections

The inelastic scattering cross sections were calculated for each

isotope with CASTHY code. The level schemes were taken from Table of Isotopes, 7th edition³⁹⁾ and are given in Table 14. The level density parameters were evaluated by Yoshida⁴⁰⁾ from the resonance level spacing and the staircase plotting of low lying levels by taking account of the systematics among neighboring nuclei. They are given in Table 15.

The direct process was considered only for the inelastic scattering to the first excited state of the even-mass isotopes. They were evaluated on the basis of the measured data up to 7 MeV. Above 7 MeV, the direct processes were calculated with DWBA and added to the compound components calculated with CASTHY. The β_2 -values in the DWBA calculation are 0.187 for ^{58}Ni , 0.211 for ^{60}Ni , 0.193 for ^{62}Ni and 0.192 for ^{64}Ni . The inelastic scattering cross sections to the first levels of even-mass isotopes are shown in Figs. 39-42.

Special care is required in constructing the natural nickel data from the isotope data. Each isotope file has about 20 discrete inelastic levels. In the ENDF/B format, however, only 40 discrete inelastic levels are allowed. Hence all the levels of isotopes cannot be adopted in the natural nickel file. In JENDL-1, the lowest 40 levels were adopted as discrete levels and the other levels were added to the continuum levels. As a result, JENDL-1 contains only a few levels of the main isotopes and many low-lying levels of the minor odd-mass isotope. The shielding benchmark tests of iron¹²⁾, however, suggested that this treatment was inadequate. In JENDL-2, we combined some levels, whose Q-values are similar, to one level. Table 14 also shows the level grouping in the natural nickel file. The total inelastic scattering cross section of natural nickel is shown in Fig. 46. The present values agree well with the measured data of Broder et al.⁴²⁾ below 5 MeV but look underestimated above 10 MeV. This problem will be discussed later. The total inelastic scattering cross sections of the

isotopic nickels are shown in Figs. 43-48.

Finally the elastic scattering cross section was adjusted by subtracting all the other partial cross sections from the total cross section.

6. Other Quantities

6.1 Angular Distributions of Emitted Neutrons

The angular distribution of the elastically scattered neutrons was calculated with the optical model for each isotope, and that of natural nickel was obtained by averaging the data of isotopes by using $a_i \sigma_{si}$ as weights (a_i and σ_{si} denote the abundance and elastic scattering cross section of isotope i , respectively). For inelastic scattering to the discrete levels, the angular distributions were also calculated with the Hauser-Feshbach model for each isotope, and isotropic scattering in the center-of-mass system was assumed for natural nickel. Isotropic scattering in the laboratory system was assumed for the inelastic scattering to the continuum levels, and for the $(n,2n)$, $(n,3n)$, $(n,n'\alpha)$ and $(n,n'p)$ reactions.

6.2 Energy Distributions of Emitted Neutrons

The energy distributions of emitted neutrons were evaluated for each isotope as follows: The simple evaporation spectrum was assumed for the inelastic scattering to the continuum states (MT=91). The same nuclear temperature was also applied to the $(n,n'\alpha)$ and $(n,n'p)$ reactions. As to the $(n,2n)$ and $(n,3n)$ reactions, the successive evaporation model⁴³⁾ was assumed: The first neutron evaporates leaving the residual nucleus in an excited state higher than the neutron separation energy, and the second neutron evaporates from the excited state corresponding to the average energy of the first neutron, and so on.

isotopic nickels are shown in Figs. 43-48.

Finally the elastic scattering cross section was adjusted by subtracting all the other partial cross sections from the total cross section.

6. Other Quantities

6.1 Angular Distributions of Emitted Neutrons

The angular distribution of the elastically scattered neutrons was calculated with the optical model for each isotope, and that of natural nickel was obtained by averaging the data of isotopes by using $a_i \sigma_{si}$ as weights (a_i and σ_{si} denote the abundance and elastic scattering cross section of isotope i , respectively). For inelastic scattering to the discrete levels, the angular distributions were also calculated with the Hauser-Feshbach model for each isotope, and isotropic scattering in the center-of-mass system was assumed for natural nickel. Isotropic scattering in the laboratory system was assumed for the inelastic scattering to the continuum levels, and for the $(n,2n)$, $(n,3n)$, $(n,n'\alpha)$ and $(n,n'p)$ reactions.

6.2 Energy Distributions of Emitted Neutrons

The energy distributions of emitted neutrons were evaluated for each isotope as follows: The simple evaporation spectrum was assumed for the inelastic scattering to the continuum states (MT=91). The same nuclear temperature was also applied to the $(n,n'\alpha)$ and $(n,n'p)$ reactions. As to the $(n,2n)$ and $(n,3n)$ reactions, the successive evaporation model⁴³⁾ was assumed: The first neutron evaporates leaving the residual nucleus in an excited state higher than the neutron separation energy, and the second neutron evaporates from the excited state corresponding to the average energy of the first neutron, and so on.

In natural nickel file, we mixed the temperatures of ^{58}Ni and ^{60}Ni with the weights of $0.7 \times \sigma$ (^{58}Ni) and $0.3 \times \sigma$ (^{60}Ni), respectively, where σ denotes the cross section of the considered reaction.

7. Discussion

7.1 Direct and Semi-direct Reactions

The direct and semi-direct reactions were ignored in the present evaluation except for the inelastic scattering to the first level of the even-A isotopes, where the DWBA calculation was made. This assumption little affects the cross sections in the energy region below a few MeV, which are important for fission reactors. In fact the presently evaluated data were proved through benchmark tests^{14,15)} to be satisfactory for fission reactor calculations.

On the other hand, however, the direct and semi-direct processes become dominant for the high energy region such as 14 MeV. Recently the energy-angle double-differential cross sections (DDX) have been measured⁴⁴⁾ near 14 MeV for materials important for fusion reactors. The results of natural nickel are compared with the DDX calculated from JENDL-2* in Fig. 49. The spectrum calculated with JENDL-2 shows considerable underestimate for emission neutron with energy from 6 to 12 MeV. This means that the inelastic scattering cross sections to the levels between 2 MeV and 8 MeV are much underestimated. On the other hand, the peak near 12.5 MeV is well reproduced by the present calculation, suggesting that the inelastic scattering cross sections to the first level, calculated with DWBA, are adequate. Furthermore, the DDX data apparently show the forward peak in the angular distribution of

* Two processing codes FAIR-DDX⁴⁵⁾ and DDXPLOT⁴⁶⁾ have been developed to calculate DDX from JENDL library.

In natural nickel file, we mixed the temperatures of ^{58}Ni and ^{60}Ni with the weights of $0.7 \times \sigma$ (^{58}Ni) and $0.3 \times \sigma$ (^{60}Ni), respectively, where σ denotes the cross section of the considered reaction.

7. Discussion

7.1 Direct and Semi-direct Reactions

The direct and semi-direct reactions were ignored in the present evaluation except for the inelastic scattering to the first level of the even-A isotopes, where the DWBA calculation was made. This assumption little affects the cross sections in the energy region below a few MeV, which are important for fission reactors. In fact the presently evaluated data were proved through benchmark tests^{14,15)} to be satisfactory for fission reactor calculations.

On the other hand, however, the direct and semi-direct processes become dominant for the high energy region such as 14 MeV. Recently the energy-angle double-differential cross sections (DDX) have been measured⁴⁴⁾ near 14 MeV for materials important for fusion reactors. The results of natural nickel are compared with the DDX calculated from JENDL-2* in Fig. 49. The spectrum calculated with JENDL-2 shows considerable underestimate for emission neutron with energy from 6 to 12 MeV. This means that the inelastic scattering cross sections to the levels between 2 MeV and 8 MeV are much underestimated. On the other hand, the peak near 12.5 MeV is well reproduced by the present calculation, suggesting that the inelastic scattering cross sections to the first level, calculated with DWBA, are adequate. Furthermore, the DDX data apparently show the forward peak in the angular distribution of

* Two processing codes FAIR-DDX⁴⁵⁾ and DDXPLOT⁴⁶⁾ have been developed to calculate DDX from JENDL library.

inelastically scattered neutrons even to the continuum states. Such a behavior is not taken into account in the present evaluation.

The above comparison between the measured and calculated data of DDX suggests that the JENDL-2 data are insufficient in the cross section values and angular distributions of the inelastic scattering for fusion neutronics application where 14 MeV neutrons have a dominant role. This problem is an important subject for JENDL-3.

7.2 Threshold Reactions

Various threshold reactions are included. For reactions whose experimental data are numerous, such as the $^{58}\text{Ni}(n,2n)$, $^{58}\text{Ni}(n,p)$ and $^{60}\text{Ni}(n,p)$ reactions, we adopted the evaluation for JENDL-1, which was made by selecting lots of experimental data. For reactions whose experimental data are scarce, the evaluation was based on the simple evaporation model^{30,34)} with compound nucleus approximation or on the analogy of the other well-known reactions by considering the difference of Q-values. These methods are rather arbitrary and considerable uncertainty remains on the values thus evaluated. More systematic evaluation is now in progress for JENDL-3 by taking account of the pre-equilibrium processes.

The $(n,n'\alpha)$ reaction was considered only for ^{58}Ni . Though the $(n,n'\alpha)$ reaction cross section is much smaller (about 1/5) than the (n,α) reaction cross section at 14 MeV, its contribution becomes the larger in the higher energy region. Furthermore, the measurements are often made for the total α -emission cross section. Figures 50-55 show the total α -emission cross section with the measured data. Naturally the present evaluated value consists of only the (n,α) cross section for ^{60}Ni , ^{61}Ni , ^{62}Ni and ^{64}Ni .

For ^{58}Ni , the (n,α) and $(n,n'\alpha)$ reaction cross sections were

evaluated without considering the total α -emission cross section, and the total α -emission cross section is systematically larger than, the recent measurements by Grimes⁴⁷⁾ and Kneff et al.⁴⁸⁾. The same tendency is observed for the natural nickel. On the other hand, the total α -emission cross section looks underestimated at 15 MeV for ^{60}Ni , ^{61}Ni and ^{64}Ni due to negligence of the $(n, n'\alpha)$ reaction cross section. In order to know the helium production rate precisely, the (n, α) and $(n, n'\alpha)$ reaction cross sections should be evaluated for all the isotopes by considering the total α -emission cross section. This will be also made in JENDL-3.

8. Conclusion

The neutron nuclear data of natural nickel and its isotopes have been evaluated for JENDL-2. The present evaluation was made on the basis of recently measured data with the aid of the model calculation by considering the feedback from the benchmark tests^{11,12)} of JENDL-1.

Special care was taken on the following points:

- 1) The disagreement observed between the calculated and measured cross sections in the resonance region was carefully studied and the background connection was made. (See Appendix 1).
- 2) The remaining resonance structure observed in the unresolved region was adopted in the evaluation of the total cross section for the natural nickel file.
- 3) The inelastic scattering levels were grouped into 40 pseudo levels in the natural nickel file not to miss the levels of the main isotopes.

Results of the benchmark tests of JENDL-2 reveal reliability of the present data for fission reactor calculations.

On the other hand, the following problems have been pointed out:

- 1) The direct and semi-direct reactions were ignored. The comparison of

evaluated without considering the total α -emission cross section, and the total α -emission cross section is systematically larger than, the recent measurements by Grimes⁴⁷⁾ and Kneff et al.⁴⁸⁾. The same tendency is observed for the natural nickel. On the other hand, the total α -emission cross section looks underestimated at 15 MeV for ^{60}Ni , ^{61}Ni and ^{64}Ni due to negligence of the $(n, n'\alpha)$ reaction cross section. In order to know the helium production rate precisely, the (n, α) and $(n, n'\alpha)$ reaction cross sections should be evaluated for all the isotopes by considering the total α -emission cross section. This will be also made in JENDL-3.

8. Conclusion

The neutron nuclear data of natural nickel and its isotopes have been evaluated for JENDL-2. The present evaluation was made on the basis of recently measured data with the aid of the model calculation by considering the feedback from the benchmark tests^{11,12)} of JENDL-1.

Special care was taken on the following points:

- 1) The disagreement observed between the calculated and measured cross sections in the resonance region was carefully studied and the background connection was made. (See Appendix 1).
- 2) The remaining resonance structure observed in the unresolved region was adopted in the evaluation of the total cross section for the natural nickel file.
- 3) The inelastic scattering levels were grouped into 40 pseudo levels in the natural nickel file not to miss the levels of the main isotopes.

Results of the benchmark tests of JENDL-2 reveal reliability of the present data for fission reactor calculations.

On the other hand, the following problems have been pointed out:

- 1) The direct and semi-direct reactions were ignored. The comparison of

DDX data at 14 MeV suggests that the JENDL-2 data are not sufficient for fusion neutronics application.

2) Various threshold reactions should be evaluated more systematically.

These problems will be solved in JENDL-3.

Acknowledgment

The authors wish to thank S. Iijima, H. Yamakoshi, T. Asami and S. Igarasi for their helpful discussion. They are indebted to T. Nakagawa for his advice in using NDES. They also thank T. Narita for his aid in treating numerical experimental data. Careful typewriting by T. Maejima is much appreciated.

DDX data at 14 MeV suggests that the JENDL-2 data are not sufficient for fusion neutronics application.

2) Various threshold reactions should be evaluated more systematically.

These problems will be solved in JENDL-3.

Acknowledgment

The authors wish to thank S. Iijima, H. Yamakoshi, T. Asami and S. Igarasi for their helpful discussion. They are indebted to T. Nakagawa for his advice in using NDES. They also thank T. Narita for his aid in treating numerical experimental data. Careful typewriting by T. Maejima is much appreciated.

References

- 1) Schmidt J.J.: "Neutron Cross Sections for Fast Reactor Materials, Part I : Evaluation", KFK-120 (1966).
- 2) Bhat M.R.: "Neutron and Gamma-Ray Production Cross Sections for Nickel", BNL-50435 (1974) (for ENDF/B-IV).
- 3) Moxon M.C.: "Neutron Cross Sections of Natural Nickel and its Isotopes below a Neutron Energy of 600 keV", AERE-R 7568 (1974).
- 4) Guenther P., Smith A., Smith D., Whalen J. and Howerton R.: "Measured and Evaluated Fast Neutron Cross Sections of Elemental Nickel", ANL/NDM-11 (1975).
- 5) Goel B: "Graphical Representation of the German Nuclear Data Library KEDAK, Part I : Nonfissile Materials", KFK 2233 (1975) (for KEDAK-3).
- 6) Bychkov V.M., Vozyakov V.V., Manoshin V.N., Pratonov V.P., Popov V.I.: Proc. 4th all Union Conf. Neutron Physics, Kiev, 18-22 April, 1977, Vol.4, p.91, INDC(CCP)-118/G (1977) (for SOKRATOR).
- 7) Howerton R.J. and MacGregor M.H.: "The LLL Evaluated Nuclear Data Library (ENDL) : Description of Individual Evaluations for Z=0-98", UCRL-50400 Volume 15, Part D, Rev.1 (1978) (for ENDL-78).
- 8) Divadeenam M.: "Ni Elemental Neutron Induced Reaction Cross-Section Evaluation", BNL-NCS-51346 (1979) (for ENDF/B-V).
- 9) Igarasi S., Nakagawa T., Kikuchi Y. and Asami T.: "Japanese Evaluated Nuclear Data Library, Version-1 - JENDL-1 -", JAERI 1261 (1979).
- 10) Kikuchi Y., Nakabasami Y. and Matsunobu H.: Unpublished.
- 11) Kikuchi Y., Hasegawa A., Takano H., Kamei T., Hojuyama T., Sasaki M., Seki Y., Zukeran A. and Otake I.: "Benchmark Tests of JENDL-1", JAERI 1275 (1982).
- 12) Kawai M., Yamano N. and Koyama K.: "Nuclear Cross Sections for Technology", Proc. Int. Conf., Knoxville, Oct. 22-26, 1979, p.586, NBS Special Publication 594 (1980).

- 13) Inoue T., Shirakata K., Kinjo K., Ikegami T. and Yamamoto M.: J. At. Energy Soc. Jpn., 23, 310 (1981) [in Japanese].
- 14) Kikuchi Y., Narita T. and Takano H.: J. Nucl. Sci. Technol., 17, 567 (1980).
- 15) Kikuchi Y. and Members of JNDC: "Nuclear Data for Science and Technology", Proc. Int. Conf., 6-10 Sept. 1982, Antwerp, P.615, D. Reidel Publishing Co. (1983).
- 16) Kikuchi Y. and Sekine N.: To be published in J. Nucl. Sci. Technol.
- 17) Wapstra A.H. and Bos K.: Atomic Data and Nuclear Data Tables, 19, No.3 (1977).
- 18) Mughabgab S.F. and Garber D.I.: "Neutron Cross Sections, Vol.1, Resonance Parameters", BNL-325, 3rd Edition (1973).
- 19) Perey F.G., Chapman G.T., Kinney W.E. and Perey C.M.: "Neutron Data of Structural Materials for Fast Reactors", Proc. Specialists' Meeting, Geel, 5-8 Dec. 1977, p.503, Pergamon Press (1979).
- 20) Syme D.B., Bowen P.H. and Gadd A.D.: ibid, p.703.
- 21) Farrell J.A., Bilpuch E.G. and Newson H.W.: Ann. Phys. US, 37, 367 (1966).
- 22) Fröhner F.: "Neutron Data of Structural Materials for Fast Reactors", Proc. Specialists' Meeting, Geel, 5-8 Dec. 1977, p.138, Pergamon Press (1979).
- 23) Hockenbury R.W., Bartolome Z.M., Tataczuk J.R., Moyer W.R. and Block R.C.: Phys. Rev., 178, 1746 (1969).
- 24) Stieglitz R.G., Hockenbury R.W. and Block R.C.: Nucl. Phys., A163, 592 (1971).
- 25) Cho M., Fröhner F.H., Kazerouni M., Müller K.N. and Rohr G.: "Nuclear Data for Reactors", Conf. Proc., Helsinki, 15-19 June 1970, Vol.1, p.619, IAEA (1970).

- 26) Beer H. and Spencer R.R.: Nucl. Phys., A240, 29 (1975).
- 27) Cierjacks S., Forti P., Kopsch D., Kropp L., Nebe J., Unseld H.: "High Resolution Total Neutron Cross Sections for Si, Cr and Ni between 0.5 and 30 MeV", KFK 1000 (Supplement 1) (1968).
- 28) Nakagawa T.: J. At. Energy Soc. Jpn., 22, 559 (1980) [in Japanese].
- 29) Kawai M.: Paper Presented at 1979 NEANDC Topical Discussions, Geel, Sept. 1979.
- 30) Pearlstein S.: Nucl. Sci. Eng., 23, 238 (1965).
- 31) Pearlstein S.: J. Nucl. Energy. 27, 81 (1973).
- 32) Auchampaugh G.F., Drake D.M. and Veeser L.R.: Proc. Symp. Neutron Cross Sections from 10 to 40 MeV, BNL, May 3-5, 1977, p.231, BNL-NCS-50681 (1977).
- 33) Seebeck U. and Bormann M.: Nucl. Phys., 68, 387 (1965).
- 34) Gilat J.: "GROGI2 - A Nuclear Evaporation Computer Code Description and User's Manual", BNL 50246 (1970).
- 35) Igarasi S.: J. Nucl. Sci. Technol., 12, 67 (1975).
- 36) Ernst A., Fröhner F.H. and Kompe D.: "Nuclear Data for Reactors", Conf. Proc., Helsinki, 15-19 June 1970, Vol.1, p.633, IAEA (1970).
- 37) Gayther D.B., Thomas B.W., Thom B. and Moxon, M.C.: "Neutron Data of Structural Materials for Fast Reactors", Proc. Specialists' Meeting, Geel, 5-8 Dec. 1977, p.547, Pergamon Press (1979).
- 38) Berman B.L.: Atomic Data and Nucl. Data Tables, 15, No.4 (1975).
- 39) Lederer C.M. and Shirley V.S.: "Table of Isotopes", 7th Edition, Wiley-Interscience (1978).
- 40) Yoshida T.: Paper Presented at 1979 NEANDC Topical Discussions, Geel, Sept. 1979.
- 41) Gilbert A. and Cameron A.G.W.: Can. J. Phys., 43, 1446 (1965).

- 42) Broder D.L., Gamalij A.F., Lashuk A.I. and Sadokhin I.P.: "Nuclear Data for Reactors", Conf. Proc., Helsinki, 15-19 June 1970, Vol.2, p.295 (1970).
- 43) Kikuchi Y.: "EVAPSPEC Manual", Unpublished.
- 44) Takahashi A., Yamamoto J., Murakami T., Oshima K., Oda H., Fujimoto K. and Sumita K.: "Nuclear Data for Science and Technology", Proc. Int. Conf., 6-10 Sept. 1982, Antwerp, p.360, D. Reidel Publishing Co. (1983).
- 45) Minami K. and Yamano N.: "FAIR-DDX : A Code for Production of Double Differential Cross Section Library", JAERI-M 84-022 (1984) [in Japanese].
- 46) Iguchi T. and Yamano N.: "DDXPLOT : A Program to Plot the Energy Angle Double-Differential Cross Sections", JAERI-M 84-033 (1984) [in Japanese].
- 47) Grimes S.M., Haight R.C., Alvar K.R., Barschall H.H. and Borchers R.R.: Phys. Rev., C19, 2127 (1979).
- 48) Kneff D.W., Oliver B.M., Nakata M.M. and Farrar H.: Proc. Symp. Neutron Cross-Sections from 10 to 50 MeV, BNL, May 12-14, 1980, p.289, BNL-NCS-51245 (1980).
- 49) Garg J.B., Rainwater J. and Havens W.W.: Phys. Rev., C3, 2447 (1971).
- 50) Beer H., Spencer R.R. and Serman J.: "Projekt Schneller Brüter, 3, Vierteljahresbericht 1972", KFK-1272/3, p.121-24 (1972).
- 51) Moxon M.C.: "Neutron Capture in the keV Energy Range in Structural Materials for Fast Reactors", Proc. NEACRP/NEANDC Specialist Meeting, May 1973, Karlsruhe, KFK 2046, p.156 (1975).
- 52) Fröhner F.H.: "Nuclear Cross Sections and Technology", Proc. Conf., Washington D.C., March 3-7, 1975, p.929, NBS Special Publication 425 (1975).

- 53) Good W.M., Paya D., Wagner R. and Tamura T.: Phys. Rev., 151, 912 (1966).
- 54) Hockenbury R.W., Kaushal N.N. and Block R.C.: "Linear Accelerator Project, Annual Technical Report Oct. 71 - Sept. 72", COO-3058-27, p.14 (1972).

References for Experimental Data in Figures

- E1: Havens W.W., Jr. and Rainwaters L.J.: Phys. Rev., 83, 1123 (1951).
- E2: Merrison A.W. and Wiblin E.R.: Proc. Roy. Soc. (London) Section A, 215, 278 (1952).
- E3: Gatti E., Germagnoli E. and Perona G.: Nuovo Cimento, 11, 262 (1954).
- E4: Hibdon C.T. and Langsdorf A.: Phys. Rev., 98, 223 (1955).
- E5: Brown R.J. and Bollinger L.M.: Nucl. Sci. Eng., 4, 576 (1958).
- E6: Garg J.B., Rainwater J. and Havens W.W.: Phys. Rev., C3, 2447 (1971).
- E7: Tattersall R.B., Rose H., Pattenden S.K. and Jowitt D.: J. Nucl. Energy, A12, 32 (1960).
- E8: Carre J.C. and Vidal R.: "Nuclear Data for Reactors", Proc. Conf. Paris, 17-21 Oct. 1966, Vol.1, p.479, IAEA, Vienna (1967).
- E9: Spitz L.M., Barnard E. and Brooks F.D.: Nucl. Phys., A121, 655 (1968).
- E10: Malik S.S., Brunkart G., Shore F.J. and Sailor V.L.: Nucl. Instrum. Methods, 86, 83 (1970).
- E11: Hibdon C.T., Langsdorf A. and Holland R.E., Phys. Rev., 85, 595 (1952).
- E12: Gibbons J.H., Macklin R.L., Miller P.D. and Neiler J.H.: Phys. Rev., 122, 182 (1961).
- E13: Cabe J. and Cance M.: "Measurements of the Neutron Total Cross-Sections of Be, ^{11}B , C, Al, Si, S, Ti, V, Ni, ^{235}U , ^{238}U , ^{239}Pu between 100 keV and 6 MeV", CEA-R-4524 (1973).
- E14: Guenther P., Smith A.B. and Whalen J.: Nucl. Sci. Eng., 59, 106 (1976).
- E15: Le Rigoreur C., Arnaud A. and Taste J.: "Measurement of Neutron Radiative Capture Cross-Sections of Chromium, Iron and Nickel from 70 keV to 550 keV", CEA-N-1661 (1973).

- E16: Perey F.G., Love T.A. and Kinney: "Test of Neutron Total Cross Section Evaluations from 0.2 to 20 MeV for C, O, Al, Si, Ca, Fe and SiO_2 ", ORNL-4823 (1972).
- E17: Macklin R.L. and Gibbons J.H.: Phys. Rev., 159, 1007 (1967).
- E18: Green L. and Mitchell J.A.: "Neutron Cross Sections and Technology", Proc. Conf., Knoxville, March 15-17, 1971, CONF-710301, p.688 (1971).
- E19: Schwartz R.B., Schrak R.A. and Heaton H.T.: "MeV Total Neutron Cross Sections", NBS-Mono-138 (1974).
- E20: Cierjacks S., Forti P., Kopsch D., Kropp L., Nebe J., Unseld H.: "High Resolution Total Neutron Cross Sections for Si, Cr, Ni between 0.5 and 30 MeV", KFK 1000 (Supplement 1) (1968).
- E21: Farrell J.A., Bilpuch E.G. and Newson H.W.: Ann. Phys. US, 37, 367 (1966).
- E22: Ernst A., Fröhner F.H. and Kompe D.: "Nuclear Data for Reactors", Conf. Proc., Helsinki, 15-19 June 1970, Vol.1, p.633, IAEA (1970).
- E23: Stoler P., Clement J. Goulding C. and Fairchild R.: "Neutron Cross Sections and Technology", Proc. Conf., Knoxville, March 15-17, 1971, CONF-710301, p.311 (1971).
- E24: Cho M., Fröhner F.H., Kazerouni M., Müller K.N. and Rohr G.: "Nuclear Data for Reactors", Conf. Proc., Helsinki, 15-19 June 1970, Vol.1, p.619, IAEA (1970).
- E25: Spencer R.R., Beer H. and Fröhner F.H.: "The Total Neutron Cross Sections of ^{50}Cr , ^{52}Cr , ^{62}Ni and ^{64}Ni in the Energy Region 10-300 keV", KFK-1517 (1972).
- E26: Beer H. and Spencer R.P.: Nucl. Phys., A240, 29 (1975).
- E27: Boschung P., Lindow J.T. and Shrader E.F.: Nucl. Phys., A161, 593 (1971).
- E28: Smith A., Guenther P., Smith D. and Whalen J.: Nucl. Sci. Eng., 72, 293 (1979).

- E29: Benveniste J.: Proc. 2nd UN Int. Conf. Peaceful Usages of Atomic Energy, Geneva, 1-13 Sept. 1958, Vol.15, p.3 (1958).
- E30: Auchampaugh G.F., Drake D.M. and Veeser L.R.: Proc. Symp. Neutron Cross Sections from 10 to 40 MeV, BNL, May 3-5, 1977, p.231, BNL-NCS-50681 (1977).
- E31: Prestwood R.J. and Bayhurst B.P.: Phys. Rev., 121, 1438 (1961).
- E32: Jeronymo J.M.F., Mani G.S., Olkowski J., Sadeghi A. and Williamson C.F.: Nucl. Phys., 47, 157 (1963).
- E33: Paulsen A. and Liskien H.: Nukleonik, 7, 117 (1965).
- E34: Chojnacki S., Decowski P., Gierlik E., Grochulski W., Marcinkowski A., Siwek K., Sledzinska I. and Wilhelmi Z.: "Ni-58 (n,2n) Cross Section in the Energy Range from 13 to 16 MeV", INR-680 (1965).
- E35: Bormann M., Dreyer F. and Zielinski U.: "Measurements of some Fast Neutron Cross Sections with the Activation Method", EANDC(E)-66, p.42 (1966).
- E36: Csikai J. and Peto G.: Acta. Phys. Acad. Sci. Hung., 23, 87 (1967).
- E37: Temperley J.K.: Nucl. Sci. Eng., 32, 195 (1968).
- E38: Fink R.W. and Lu W.D.: Bull. Am. Phys. Soc., 15, 1372 (1970).
- E39: Hemingway J.D.: J. Nucl. Energy, 27, 241 (1973).
- E40: Araminowicz J. and Dresler J.: "Investigation of the (n,2n) reactions at 14.6 MeV for 42 Nuclides", INR-1464, p.14 (1973).
- E41: Qaim S.M. and Molla N.I.: Nucl. Phys., A283, 269 (1977).
- E42: Hudson C.G., Alford W.L. and Ghorai S.K.: Ann. Nucl. Energy, 5, 589 (1972).
- E43: Lees E.W., Patrick B.H. and Lindley S.: "A Search for the $^{238}\text{U}(\gamma,\alpha)$ ^{234}Th and $^{238}\text{U}(n,\alpha n)$ ^{234}Th and Measurements of the $^{60}\text{Ni}(n,p)$ ^{60}Co and $^{62}\text{Ni}(n,\alpha)$ ^{59}Fe cross sections at 14.7 MeV Neutron Energy from Associated Monitor Foils", AERE-R-9390 (1979).

- E44: Adamski L., Herman M. and Marcinkowski A.: Ann. Nucl. Energy, 7, 397 (1980).
- E45: Seebeck U. and Bormann M.: Nucl. Phys., 68, 387 (1965).
- E46: Kumabe I. and Fink R.W.: Nucl. Phys., 15, 316 (1960).
- E47: Glover R.N. and Weigold E.: Nucl. Phys., 29, 309 (1962).
- E48: Bramlett E.T. and Fink R.W.: Phys. Rev., 131, 2649 (1963).
- E49: Cross W.G., Clarke R.L., Morin K., Slinn G., Ahmed N.M. and Beg K.: "Activation Cross Sections of Fe and Ni Isotopes for 14.5 MeV Neutrons", EANDC(CAN)-16, p.1 (1963).
- E50: Barrall R.C., Silbergeld M. and Gardner D.G.: Nucl. Phys., A138, 387 (1969).
- E51: Maslov G.N., Nasyrov F. and Pashkin N.F.: "The Experimental Cross Sections of the Nuclear Reactions for 14 MeV Neutrons", YK-9, p.50 (1972).
- E52: Weigel H., Michel R. and Herr W.: Radiochimica Acta, 22, 11 (1975).
- E53: Fukuda K., Matsuo K., Shirahama S. and Kumabe I.: "Activation Cross Sections on Fe, Co, Ni, Zr and Mo for 14.6 MeV Neutrons", NEANDC(J)-56/U, p.44 (1978).
- E54: Allan D.L.: Proc. Phys. Soc. (London), A70, 195 (1957).
- E55: March P.V. and Morton W.T.: Philosophical Magazine, 3, 577 (1958).
- E56: Allan D.L.: Nucl. Phys., 24, 274 (1961).
- E57: Preiss I.L. and Fink R.W.: Nucl. Phys., 15, 326 (1960).
- E58: Val'ter A.K., Gonchar V.Ju, Zaljubovskij I.I., Latyshev G.D. and Chursin G.P.: Izv. Akad. Nauk SSSR, Ser. Fiz., 26, 1079 (1962).
- E59: Barry J.F.: J. Nucl. Energy, 16, 467 (1962).
- E60: Debertin K. and Rossle E.: Nucl. Phys. 70, 89 (1965).
- E61: Bormann M., Dreyer F., Neuert H., Riehle I. and Zielinski U.: "Nuclear Data for Reactors", Conf. Proc., Paris, 17-21 Oct. 1966, Vol.1, p.225, IAEA (1967).

- E62: Okumura S.: Nucl. Phys. A93, 74 (1967).
- E63: Decowski P., Gochulski W., Marcinkowski A., Siwek K., Sledzinska I. and Wilhelmi Z: Nucl. Phys., A112, 513 (1968).
- E64: Paulsen A. and Widera R.: Proc. Conf. Chem. Nucl. Data, Measurements and Applicat., Univ. of Kent, Canterbury, 20-22 Sept. 1971, p.129 (1971).
- E65: Dresler J., Araminowicz J. and Garuska U.: "Cross Sections of the (n,p) Reactions at 14.6 MeV for 13 Nuclides", INR-1464, p.12 (1973).
- E66: Smith D.L. and Meadows J.W.: Nucl. Sci. Eng., 58, 314 (1975).
- E67: Nemilov Ju.A. and Trofimov Ju. N.: "Cross Sections of (n,p) Reactions on Ni-58, Fe-56 and Zn-64 isotopes at Neutron Energies 7.6 - 9.3 MeV", YFI-25, p.46 (1977).
- E68: Wu N.W. and Chon J.C.: Nucl. Sci. Eng., 63, 268 (1977).
- E69: Paulsen A. and Liskien H.: Nukleonik, 10, 91 (1967).
- E70: Levkovskij V.N., Vinitskaja G.P., Kovel'skaja G.E. and Stepanov V.M.: Nucl Phys., 10, 44 (1969).
- E71: Paul E.B. and Clarke R.L.: Can. J. Phys., 31, 267 (1953).
- E72: Van Loef J.J.: Nucl. Phys., 24, 340 (1961).
- E73: Strain J.E. and Cross W.J.: "14 MeV Neutron Reactions", ORNL-3672 (1965).
- E74: Paulsen A., Liskien H., Arnotte F. and Widera R.: Nucl. Sci. Eng., 78, 377 (1985).
- E75: Yu-Wen Yu and Gardner D.G.: Nucl. Phys., A98, 451 (1967).
- E76: Levkovsky V.N., Kovelskaya G.E., Vinitskaya G.P., Stepanov V.M. and Sokolsky V.V.: Izv. Akad. Nauk SSSR, Ser. Fiz., 8, 7 (1968).
- E77: Belanova T.S.: Sov. Phys. JETP, 34, 397 (1958).
- E78: Diven B.C., Terrell J. and Hemmendinger A.: Phys. Rev., 120, 556 (1960).

- E79: Stavisskij Yu.Ya. and Shapar A.V.: Atomnaya Energiya, 10, 264 (1961).
- E80: Gayther D.B., Thomas B.W., Thom B. and Moxon M.C.: "Neutron Data of Structural Materials for Fast Reactors", Proc. Specialists' Meeting, Geel, 5-8 Dec. 1977, p.547. Pergamon Press (1979).
- E81: Towle J.H. and Owens R.O.: Nucl. Phys., A100, 257 (1967).
- E82: Rodgers W.L., Shrader E.F. and Lindon J.T.: "Neutron Scattering from ^{12}C , ^{56}Fe , ^{65}Cu , ^{58}Ni and ^{60}Ni ", COO-1573-33, p.2 (1967).
- E83: Boschung P., Lindon J.T. and Shrader E.F.: Nucl. Phys., A161, 593 (1971).
- E84: Rodgers V.C., Beghian L.E., Clikeman F.M. and Mahoney F.S.: Nucl. Sci. Eng., 45, 297 (1971).
- E85: Konobeevskij E.S., Musaelyan R.M., Popov V.I. and Surkova I.V.: Izv. Akad. Nauk SSSR, Ser. Fiz, 35, 2345 (1971).
- E86: Etemad M.A.: "Neutron Inelastic Scattering Cross Sections in the Energy Range 2 to 4.5 MeV. Measurements and Calculations", AE-481 (1973).
- E87: Sokolov L.S., Fjodorov M.B., Korbetskij E.V., Surovitskaja N.T. and Jakovenko T.I.: Ukrainskij Fizichnij Zhurnal, 18, 263 (1973).
- E88: Korzh I.A., Mishchenko V.A., Mozhghukhin E.N., Golubova A.A., Pravdivyj N.M., Sanzhur I.E. and Pasechnik M.V.: Proc. 3rd All-Union Conf. Neutron Physics, Kiev, 9-13 June 1975, Vol.4, p.220 (1976).
- E89: Broder D.L., Gamalij A.F., Lashuk A.I. and Sadokhin I.P.: "Nuclear Data for Reactors", Conf. Proc., Helsinki, 15-19 June 1970, Vol.2, p.295, IAEA (1970).
- E90: Fujita I., Sonoda M., Katase A., Wakuta Y., Tawara H., Hyakutake M. and Iwatani K.: J. Nucl. Sci. Technol. 9, 301 (1973).
- E91: Joensson B., Nyberg K. and Bergqvist I.: Arkiv for Fysik, 39, 295 (1969).

- E92: Takahashi A., Yamamoto J., Murakami T. Oshima K., Oda H., Fujimoto K. and Sumita K.: "Nuclear Data for Science and Technology", Proc. Int. Conf. 6-10 Sept. 1982, Antwerp, p.360, D. Reidel Publishing Co. (1983).
- E93: Grimes S.M., Haight R.C., Alvar K.R., Barschall H.H. and Bouchers R.R.: "Charged Particle Emission in Reactions of 15 MeV Neutrons with Isotopes of Chromium, Iron, Nickel and Copper", UCRL-81802 (1978).
- E94: Kneff D.W., Oliver B.M., Nakata M.M. and Farrar H.: Proc. Symp. Neutron Cross-Sections from 10 to 50 MeV, BNL, May 12-14, 1980, p.289, BNL-NCS-51245 (1980).
- E95: Dolja G.P., Kljucharjov A.P., Bozhko V.P., Golovnja V. Ja., Kachan A.S. and Tutubalin A.I.: Proc. 3rd All-Union Conf. Neutron Physics, Kiev, 3-13 June 1975, Vol.4, p.173 (1975).
- E96: Kapchigashev P. and Popov U.P.: Atomnaya Energiya, 15, 120 (1963).
- E97: Hockenbury R.W., Bartolome Z.M., Tatarczuk J.R., Moyer W.R. and Block R.C.: Phys. Rev., 178, 1746 (1969).
- E98: Grench H.A.: Phys. Rev., B140, 1277 (1965).
- E99: Perey F.G., Le Rigoleur C.O. and Kinney W.E.: "Nickel-60 Neutron Elastic- and Inelastic-Scattering Cross Sections from 6.5 to 8.5 MeV", ORNL-4523 (1970).
- E100: Kinney W.E. and Perey F.G.: "Ni Neutron Elastic- and Inelastic- Scattering Cross Sections from 4.07 to 8.56 MeV", ORNL-4807 (1974).

Table 1 Status of presently evaluated quantities

Natural nickel

Quantities	Energy range(eV)*		Comments
	min	max	
a) Resonance data			Constructed from isotopic data
Resonance parameters	-2.85+4	6.0+5	
Capture resonance integral	5.0 -1		Table 11
b) Cross sections			
Total	1.0 -5	2.0+7	Figs. 1 and 7
Elastic scattering	1.0 -5	2.0+7	
Total inelastic scattering	6.85+4	2.0+7	Fig. 43
Inelastic scattering			
to the lowest discrete level (1st)	6.85+4	2.0+7	
to the highest discrete level (40th)	4.55+6	2.0+7	
to the continuum levels	2.57+6	2.0+7	
Capture	1.0 -5	2.0+7	Figs. 1, 33 and A6
(n,2n) reaction	7.95+6	2.0+7	Fig. 10
(n,3n) reaction	1.68+7	2.0+7	
(n,n' α) reaction	6.51+6	2.0+7	
(n,n' p) reaction	8.31+6	2.0+7	
(n, p) reaction	1.0 -5	2.0+7	Fig. 21
(n, α) reaction	1.0 -5	2.0+7	Fig. 27
c) Angular distributions of secondary neutrons			Optical model
Elastic scattering	1.0 -5	2.0+7	
Inelastic scattering to the discrete levels	6.85+4	2.0+7	
d) Energy distributions of secondary neutrons			Evaporation model
Inelastic scattering to the continuum levels	2.57+6	2.0+7	Fig. 49
(n,2n) reaction	7.95+6	2.0+7	Fig. 49
(n,3n) reaction	1.68+7	2.0+7	
(n,n' α) reaction	6.51+6	2.0+7	
(n,n' p) reaction	8.31+6	2.0+7	
(n, p) reaction	1.0 -5	2.0+7	
(n, α) reaction	1.0 -5	2.0+7	

*2.0+7 denotes 2.0×10^7 .

⁵⁸Ni

Quantities	Energy range(eV)*		Comments
	min	max	
a) Resonance data			
Resonance parameters	-2.85+4	6.0+5	Table 6
Capture resonance integral	5.0 -5		Table 11
b) Cross sections			
Total	1.0 -5	2.0+7	Figs. 2 and 8
Elastic scattering	1.0 -5	2.0+7	
Total inelastic scattering	1.48+6	2.0+7	Fig. 44
Inelastic scattering			
to the lowest discrete level (1st)	1.48+6	2.0+7	Fig. 39
to the highest discrete level (22nd)	4.55+6	2.0+7	
to the continuum levels	4.60+6	2.0+7	
Capture	1.0 -5	2.0+7	Figs. 2 and 34
(n,2n) reaction	1.24+7	2.0+7	Fig. 11
(n,n' α) reaction	6.51+6	2.0+7	Fig. 16
(n,n' p) reaction	8.31+6	2.0+7	Fig. 17
(n, p) reaction	1.0 -5	2.0+7	Fig. 22
(n, α) reaction	1.0 -5	2.0+7	Fig. 28
c) Angular distributions of secondary neutrons			Optical model
Elastic scattering	1.0 -5	2.0+7	
Inelastic scattering to the discrete levels	1.48+6	2.0+7	
d) Energy distributions of secondary neutrons			Evaporation model
Inelastic scattering to the continuum levels	4.60+6	2.0+7	
(n,2n) reaction	1.24+7	2.0+7	
(n,n' α) reaction	6.51+6	2.0+7	
(n,n' p) reaction	8.31+6	2.0+7	
(n, p) reaction	1.0 -5	2.0+7	
(n, α) reaction	1.0 -5	2.0+7	

*2.0+7 denotes 2.0×10^7 .

$^{60}_{\text{Ni}}$

Quantities	Energy range(eV)*		Comments
	min	max	
a) Resonance data			
Resonance parameters	-5.5	+3	6.0+5
Capture resonance integral	5.0	-5	Table 11
b) Cross sections			
Total	1.0	-5	2.0+7
Elastic scattering	1.0	-5	2.0+7
Total inelastic scattering	1.35+6		2.0+7
Inelastic scattering			Fig. 45
to the lowest discrete level (1st)	1.35+6		2.0+7
to the highest discrete level (22nd)	3.94+6		2.0+7
to the continuum levels	3.96+6		2.0+7
Capture	1.0	-5	2.0+7
(n,2n) reaction	1.16+7		2.0+7
(n,n'p) reaction	9.70+6		2.0+7
(n,p) reaction	2.08+6		2.0+7
(n, α) reaction	1.0	-5	2.0+7
c) Angular distributions of secondary neutrons			Optical model
Elastic scattering	1.0	-5	2.0+7
Inelastic scattering to the discrete levels	1.35+6		2.0+7
d) Energy distributions of secondary neutrons			Evaporation model
Inelastic scattering to the continuum levels	3.96+6		2.0+7
(n,2n) reaction	1.16+7		2.0+7
(n,n'p) reaction	9.70+6		2.0+7
(n,p) reaction	2.08+6		2.0+7
(n, α) reaction	1.0	-5	2.0+7

* $2.0+7$ denotes 2.0×10^7 .

⁶¹Ni

Quantities	Energy range(eV)*		Comments
	min	max	
a) Resonance data			
Resonance parameters	-1.8 +3	6.85+4	Table 8
Capture resonance integral	5.0 -1		Table 11
b) Cross sections			
Total	1.0 -5	2.0+7	Fig. 4
Elastic scattering	1.0 -5	2.0+7	
Total inelastic scattering	6.85+4	2.0+7	Fig. 46
Inelastic scattering			
to the lowest discrete level (1st)	6.85+4	2.0+7	
to the highest discrete level (20th)	2.51+6	2.0+7	
to the continuum levels	2.57+6	2.0+7	
Capture	1.0 -5	2.0+7	Figs. 4 and 36
(n,2n) reaction	7.95+6	2.0+7	Fig. 13
(n,n'p) reaction	1.00+7	2.0+7	Fig. 19
(n,p) reaction	5.49+5	2.0+7	Fig. 24
(n, α) reaction	1.0 -5	2.0+7	Fig. 30
c) Angular distributions of secondary neutrons			
Elastic scattering	1.0 -5	2.0+7	
Inelastic scattering to the discrete levels	6.85+4	2.0+7	
d) Energy distributions of secondary neutrons			
Inelastic scattering to the continuum levels	2.57+6	2.0+7	Evaporation model
(n,2n) reaction	7.95+6	2.0+7	
(n,n'p) reaction	1.00+7	2.0+7	
(n,p) reaction	5.49+5	2.0+7	
(n, α) reaction	1.0 -5	2.0+7	

* 2.0+7 denotes 2.0×10^7 .

^{62}Ni

Quantities	Energy range(eV) [*]		Comments
	min	max	
a) Resonance data			
Resonance parameters	4.6 +3	6.0+5	Table 9
Capture resonance integral	5.0 -1		Table 11
b) Cross sections			
Total	1.0 -5	2.0+7	Fig. 5
Elastic scattering	1.0 -5	2.0+7	
Total inelastic scattering	1.19+6	2.0+7	Fig. 47
Inelastic scattering			
to the lowest discrete level (1st)	1.19+6	2.0+7	Fig. 41
to the highest discrete level (21st)	3.92+6	2.0+7	
to the continuum levels	4.03+6	2.0+7	
Capture	1.0 -5	2.0+7	Figs. 5 and 37
(n,2n) reaction	1.08+7	2.0+7	Fig. 14
(n,n'p) reaction	1.13+7	2.0+7	Fig. 20
(n,p) reaction	4.53+6	2.0+7	Fig. 25
(n, α) reaction	4.44+5	2.0+7	Fig. 31
c) Angular distributions of secondary neutrons			Optical model
Elastic scattering	1.0 -5	2.0+7	
Inelastic scattering to the discrete levels	1.19+6	2.0+7	
d) Energy distributions of secondary neutrons			Evaporation model
Inelastic scattering to the continuum levels	4.03+6	2.0+7	
(n,2n) reaction	1.08+7	2.0+7	
(n,n'p) reaction	1.13+7	2.0+7	
(n,p) reaction	4.53+6	2.0+7	
(n, α) reaction	4.44+5	2.0+7	

*2.0+7 denotes 2.0×10^7 .

⁶⁴Ni

Quantities	Energy range(eV) [*]		Comments
	min	max	
a) Resonance data			
Resonance parameters	9.52+3	6.0+5	Table 10
Capture resonance integral	5.0 -1		Table 11
b) Cross sections			
Total	1.0 -5	2.0+7	Fig. 6
Elastic scattering	1.0 -5	2.0+7	
Total inelastic scattering	1.37+6	2.0+7	Fig. 48
Inelastic scattering			
to the lowest discrete level (1st)	1.37+6	2.0+7	Fig. 42
to the highest discrete level (20th)	4.03+6	2.0+7	
to the continuum levels	4.15+6	2.0+7	
Capture	1.0 -5	2.0+7	Figs. 6 and 38
(n,2n) reaction	9.81+6	2.0+7	Fig. 15
(n,3n) reaction	1.68+7	2.0+7	Fig. 15
(n,p) reaction	6.63+6	2.0+7	Fig. 26
(n,α) reaction	2.48+6	2.0+7	Fig. 32
c) Angular distributions of secondary neutrons			Optical model
Elastic scattering	1.0 -5	2.0+7	
Inelastic scattering to the discrete levels	1.37+6	2.0+7	
d) Energy distributions of secondary neutrons			Evaporation model
Inelastic scattering to the continuum levels	4.15+6	2.0+7	
(n,2n) reaction	9.81+6	2.0+7	
(n,3n) reaction	1.68+7	2.0+7	
(n,p) reaction	6.63+6	2.0+7	
(n,α) reaction	2.48+6	2.0+7	

* $2.0+7$ denotes 2.0×10^7 .

Table 2 Isotopic abundances, exact masses and
various reaction Q-values of Ni isotopes.

	^{58}Ni	^{60}Ni	^{61}Ni	^{62}Ni	^{64}Ni	
a) Abundance*						Z
(%)	67.86	26.21	1.19	3.66	1.08	
b) Exact mass**						
(a.m.u.)	57.9354	59.9308	60.9311	61.9284	63.9280	
c) Q-values***						
(MeV)						
(n,2n)	-12.1970	-11.3890	-7.8206	-10.5980	- 9.6570	
(n,3n)	-22.4657 ⁺	-20.3893 ⁺	-19.5276 ⁺	-18.7186 ⁺	-16.4959	
(n,n' α)	- 6.3978	- 6.2910 ⁺	- 6.4650 ⁺	- 7.0815 ⁺	- 8.0858 ⁺	
(n,n'p)	- 8.1711	- 9.533	-9.8617	-11.1376	-12.5371 ⁺	
(n,p)	0.4022	- 2.4011	-0.5396	- 4.4589	- 6.5244	
(n, α)	2.901	1.3555	3.5795	- 0.4373	- 2.4412	

* Taken from the recommendation by Moxon³⁾

** Taken from the compilation of Wapstra and Bos¹⁷⁾

+ Not evaluated in JENDL-2

Table 3 The 2200m/s cross sections

(barns)

	Present *	Total	Capture	
		BNL-325(3) ¹⁸⁾	Present *	BNL-325(3) ¹⁸⁾
Natural	21.20	—	4.429	4.43 ± 0.16
⁵⁸ Ni	30.62	30.4 ± 0.4	4.605	4.6 ± 0.3
⁶⁰ Ni	3.87	3.8 ± 0.2	2.801	2.8 ± 0.2
⁶¹ Ni	12.12	12.1 ± 0.8	2.506	2.5 ± 0.8
⁶² Ni	23.70	23.7 ± 0.5	14.20	14.2 ± 0.3
⁶⁴ Ni	1.52	—	1.480	1.49 ± 0.03

* Calculated from the resonance parameters.

Table 4 Measured data on the basis of which the evaluation
of resonance parameters was made.

Isotopes	Type *	Measured Data
⁵⁸ Ni	T	Perey et al. ¹⁹⁾ , Symme and Bowen ²⁰⁾ , Farrell et al. ²¹⁾
	C	Perey et al. ¹⁹⁾ , Fröhner ²²⁾ , Hockenbury et al. ²³⁾
⁶⁰ Ni	T	Symme and Bowen ²⁰⁾ , Stieglitz et al. ²⁴⁾ , Farrell et al. ²¹⁾
	C	Fröhner ²²⁾ , Stieglitz et al. ²⁴⁾ , Hockenbury et al. ²³⁾
⁶¹ Ni	T	Cho et al. ²⁵⁾
	C	Fröhner ²²⁾ , Hockenbury et al. ²³⁾
⁶² Ni	T	Beer and Spencer ²⁶⁾ , Farrell et al. ²¹⁾
	C	Beer and Spencer ²⁶⁾
⁶⁴ Ni	T	Beer and Spencer ²⁶⁾ , Farrell et al. ²¹⁾
	C	Beer and Spencer ²⁶⁾

* T denotes transmission measurements, and C capture measurements

Table 5 Status of resonance parameters

Isotope	Defined energy range			S-wave resonances			P-wave resonances			Effective scattering radius	
	Min (eV)	Max (keV)	No	negative (keV)	E min (keV)	E max (keV)	No	E min (keV)	E max (keV)	(fm)	
^{58}Ni	10^{-5}	600	32	-28.5, -5	15.2	600	120	6.9	604	7.5	
^{60}Ni	10^{-5}	600	38	-5.5	12.5	595	69	1.3	567	6.5	
^{61}Ni	10^{-5}	68.6	32	-1.8	7.15	68.8	25	1.35	30.1	6.4	
^{62}Ni	10^{-5}	600	33	non	4.6	592	49	8.9	601	6.2	
^{64}Ni	10^{-5}	600	26	non	14.3	584	37	9.52	566	6.4	

Table 6 Resonance parameters of ^{58}Ni

ENERGY (KEV)	L	J	NEUTRON WIDTH *	GAMMA WIDTH *	MWS **	MISCELLANEOUS ***	REFERENCE
(EV)						(EV)	(EV)
-28.8	0	0.5	7870	2.0			JENDL-2
-28.5	0	0.5	16544.0	9.0			JENDL-1
-28.5	0	0.5	11817.0	2.14			ENDF-B-4
-28.5 ± 5.0	0	0.5		9.0 ± 0.6			BNL325(3)
-28.5	0		7870	2.0			73HDXON
-28.5	0	0.5	7870	{ 2.0 }			74HDXON
-28.5	0		16400 ± 900	9			71GARD
-5.5	0	0.5	1060	1.81			JENDL-2
-5.5	0		1081	1.87			73HDXON
-5.5	0	0.5	1081	{ 1.87 }			74HDXON
6.89	1	0.5	0.02	{ 1.0 }	0.020 ± 0.001		JENDL-2
6.89	1	0.5	0.0225	1.0			JENDL-1
6.89	1	0.5	0.023	0.6			ENDF-B-4
6.89	1				0.022 ± 0.003		BNL325(3)
6.89	0	0.5	0.022	1.0			73HDXON
6.89	1	0.5	0.022	{ 1.0 }	0.022 ± 0.002		74HDXON
6.89	1				0.022 ± 0.002		69HOCKENBURY
6.90	1		{ 0.02 }		0.020 ± 0.001		77PEREY
12.63	1	0.5	0.025	{ 1.0 }	0.024 ± 0.002		JENDL-2
12.6	1	0.5	0.031	1.0			JENDL-1
12.6	0	0.5	{ 0.03 }	1.0			73HDXON
12.6	1	0.5	{ 0.03 }	{ 1.0 }			74HDXON
12.63	1		{ 0.02 }		0.024 ± 0.002		69HOCKENBURY
12.63	1						77PEREY
13.344	1	0.5	9.5	0.712	0.661 ± 0.047		JENDL-2
13.3	1	0.5	0.64	1.0			JENDL-1
13.3	1	1.5	0.22	0.6			ENDF-B-4
13.34 ± 0.03	1				0.39 ± 0.05		BNL325(3)
13.3	0	0.5	0.47	1.0			73HDXON
13.34	1	0.5	0.5	{ 1.0 }			74HDXON
13.3					0.32 ± 0.03		69HOCKENBURY
13.34 ± 0.03	1				0.49 ± 0.10		72BEER
13.34 ± 0.03	1				0.50 ± 0.08		77FROEHNER
13.344 ± 4	1		9.5 ± 0.3	0.712	0.661 ± 0.047		77PEREY
13.42	1		4.9 ± 2.5				77SYME1
13.622	1	0.5	1.8	0.904	0.604 ± 0.015		JENDL-2
13.6	1	1.5	0.4	1.0			JENDL-1
13.6	1	1.5	0.46	0.6			ENDF-B-4
13.66 ± 0.04	1				0.57 ± 0.05		BNL325(3)
13.6	0	0.5	1.08	1.0			73HDXON
13.66	1	0.5	1.16	{ 1.0 }			74HDXON
13.6					0.52 ± 0.05		69HOCKENBURY
13.66 ± 0.04	1				0.63 ± 0.12		72BEER
13.68 ± 0.03	1				0.63 ± 0.20		77FROEHNER
13.622 ± 1	1		1.8 ± 0.2	0.904	0.604 ± 0.015		77PEREY
13.63	1		2.9 ± 1.2				77SYME1
15.2	0	0.5	1380	2.054	2.052 ± 0.064		JENDL-2
15.5	0	0.5	1200.0	2.1			JENDL-1
15.5	0	0.5	1400.0	2.14			ENDF-B-4
15.50 ± 0.04	0	0.5	1200 ± 100	2.1 ± 0.7			BNL325(3)
15.42	0		1150	2.0			73HDXON
15.375	0	0.5	1190	2.1			74HDXON
16.5	0						66FARRELL
15.3 ± 0.2	0		1140 ± 30				71GARD
15.4 ± 0.1	0		1200 ± 150	2.1 ± 0.7			72BEER
15.4 ± 0.1	0		1200 ± 30	1.42 ± 0.18			75FROEHNER
15.4 ± 0.1	0	0.5	{ 1140 }	1.48 ± 0.22			77FROEHNER
15.344 ± 0.01	0		1380 ± 20	2.054	2.052 ± 0.064		77PEREY
15.2011 ± 0.0255	0		1368.9 ± 0.16				77SYME1
16.5	1	0.5	0.02	1.0			JENDL-1
16.5	0	0.5	{ 0.02 }	1.0			73HDXON
16.5	1	0.5	{ 0.02 }	{ 1.0 }			74HDXON
16.5							69HOCKENBURY
17.21	1	0.5	0.027	{ 1.0 }	0.026 ± 0.004		JENDL-2
17.2	1	0.5	0.02	1.0			JENDL-1
17.2	0	0.5	{ 0.02 }	1.0			73HDXON
17.2	1	0.5	{ 0.02 }	{ 1.0 }			74HDXON
17.2							69HOCKENBURY
17.21 ± 0.04	1				0.02 ± 0.01		77FROEHNER
17.21	1		{ 0.03 }		0.026 ± 0.004		77PEREY
18.99	1	0.5	0.071	{ 1.0 }	0.067 ± 0.004		JENDL-2
19.0	1	0.5	0.075	1.0			JENDL-1
19.0	1	1.5	0.033	0.6			ENDF-B-4
19.03 ± 0.05	1				0.07 ± 0.01		BNL325(3)
19.0	0	0.5	0.067	1.0			73HDXON
19.03	1	0.5	0.071	{ 1.0 }			74HDXON
19.0					0.063 ± 0.010		69HOCKENBURY
19.03 ± 0.05	1				0.08 ± 0.02		72BEER
19.05 ± 0.04	1				0.08 ± 0.02		77FROEHNER
18.99	1		{ 0.081 }		0.067 ± 0.004		77PEREY
20.011	1	0.5	1.5	0.319	0.263 ± 0.008		JENDL-2
20.0	1	0.5	0.282	1.0			JENDL-1
20.0	1	1.5	0.12	0.6			ENDF-B-4
20.04 ± 0.05	1				0.22 ± 0.03		BNL325(3)
20.0	0	0.5	0.25	1.0			73HDXON

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
20.04	1	0.5	0.26	{ 1.0 }	0.20 ± 0.02	CCS = 26.0 ± 2.7	74MOXON
20.0					0.24 ± 0.05		69HOCKENBURY
20.04 ± 0.05	1	1			0.24 ± 0.05		72BEER
20.04 ± 0.04	1	1			0.24 ± 0.05		77FROEHNER
20.011 ± 6	1		R 1.5 ± 1	0.319	0.263 ± 0.008		77PEREY
21.123	1	0.5	4.7	0.782	0.670 ± 0.020		JENDL-2
21.1	1	1.5	0.388	1.0		GT = 1.388	JENDL-1
21.1	1	0.5	8.4	0.6		GT = 9.0	ENDF-B-4
21.16 ± 0.05	1	1			0.56 ± 0.06		BNL325(3)
21.1	0	0.5	1.27	1.0			73MOXON
21.16	1	0.5	1.28	{ 1.0 }			74MOXON
21.1					0.56 ± 0.06	CCS = 70 ± 7	69HOCKENBURY
21.16 ± 0.05	1	1			0.57 ± 0.11		72BEER
21.15 ± 0.04	1	1			0.61 ± 0.10		77FROEHNER
21.123 ± 3	1		R 4.7 ± 1	0.782	0.670 ± 0.020		77PEREY
26.04	1	0.5	0.37	{ 1.0 }	0.271 ± 0.007		JENDL-2
26.08	1	0.5	0.333	1.0		GT = 1.333	JENDL-1
26.08 ± 0.07	1	1			0.25 ± 0.05		BNL325(3)
26.08 ± 0.07	1	0.5	0.33	{ 1.0 }	0.25 ± 0.05		74MOXON
26.08 ± 0.07	1	1			0.25 ± 0.05		72BEER
26.08 ± 0.04	1	1			0.27 ± 0.05		77FROEHNER
26.04	1		R 0.6)		0.271 ± 0.007		77PEREY
26.615	1	0.5	2.1	1.4	0.847 ± 0.017		JENDL-2
26.6	1	0.5	2.33	1.0		GT = 3.33	JENDL-1
26.6	1	1.5	0.84	0.6		GT = 1.44	ENDF-B-4
26.67 ± 0.07	1	1			0.70 ± 0.07		BNL325(3)
26.6	0	0.5	2.33	1.0			73MOXON
26.67	0	0.5	1.1	{ 2.0 }			74MOXON
26.6					0.70 ± 0.07	CCS = 68 ± 7	69HOCKENBURY
26.67 ± 0.07	1	1			0.73 ± 0.14		72BEER
26.65 ± 0.04	1	1			0.78 ± 0.15		77FROEHNER
26.615 ± 1	1		R 2.8 ± 0.4	1.216	0.847 ± 0.017		77PEREY
26.63	1		1.925 ± 0.5				77SYME1
26.63	1		2.03 ± 6.3				77SYME2
26.63	1		1.64 ± 0.34				77SYME3
27.62	1	0.5	{ 0.031	{ 1.0 }	0.031 ± 0.006		JENDL-2
27.62	1		R 0.031		0.031 ± 0.006		77PEREY
32.23	1	0.5	0.70	{ 1.0 }	0.413 ± 0.021		JENDL-2
32.23	1		R 4)				77PEREY
32.355	1	0.5	9.7	1.4	1.211 ± 0.041		JENDL-2
32.4	1	1.5	2.23	1.0		GT = 3.23	JENDL-1
32.4	1	1.5	2.57	1.0		GT = 3.57	ENDF-B-4
32.36 ± 0.08	1	1	{ 1.5)		1.38 ± 0.15		BNL325(3)
32.4	0	1.5	2.56	1.0			73MOXON
32.36	0	0.5	3.95	{ 2.0 }			74MOXON
32.4					1.44 ± 0.15	CCS = 114 ± 12	69HOCKENBURY
32.36 ± 0.08	1	1			1.26 ± 0.25		72BEER
32.34 ± 0.05	1	1			1.40 ± 0.25		77FROEHNER
32.355	1		R 18.4 ± 1.2	1.296	1.211 ± 0.041		77PEREY
32.38	1		15.0 ± 1.2				77SYME1
32.38	1		5.7 ± 0.9				77SYME2
32.38	1		5.7 ± 0.9				77SYME3
34.20	1	0.5	1.97	0.94	0.635 ± 0.025		JENDL-2
34.2	1	1.5	0.5	1.0		GT = 1.5	JENDL-1
34.2	1	1.5	0.71	0.6		GT = 1.31	ENDF-B-4
34.24 ± 0.08	1	1			0.65 ± 0.08		BNL325(3)
34.2	0	0.5	1.85	1.0			73MOXON
34.24	1	0.5	1.91	{ 1.0 }			74MOXON
34.2					0.65 ± 0.08	CCS = 49.5 ± 5.0	69HOCKENBURY
34.24 ± 0.08	1	1			0.69		72BEER
34.23 ± 0.05	1	1			0.70 ± 0.11		77FROEHNER
34.20	1		R 2)		0.635 ± 0.025		77PEREY
34.22	1		2.5 ± 1.0				77SYME2
34.22	1		1.84 ± 0.5				77SYME3
35.04	1	0.5	0.032	{ 1.0 }	0.031 ± 0.005		JENDL-2
35.04	1		R 0.031		0.031 ± 0.005		77PEREY
36.089	0	0.5	16.7	1.24	1.15 ± 0.043		JENDL-2
36.1	1	1.5	0.887	1.0		GT = 1.887	JENDL-1
36.1	1	1.5	1.43	0.6		GT = 2.03	ENDF-B-4
36.12 ± 0.09	1	1			0.86 ± 0.10		BNL325(3)
36.2	0	0.5	6.12	1.0			73MOXON
36.12	0	0.5	1.60	{ 2.0 }			74MOXON
36.1					0.86 ± 0.10	CCS = 62 ± 7	69HOCKENBURY
36.12 ± 0.09	1	1			1.01		72BEER
36.12 ± 0.05	1	1			0.99 ± 0.15		77FROEHNER
36.089 ± 0.002	0		14.9 ± 1.2	1.321	1.214 ± 0.043		77PEREY
36.102 ± 0.0037	0		16.97 ± 0.5				77SYME
39.51	1	0.5	2.22	1.1	0.74 ± 0.028		JENDL-2
39.59	1	1.5	0.493	1.0			JENDL-1
39.59 ± 0.10	1	1			0.66 ± 0.15		BNL325(3)
39.5	0	0.5	{ 1.3)	1.0			73MOXON
39.59 ± 0.10	0	0.5	1.95	{ 1.0 }	0.66 ± 0.13		74MOXON
39.5							69HOCKENBURY
39.59 ± 0.10	1	1			0.66		72BEER
39.55 ± 0.06	1	1			0.64 ± 0.10		77FROEHNER
39.51	1		R 2)		0.750 ± 0.028		77PEREY

ENERGY (KEV.)	L	J	NEUTRON WIDTH (EV.)	GAMMA WIDTH (EV.)	WHS (EV.)	MISCELLANEOUS	REFERENCE
39.54	1		2.25 ± 0.45				
39.54	1		2.21 ± 0.8				77SYME1
39.54	1		2.21 ± 0.36				77SYME2
39.54	1						77SYME3
43.95	1	0.5	0.12	{ 1.0 }	0.111 ± 0.009		JENDL-2
43.88 ± 0.06	1			{ 0.1 }	0.14 ± 0.03		77FROEHNERR
43.95	1				0.111 ± 0.009		77PEREY
47.849	1	0.5	7.7	1.46	1.231 ± 0.039		JENDL-2
47.9	1	1.5	1.85	1.0		GT = 2.86	JENDL-1
47.9	1	1.5	3.76	1.0		GT = 4.76	ENDF-B-4
47.8 ± 0.2	1	1.5			1.3 ± 0.2		BNL325(3)
47.9	0	1.5	3.75	1.0			73MOXON
47.80	0	1	0.5	{ 2.0 }			74MOXON
47.9					1.58 ± 0.18	GGS = 87.5 ± 11.0	69HOCKENBURY
47.8 ± 0.15	1	1			0.98		72BEER
47.81 ± 0.07	1				1.03 ± 0.15		77FROEHNERR
47.849 ± 1	1		6.8 ± 0.6	1.504	1.231 ± 0.039		77PEREY
47.875	1		8.9 ± 0.85				77SYME1
47.875	1		11.5 ± 1.3				77SYME2
47.875	1		7.0 ± 0.8				77SYME3
51.85	1	1.5	0.85	{ 1.0 }	0.920 ± 0.100		JENDL-2
51.85	1		{ 2 }		0.920 ± 0.100		77PEREY
52.16	1	1.5	1.36	0.7	0.923 ± 0.043		JENDL-2
52.0	1	1.5	3.0	1.0		GT = 4.0	JENDL-1
52.0 ± 0.2	1	1	{ 1.5 }		1.5 ± 0.3		BNL325(3)
52.0 ± 0.15	0	1	0.5	{ 2.0 }	1.45 ± 0.30		74MOXON
52.1					1.46		69HOCKENBURY
52.00 ± 0.15	1	1			1.70 ± 0.30		72BEER
52.01 ± 0.08	1				0.923 ± 0.043		77PEREY
52.16			{ 2 }				77SYME1
52.2	1		1.265 ± 0.6				77SYME2
52.2	1		0.42 ± 0.8				77SYME3
52.2	1		2.25 ± 0.5				
52.26	1		0.53 ± 0.45				77SYME1
54.71	1	0.5	{ 0.3 }	{ 1.0 }	0.23 ± 0.022		JENDL-2
54.8	1	0.5	0.43	1.0		GT = 1.43	JENDL-1
54.8	1	0.5	0.69	0.6		GT = 1.29	ENDF-B-4
54.1 ± 0.2	1	1			0.30 ± 0.10		BNL325(3)
54.8					0.32 ± 0.10		69HOCKENBURY
54.70 ± 0.15	1	1			0.28		72BEER
54.64 ± 0.08	1				0.30 ± 0.05		77FROEHNERR
54.71	1		{ 0.3 }		0.213 ± 0.022		77PEREY
58.637	1	0.5	1.77	0.99	0.634 ± 0.031		JENDL-2
58.6	1	1.5	0.351	1.0		GT = 1.351	JENDL-1
58.6 ± 0.2	1	1			0.52 ± 0.15		BNL325(3)
58.60 ± 0.15	1	1			0.52		72BEER
58.60 ± 0.10	1				0.60 ± 0.09		77FROEHNERR
58.637 ± 6	1		1.46 ± 0.4	1.122	0.634 ± 0.031		77PEREY
58.69	1		2.2 ± 0.5				77SYME1
58.69	1		2.6 ± 1.4				77SYME2
58.69	1		1.7 ± 0.55				77SYME3
60.080	1	0.5	21.4	0.707	0.684 ± 0.017		JENDL-2
60.1	1	1.5	0.282	1.0		GT = 1.282	JENDL-1
60.1 ± 0.2	1	1			0.44		BNL325(3)
60.1					0.44		69HOCKENBURY
60.10 ± 0.15	1	1			0.64 ± 0.09		72BEER
60.10 ± 0.10	1				0.64 ± 0.09		77FROEHNERR
60.080 ± 1	1		{ 21.4 } ± 0.8	0.707	0.684 ± 0.017		77PEREY
60.12	1		20.3 ± 1.0				77SYME1
60.12	1		21.2 ± 3.0				77SYME2
60.12	1		21.2 ± 3.0				77SYME3
61.719	1	0.5	15.5	1.58	1.43 ± 0.045		JENDL-2
61.8	1	1.5	0.55	1.0		GT = 1.55	JENDL-1
61.8 ± 0.2	1	1			0.71 ± 0.15		BNL325(3)
61.8					0.71		69HOCKENBURY
61.75 ± 0.15	1	1			1.11 ± 0.16		72BEER
61.75 ± 0.10	1				1.433 ± 0.045		77FROEHNERR
61.719 ± 2	1		14.7 ± 1.1	1.588			77PEREY
61.76	1		17.5 ± 2.0				77SYME1
61.76	1		16.1 ± 3.5				77SYME2
61.76	1		16.1 ± 2.4				77SYME3
62.94	0	0.5	3550.0	2.3			JENDL-2
63.0	0	0.5	3600.0	3.2			JENDL-1
62.8	0	0.5	3300.0	2.14			ENDF-B-4
63.0 ± 0.2	0	0.5	3600 ± 200	3.2 ± 0.8			BNL325(3)
63.2	0		3580	2.0			73MOXON
63.098	0	0.5	3580	3.2			74MOXON
60.0	0						66FARRELL
63.2 ± 0.3	0		3650 ± 330				71GARG
63.0 ± 0.20	0		3600 ± 200	3.2 ± 0.8			72BEER
63.0 ± 0.2	0		3600 ± 200	2.3 ± 0.3			75FROEHNERR
63.0 ± 0.3	0	0.5	3600	2.2 ± 0.3			77FROEHNERR
63.209 ± 0.07	0		3550 ± 100				77PEREY
62.9406 ± 0.0153	0		3441.4 ± 0.19				77SYME
66.39	1	0.5	1.7	{ 1.0 }	0.629 ± 0.035		JENDL-2

JAERI-M 85-101

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	MWS (EV)	MISCELLANEOUS	REFERENCE
66.4	1	0.5	0.56	1.0		GT = 1.56	JENDL-1 BNL325(3) 69HOCKENBURY 72BEER 77FROEHNERR 77PEREY
66.4 ± 0.2	1	1			0.36		
66.4	1	1			0.36		
66.40 ± 0.15	1	1			0.55 ± 0.08		
66.40 ± 0.13	1	1	1.5		0.629 ± 0.035		
66.39	1	1	1.5				
68.60	1	0.5	0.3	(1.0)	0.23 ± 0.021	GT = 1.182	JENDL-2 JENDL-1 BNL325(3) 72BEER 77FROEHNERR 77PEREY
68.8	1	0.5	0.182	1.0			
68.8 ± 0.2	1	1			0.24		
68.75 ± 0.20	1	1			0.24		
68.56 ± 0.14	1	1			0.30 ± 0.05		
68.60	1	1	0.2		0.220 ± 0.021		
69.835	1	0.5	6.8	0.59	0.544 ± 0.030		JENDL-2
69.8	1	1.5	0.298	1.0		GT = 1.298	JENDL-1 BNL325(3) 72BEER 77FROEHNERR 77PEREY
69.8 ± 0.2	1	1			0.46		
69.80 ± 0.20	1	1			0.46		
69.81 ± 0.15	1	1			0.55 ± 0.09		
69.835 ± 3	1	1	5.5 ± 1	0.593	0.544 ± 0.030		
69.89	1	1	13.8 ± 2.8				
69.89	1	1	9.8 ± 3.0				
69.89	1	1	5.1 ± 1.2				
77.97	1	0.5	0.28	(1.0)	0.216 ± 0.022		JENDL-2
78.0	1	0.5	0.136	1.0			JENDL-1 BNL325(3) 69HOCKENBURY 72BEER
78.0 ± 0.2	1	1			0.12 ± 0.03		
78.2	1	1			0.12 ± 0.03		
77.95 ± 0.20	1	1			0.18 ± 0.05		
77.86 ± 0.15	1	1			0.216 ± 0.022		
77.97	1	1	0.3				
81.22	1	1.5	1.43	(1.0)	1.176 ± 0.050		JENDL-2
81.1	1	1.5	0.575	1.0			JENDL-1 BNL325(3) 69HOCKENBURY 72BEER
81.1 ± 0.2	1	1			0.73		
81.3	1	1			0.73		
81.10 ± 0.20	1	1			1.08 ± 0.20		
81.10 ± 0.15	1	1			1.176 ± 0.050		
81.22	1	1	2.5				
82.778	1	0.5	65.0	2.44	2.350 ± 0.087		JENDL-2
82.7 ± 0.3	1	1	72.3 ± 3	2.429	2.0 ± 0.5		77FROEHNERR 77PEREY
82.778 ± 2	1	1	60.0 ± 3.0		2.350 ± 0.087		77SYME1
82.84	1	1	74.0 ± 6.0				77SYME2
82.84	1	1	63.0 ± 3.0				77SYME3
83.1 ± 0.2	0	110	± 40	3.5 ± 0.7		WGH = 0.38 ± 0.14	
83.10 ± 0.20	0	0.5	110 ± 40	3.5 ± 0.7			BNL325(3) 74MOXON 69HOCKENBURY 72BEER
83.0	0	110	± 40	3.5 ± 0.7			
83.10 ± 0.20	0	110	± 40	3.5 ± 0.7			
83.29	1	0.5	2.3	(1.0)	0.696 ± 0.043		JENDL-2
83.29	1	1	2		0.696 ± 0.043		77PEREY
83.750	1	0.5	36.0	1.38	1.329 ± 0.059		JENDL-2
83.6 ± 0.3	1	1	41 ± 2	1.374	1.5 ± 0.4		77FROEHNERR 77PEREY
83.750 ± 2	1	1	25.4 ± 2.9		1.329 ± 0.059		77SYME1
83.82	1	1	42.1 ± 7.2				77SYME2
83.82	1	1	36.4 ± 2.2				77SYME3
84.77	1	0.5	0.2	(1.0)	0.164 ± 0.022		JENDL-2
84.77	1	1	0.2		0.164 ± 0.022		77PEREY
89.890	1	0.5	8.1	0.88	0.79 ± 0.048		JENDL-2
89.8	1	1.5	0.29	1.0			JENDL-1 BNL325(3) 72BEER
89.8 ± 0.2	1	1			0.45		
89.84 ± 0.20	1	1			0.45		
89.78 ± 0.20	1	1			0.69 ± 0.10		
89.890 ± 7	1	1	7.3 ± 1.2	0.892	0.795 ± 0.048		
89.93	1	1	6.1 ± 3.0				77SYME1
89.93	1	1	13.0 ± 3.0				77SYME2
89.93	1	1	8.8 ± 1.5				77SYME3
92.60	1	0.5	0.22	(1.0)	0.18 ± 0.024		JENDL-2
92.3	1	0.5	0.205	1.0			JENDL-1 BNL325(3) 72BEER
92.3 ± 0.2	1	1			0.17		
92.25 ± 0.20	1	1			0.17		
92.35 ± 0.22	1	1			0.25 ± 0.04		
92.60	1	1	0.2		0.153 ± 0.024		
95.55	1	1.5	1.5	(1.0)	1.20 ± 0.068		JENDL-2
94.5	1	1.5	0.818	1.0			JENDL-1 BNL325(3) 72BEER
94.5 ± 0.3	1	1			0.9 ± 0.2		
94.45 ± 0.25	1	1			0.9 ± 0.2		
95.55 ± 0.25	1	1			1.05 ± 0.15		
95.55	1	1	2.5		1.207 ± 0.068		
95.9							69HOCKENBURY
96.84	1	0.5	3.4	0.60	0.51 ± 0.039		JENDL-2
97.0	1	1.5	0.333	1.0			JENDL-1 BNL325(3) 72BEER
97.0 ± 0.3	1	1			0.5 ± 0.1		
97.00 ± 0.25	1	1			0.5 ± 0.1		
97.00 ± 0.26	1	1			0.66 ± 0.10		

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
96.84	1		1		0.486 ± 0.039		77PEREY
96.88	1		4.1 ± 2.0				77SYME1
96.88	1		1.3 ± 2.2				77SYME2
96.88	1		3.7 ± 1.05				77SYME3
97.487	1	0.5	14.7	0.385	0.375		JENDL-2
97.487 ± 4	1		14.7 ± 1.5	0.385	0.375 ± 0.033		77PEREY
97.577	1		15.5 ± 1.9				77SYME1
97.577	1		13.2 ± 1.32				77SYME2
97.577	1		11.2 ± 0.7				77SYME3
101.295	1	0.5	4.5	1.62	1.189 ± 0.055	GT = 2.0	JENDL-2
101.1	1		1.5	1.0	1.0 ± 0.2		JENDL-1
101.1 ± 0.3	1						BNL325(3)
101.							69HOCKENBURY
101.10 ± 0.25	1				1.0 ± 0.2		72BEER
101.1 ± 0.27	1				0.95 ± 0.24		77FROEHNERR
101.295 ± 6	1		7.1 ± 1.4	1.428	1.189 ± 0.055		77PEREY
101.38	1		11.25 ± 3.25				77SYME1
101.38	1		2.95 ± 1.6				77SYME2
101.38	1		3.15 ± 0.98				77SYME3
105.315	1	0.5	18.4	2.24	2.0 ± 0.132	GT = 10.0	JENDL-2
105.3	1		9.0	1.0	1.8 ± 0.4		JENDL-1
105.3 ± 0.3	1						BNL325(3)
105.							69HOCKENBURY
105.30 ± 0.25	1				1.8 ± 0.4		72BEER
105.3 ± 0.28	1				1.60 ± 0.40		77FROEHNERR
105.315 ± 4	1		14.6 ± 1.4	2.400	2.060 ± 0.132		77PEREY
105.38	1		25.4 ± 2.3				77SYME1
105.38	1		27.7 ± 6.3				77SYME2
105.38	1		21.7 ± 3.3				77SYME3
107.61	1	0.5	7.7	1.88	1.377 ± 0.098		JENDL-2
107.61	1		1.3		1.377 ± 0.098		77PEREY
107.74	1		7.7 ± 1.9				77SYME1
108.188	0	0.5	1100.0	3.8		GT = 1403.5 GT = 1002.1 WCH = 4.27 ± 0.61 GT = 2000 CNO = 6.114	JENDL-2
107.7	0	0.5	1400.0	3.5			JENDL-1
107.0	0	0.5	1000.0	2.14			ENDF-B-4
107.7 ± 0.5	0	0.5	1400 ± 200	3.5 ± 0.8			BNL325(3)
108.0	0		1280	2.0			73MOXON
107.9	0	0.5	1253	3.5			74MOXON
107.0	0						66FARRELL
107							69HOCKENBURY
108.0 ± 0.5	0		1470 ± 170				71GARG
107.7 ± 0.5	0		1500 ± 300	3.5 ± 0.8			72BEER
107.6 ± 0.3	0		1400 ± 300	3.8 ± 0.9			75FROEHNERR
107.7 ± 0.5	0	0.5	1500	3.8 ± 0.8			77PEREY
108.188 ± 0.014	0		1100 ± 50				77SYME
108.149 ± 0.0103	0		1071.82 ± 0.35				
110.589	1	0.5	4.8	0.995	0.823 ± 0.052	GT = 2.86	JENDL-2
110.7	1		1.5	1.86	1.0		JENDL-1
110.7 ± 0.3	1				1.3 ± 0.3		BNL325(3)
110.							69HOCKENBURY
110.7 ± 0.3	1				1.3 ± 0.3		72BEER
110.7 ± 0.3	1				1.1 ± 0.3		77FROEHNERR
110.589 ± 8	1		4.8 ± 1.5	0.995	0.823 ± 0.052		77PEREY
110.67	1		4.3 ± 2.5				77SYME1
110.67	1		9.8 ± 3.5				77SYME2
110.67	1		4.5 ± 1.5				77SYME3
111.30	1	1.5	0.86	1.0	0.927 ± 0.058		JENDL-2
111.30	1		1.3		0.927 ± 0.058		77PEREY
117.733	1	0.5	10.6	1.028	0.939 ± 0.061	GT = 1.667	JENDL-2
117.5	1		0.667	1.0			JENDL-1
117.5 ± 0.3	1				0.8 ± 0.3		BNL325(3)
117.5 ± 0.3	1				0.6 ± 0.3		72BEER
117.5 ± 0.3	1				0.75 ± 0.25		77FROEHNERR
117.733 ± 5	1		10.6 ± 1.4	1.028	0.939 ± 0.061		77PEREY
117.82	1		15.2 ± 2.5				77SYME1
117.82	1		16.9 ± 3.4				77SYME2
117.82	1		10.1 ± 0.7				77SYME3
118.5	1	1.5	4.2	0.7	1.2 ± 0.4		JENDL-2
118.5 ± 0.3	1				1.2 ± 0.4		77FROEHNERR
118.07	1		4.2 ± 1.5				77SYME1
119.648	1	1.5	7.2	1.61	2.634 ± 0.114		JENDL-2
120.3 ± 0.3	0	0.5		3.3 ± 0.6		GT = 632.0 GT = 702.14 WCH = 1.98 ± 0.57	BNL325(3)
120.					3.3 ± 0.6		69HOCKENBURY
120.3 ± 0.3	0	1			2.4 ± 0.8		72BEER
120.0 ± 0.3	1				2.4 ± 0.8		77FROEHNERR
119.648 ± 4	1		6.5 ± 1.3	4.421	2.634 ± 0.114		77PEREY
119.75	1		8.2 ± 2.4				77SYME1
119.75	1		12.8 ± 3.0				77SYME2
119.75	1		6.6 ± 1.2				77SYME3
123.381	0	0.5	435.0	3.5			JENDL-2
123.8	0	0.5	630.0	2.0			JENDL-1
122.5	0	0.5	700.0	2.14			ENDF-B-4
125.0 ± 0.5	0	0.5	700 ± 200	3.2 ± 0.6			BNL325(3)
123.8	0		630	2.0			73MOXON

JAERI-M 85-101

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WNS (EV)	MISCELLANEOUS	REFERENCE
124.34	0	1	0.5	617	3.2		
122.5	0					GT = 1000 GNC = 2.857	74MOXON 66FARRELL
124						GNC = 2.10 ± 0.57	69HOCKENBURY
123.8 ± 0.6	0			740 ± 200			71GRG
125.0 ± 0.5	0			750 ± 250	3.2 ± 0.6		72BEER
124.0 ± 0.5	0			700 ± 250	3.5 ± 0.6		75FROEHNERR
124.0 ± 0.5	0	0.5		750 ± 250	3.5 ± 0.6		77FROEHNERR
123.381 ± 0.0081	0			435.0 ± 0.35			77SYME
125.27	1	0.5		4.1	(1.0)		JENDL-2
125.27	1			10.8 ± 3.2			77SYME1
125.27	1			5.1 ± 3.0			77SYME2
125.27	1			3.0 ± 1.2			77SYME3
126.83	1	0.5		7.3	(1.0)		JENDL-2
126.83	1			7.3 ± 2.0			77SYME1
128.29	1	0.5		7.4	(1.0)		JENDL-2
128.29	1			7.4 ± 2.4			77SYME1
130.2	1	0.5	20.5		0.71	0.69 ± 0.14	JENDL-2
130.2 ± 0.4	1					0.69 ± 0.14	77FROEHNERR
129.91	1			20.0 ± 3.2			77SYME1
129.91	1			19.65 ± 3.2			77SYME2
129.91	1			20.95 ± 1.6			77SYME3
133.55	1	1.5	21.0		1.05	2.0 ± 0.4	JENDL-2
133.0 ± 0.4	1					2.0 ± 0.4	77FROEHNERR
133.55	1			18.5 ± 2.9			77SYME1
133.55	1			179.0 ± 3.5			77SYME2
133.55	1			22.1 ± 1.8			77SYME3
135.72	1	0.5	9.4		(1.0)		JENDL-2
135.72	1			9.4 ± 2.5			77SYME1
136.07	1	0.5	10.6		(1.0)		JENDL-2
136.07	1			10.6 ± 2.5			77SYME1
137.319	0	0.5	2617.28		2.2		JENDL-2
137.5	0	0.5	1760.0		2.0		JENDL-1
136.0	0	0.5	2200.0		2.14		ENDF-B-4
137.5 ± 0.7	0	0.5	1760 ± 200				BNL325(3)
137.5	0		1760		2.0		73MOXON
137.5	0	0.5	1760		(2.0)		74MOXON
136.0	0						66FARRELL
137.5 ± 0.7	0		1760 ± 200				71GRG
136.8 ± 0.7	0	0.5	1760 ± 200				77FROEHNERR
137.319 ± 0.019	0		2617.28 ± 0.52				77SYME
139.913	0	0.5	2857.4		2.2		JENDL-2
140.5	0	0.5	3460.0		2.0		JENDL-1
138.5	0	0.5	3000.0		2.14		ENDF-B-4
140.5 ± 0.8	0	0.5	3460 ± 500				BNL325(3)
140.5	0		3460		2.0		73MOXON
140.5	0	0.5	3460		(2.0)		74MOXON
138.5	0						66FARRELL
140.5 ± 0.8	0		3460 ± 490				71GRG
139.7 ± 0.7	0	0.5	3460 ± 490				77FROEHNERR
139.913 ± 0.021	0		2867.4 ± 0.62				77SYME
145.14	1	1.5	133		1.2	2.4 ± 0.46	JENDL-2
142.9 ± 0.8	1					2.4 ± 0.46	77FROEHNERR
145.14	1		141.0 ± 5.1				77SYME1
145.14	1		150.0 ± 10.0				77SYME2
145.14	1		121.0 ± 5.0				77SYME3
148.73	1	1.5	136.5		1.25	2.5 ± 0.5	JENDL-2
148.74	1	0.5	160.0		1.0		JENDL-1
147.5	1	0.5	160.0		0.6		ENDF-B-4
147.5 ± 0.8	1		175				BNL325(3)
148.74	0	0.5	160		1.0		73MOXON
147.5	0	0.5	175 ± 15				66FARRELL
146.5 ± 0.9	1		175 ± 15				77FROEHNERR
148.73	1		136.5 ± 3.5				77SYME1
148.73	1		141.0 ± 9.2				77SYME2
148.73	1		136.0 ± 5.0				77SYME3
151.32	1	1.5	19.1		(0.9)	1.7	JENDL-2
150.5 ± 1.0	1					1.7 ± 0.4	77FROEHNERR
151.32	1		15.75 ± 5.0				77SYME1
151.32	1		20.8 ± 3.3				77SYME2
151.32	1		19.1 ± 1.6				77SYME3
151.73	1	0.5	7.5		(1.0)		JENDL-2
151.73	1		7.5 ± 4.0				77SYME1
156.5	1	0.5	89.0		(1.0)		JENDL-2
156.5	1		75.0 ± 18.0				77SYME2
156.5	1		95.0 ± 12.0				77SYME3
156.92	1	0.5	56.0		(1.0)		JENDL-2
156.92	1		50.0 ± 6.0				77SYME1
156.92	1		63.0 ± 24.0				77SYME2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	MWS (EV)	MISCELLANEOUS	REFERENCE
156.92	1		102.0 ± 18.0				77SYME3
157.251	0	0.5	4960.9	3.0			JENDL-2
159.5	0	0.5	6000.0	2.0			JENDL-1
157.4	0	0.5	6250.0	2.14			ENDF-B-4
159.5 ± 0.9	0	0.5	6000 ± 1000			WCH = 15.02 ± 2.50	BNL325131
159.0	0		5640	2.0			73MOXON
159.0	0	0.5	7010	(2.0)			74MOXON
157.0	0						66FARRELL
159 ± 1	0		7380 ± 2110				71GARG
159.5 ± 2.0	0	0.5	6000 ± 6000	3.0 ± 1.0			77FROEHNERR
157.251 ± 0.032	0		4960.9 ± 0.66				77SYME
161.12	1	1.5	17.2	1.83	3.3		JENDL-2
161.0 ± 1.2	1		17.2 ± 3.1		3.3 ± 1.1		77FROEHNERR
161.12	1						77SYME1
165.97	1	0.5	40.0	(1.0)			JENDL-2
165.97	1		40.0 ± 4.0				77SYME1
165.97	1		32.8 ± 7.3				77SYME2
165.97	1		41.3 ± 4.1				77SYME3
166.98	1	1.5	32.0	1.03	2.0 ± 1.0		JENDL-2
167.0 ± 1.3	1		32.0 ± 3.5		2.0 ± 1.0		77FROEHNERR
166.98	1						77SYME1
168.675	0	1	319.73	2.5			JENDL-2
169.0	0	0.5	750.0	2.0			JENDL-1
167.5	0	0.5	500.0	2.14			ENDF-B-4
169.0 ± 1.0	0	0.5	750 ± 220			WCH = 1.82 ± 0.54	BNL325131
169.0	0		640	2.0			73MOXON
169.0	0	0.5	640	(2.0)			74MOXON
167.5	0						66FARRELL
169 ± 1	0		870 ± 220				71GARG
169.0 ± 2	0		750 ± 220	B 2.5 ± 1.0			77FROEHNERR
168.675 ± 0.014	0		319.73 ± 0.62				77SYME
175.14	1	1.5	72.5	1.5	3.0		JENDL-2
173.5 ± 1.5	1		76.5 ± 5.7		3.0 ± 1.0		77FROEHNERR
175.14	1		65.5 ± 10.0				77SYME1
175.14	1		70.5 ± 5.0				77SYME2
175.14	1						77SYME3
180.13	1	0.5	14.5	(1.0)			JENDL-2
180.13	1		14.5 ± 8.8				77SYME1
180.59	1	0.5	15.2	(1.0)			JENDL-2
180.59	1		31.0 ± 12.0				77SYME1
180.59	1		14.1 ± 2.5				77SYME2
180.59	1		18.2 ± 6.4				77SYME3
181.28	1	0.5	13.7	(1.0)			JENDL-2
181.28	1		10.8 ± 6.8				77SYME1
181.28	1		12.8 ± 2.9				77SYME2
181.28	1		21.7 ± 6.7				77SYME3
182.9	1	0.5	22.0	(1.0)			JENDL-2
182.9	1		20.5 ± 6.0				77SYME2
182.9	1		23.0 ± 2.5				77SYME3
184.53	1	1.5	140.3	4.0	8.0		JENDL-2
184.74	1	0.5	227.0	1.0			JENDL-1
183.5	1	1.5	127.0	0.6			ENOF-B-4
183.5 ± 1.1	0		250				BNL325131
184.74	0	0.5	227	1.0			73MOXON
183.5	0		248.5 ± 21.5				66FARRELL
183.8 ± 1.7	1		131.0 ± 9.0		8.0 ± 3.0		77FROEHNERR
184.53	1		161.0 ± 10.3				77SYME1
184.53	1		138.0 ± 4.1				77SYME2
184.53	1						77SYME3
185.91	1	0.5	32.5	(1.0)			JENDL-2
185.91	1		33.0 ± 4.0				77SYME1
185.91	1		47.4 ± 8.0				77SYME2
185.91	1		30.0 ± 3.0				77SYME3
191.415	0	0.5	2381.32	3.0			JENDL-2
193.0	0	0.5	3500.0	2.0			JENDL-1
190.5	0	0.5	3000.0	2.14			ENOF-B-4
193.0 ± 1.2	0	0.5	3500 ± 500			WCH = 7.97 ± 1.14	BNL325131
192.0	0		3620	2.0			73MOXON
192.0	0	0.5	3620	(2.0)			74MOXON
190.5	0						66FARRELL
192 ± 1	0		4050 ± 580				71GARG
193.0 ± 2.0	0	0.5	3500 ± 640	B 3.0 ± 1.0			77FROEHNERR
191.415 ± 0.021	0		2381.32 ± 0.64				77SYME
196.08	1	0.5	8.3	(1.0)			JENDL-2
196.08	1		8.7 ± 3.9				77SYME1
196.08	1		27.0 ± 8.5				77SYME2
196.08	1		8.0 ± 2.5				77SYME3
198.05	1	1.5	22.5	(1.9)	3.5		JENDL-2

JAERI-M 85-101

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	HW5 (EV)	MISCELLANEOUS	REFERENCE
199.0 ± 2.0					3.5 ± 1.2		77FROEHNERR 77SYME1
198.05	1	1	22.5 ± 5.0				
201.27	1	0.5	18.0	(1.0)			JENDL-2 77SYME1
201.27	1		18.0 ± 5.0				
202.43	1	0.5	11.5	(1.0)			JENDL-2 77SYME1
202.43	1		14.5 ± 4.5				77SYME2
202.43	1		16.0 ± 8.0				77SYME3
202.43	1		9.4 ± 3.1				
205.46	0	0.5	6518.8	4.5			JENDL-2
207.0	0	0.5	6800.0	2.0			JENDL-1
204.5	0	0.5	7500.0	2.14			ENDF-B-4
207.0 ± 1.5	0	0.5	6800 ± 1200				BNL325(3)
207.0	0		6820	2.0			73MOXON
207.0	0	0.5	6820	(2.0)			74MOXON
204.5	0						66FARRELL
207. ± 1.5	0		6030 ± 200				71CARGO
207.8 ± 2.5	0	0.5	6800	8 4.5 ± 2.0			77FROEHNERR
205.46 ± 0.045	0		6518.8 ± 0.87				77SYME
207.18	1	0.5	285	(1.0)			JENDL-2
207.18	1		382.0 ± 100.0				77SYME1
207.18	1		285.0 ± 100.0				77SYME2
207.18	1		185.0 ± 100.0				77SYME3
216.52	1	1.5	190.0	4.0	8.0 ± 3.0		JENDL-2
216.24	1	0.5	245.0	1.0			JENDL-1
215.8	1	0.5	245.0	0.6			ENDF-B-4
215.0 ± 1.5	1		260				BNL325(3)
216.24	0	0.5	245	1.0			73MOXON
215.0	0		262.5 ± 17.5				66FARRELL
215.8 ± 2.0	1		260		8 ± 3		77FROEHNERR
216.52	1		200.0 ± 14.6				77SYME1
216.52	1		180.0 ± 12.5				77SYME2
216.52	1		190.0 ± 17.0				77SYME3
217.9	1	0.5	23.0	(1.0)			JENDL-2
217.9	1		34.5 ± 9.5				77SYME1
217.9	1		17.6 ± 11.0				77SYME2
217.9	1		20.8 ± 5.0				77SYME3
230.9	1	0.5	78.5	(1.0)			JENDL-2
230.9	1		78.5 ± 11.0				77SYME1
232.289	0	0.5	4227.66	9.0			JENDL-2
232.24	0	0.5	6000.0	2.0			JENDL-1
231.0	0	0.5	6000.0	2.14			ENDF-B-4
231.0 ± 1.8	0	0.5	6000				BNL325(3)
232.24	0		6000	2.0			73MOXON
232.24	0	0.5	6000	(2.0)			74MOXON
231.0	0						66FARRELL
230.4 ± 3.0	0	0.5	6000	8 9 ± 4			77FROEHNERR
232.289 ± 0.033	0		4227.66 ± 0.87				77SYME
236.0	1	0.5	14.5	(1.0)			JENDL-2
236.0	1		6.7 ± 7.3				77SYME2
236.0	1		16.0 ± 3.1				77SYME3
242.65	1	0.5	36.0	(1.0)			JENDL-2
242.65	1		51.0 ± 14.0				77SYME1
242.65	1		23.7 ± 9.0				77SYME2
242.65	1		36.0 ± 4.3				77SYME3
243.85	1	0.5	24.0	(1.0)			JENDL-2
243.85	1		17.0 ± 9.0				77SYME1
243.85	1		25.0 ± 4.0				77SYME2
243.85	1						77SYME3
245.105	0	0.5	138.43	(2.0)			JENDL-2
244.24	0	0.5	250.0	2.0			JENDL-1
243.0	0	0.5	250.0	2.14			ENDF-B-4
243.0 ± 1.8	0	0.5	250				BNL325(3)
244.24	0		250	2.0			73MOXON
244.24	0	0.5	250	(2.0)			74MOXON
243.0	0						66FARRELL
245.105 ± 0.031	0		138.43 ± 0.94				77SYME
249.39	1	0.5	281.0	(1.0)			JENDL-2
248.74	1	0.5	343.0	1.0			JENDL-1
248.1	1	0.5	343.0	0.6			ENDF-B-4
247.5 ± 1.8	1		360				BNL325(3)
248.74	0	0.5	343	1.0			73MOXON
247.5	0		363.5 ± 20.5				66FARRELL
249.39	1		268.0 ± 20.0				77SYME1
249.39	1		290.0 ± 16.0				77SYME2
249.39	1		281.0 ± 8.0				77SYME3
250.625	1	0.5	45.0	(1.0)			JENDL-2
250.625	1		68.0 ± 16.0				77SYME1
250.625	1		57.5 ± 11.0				77SYME2
250.625	1		41.0 ± 4.4				77SYME3
254.18	1	0.5	23.5	(1.0)			JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GRMMA WIDTH (EV)	WNS (EV)	MISCELLANEOUS	REFERENCE
254.18	1		28.0 ±11.2				77SYME1
254.18	1		23.5 ±14.0				77SYME2
254.18	1		21.5 ± 6.5				77SYME3
254.85	1	0.5	32.0	(1.0)			JENDL-2
254.85	1		32.0 ±10.0				77SYME1
259.27	1	0.5	77.0	(1.0)			JENDL-2
258.24	1	0.5	75.0	1.0			JENDL-1
258.24	C	0.5	(75)	1.0			73MOXON
257.5	> 0						66FARRELL
259.27	1		77.0 ±15.0				77SYME1
267.7	1	0.5	36.0	(1.0)			JENDL-2
267.7	1		80.0 ±14.5				77SYME1
267.7	1		9.0 ±16.0				77SYME2
267.7	1		36.0 ± 8.5				77SYME3
269.44	1	0.5	71.0	(1.0)			JENDL-2
269.44	1		54.0 ±13.0				77SYME1
269.44	1		71.0 ±25.0				77SYME2
269.44	1		76.0 ± 9.0				77SYME3
271.608	0	0.5	5510.2	(2.0)			JENDL-2
271.24	0	0.5	6000.0	2.0			JENDL-1
271.5	0	0.5	7500.0	2.14			ENDF-B-4
270.0 ± 2.0	0	0.5	6000				BNL325(3)
271.24	0		6000	2.0			73MOXON
271.24	0	0.5	6000	(2.0)			74MOXON
270.0	0						66FARRELL
271.608 ± 0.042	0		5510.2 ± 1.0				77SYME
273.55	1	0.5	67.0	(1.0)			JENDL-2
273.55	1		67.0 ±10.0				77SYME1
277.3	1	0.5	81.0	(1.0)			JENDL-2
277.3	1		81.0 ±16.0				77SYME1
278.48	1	0.5	152.0	(1.0)			JENDL-2
278.48	1		114.5 ±16.0				77SYME1
278.48	1		142.0 ±22.0				77SYME2
278.48	1		169.0 ±10.0				77SYME3
280.997	0	0.5	1522.1	(2.0)			JENDL-2
279.24	0	0.5	2000.0	2.0			JENDL-1
279.0	0	0.5	800.0	2.14			ENDF-B-4
278.0 ± 2.0	0	0.5	2000				BNL325(3)
279.24	0		2000	2.0			73MOXON
278.0	0						66FARRELL
280.997 ± 0.027	0		1522.1 ± 1.1				77SYME
289.3	1	0.5	195.0	(1.0)			JENDL-2
287.74	1	0.5	200.0	1.0			JENDL-1
287.6	1	0.5	200.0	0.6			ENDF-B-4
286.5 ± 2.0	1		215				BNL325(3)
287.74	0	0.5	200	1.0			73MOXON
286.5	0		215 ± 15				66FARRELL
289.3	1		210.0 ±20.0				77SYME1
289.3	1		196.0 ±31.0				77SYME2
289.3	1		187.0 ±14.0				77SYME3
297.5	1	0.5	63.0	(1.0)			JENDL-2
297.5	1		61.5 ±14.0				77SYME2
297.5	1		64.0 ± 6.0				77SYME3
300.01	1	0.5	33.0	(1.0)			JENDL-2
300.01	1		50.0 ±20.0				77SYME1
300.01	1		30.0 ±14.0				77SYME2
300.01	1		32.0 ± 6.0				77SYME3
301.32	1	0.5	90.0	(1.0)			JENDL-2
301.32	1		151.0 ±26.0				77SYME1
301.32	1		93.0 ±30.0				77SYME2
301.32	1		51.0 ±20.0				77SYME3
304.74	0	0.5	750.0	(2.0)			JENDL-2
304.24	0	0.5	750.0	2.0			JENDL-1
304.0	0	0.5	750.0	2.14			ENDF-B-4
303.0 ± 2.0	0	0.5	750				BNL325(3)
304.24	0		750	2.0			73MOXON
304.74	0	0.5	750	(2.0)			74MOXON
303.5	0						66FARRELL
307.28	1	0.5	155.0	1.0			77SYME1
307.74	1	0.5	50.0	1.0			73MOXON
307.74	0	0.5	(50)	1.0			66FARRELL
306.5	> 0						77SYME1
307.28	1		155.0 ±22.0				77SYME2
316.97	1	0.5	71.0	1.0			JENDL-2
316.97	1		71.0 ±22.0				77SYME1
321.22	1	0.5	96.0	1.0			JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	MHS (EV)	MISCELLANEOUS	REFERENCE
321.22	1		96.0 ±25.0				77SYME1
325.2	0	0.5	2000.0	{ 2.0 }			JENDL-2
326.24	0	0.5	2000.0	2.0			JENDL-1
325.0	0	0.5	1500.0	2.14			ENDF-B-4
325.0 ± 2.0	0	0.5	2000				BNL325(3)
326.24	0	0.5	2000	2.0			73MOXON
326.24	0	0.5	2000	{ 2.0 }			74MOXON
325.0	0					GT = 2000	66FARRELL
						CNO = 3.508	
334.747	1	0.5	552.0	{ 1.0 }			JENDL-2
335.74	1	0.5	592.0	1.0			JENDL-1
333.2	1	0.5	592.0	0.6			ENDF-B-4
334.5 ± 2.5	■		624				BNL325(3)
335.74	0	0.5	592	1.0			73MOXON
334.5	0	0.5	624	±32			66FARRELL
334.747	1		552.0	±28.0			77SYME1
343.3	1	0.5	151.0	{ 1.0 }			JENDL-2
343.3	1		151.0	±18.2			77SYME1
344.1	1	0.5	222.0	{ 1.0 }			JENDL-2
344.74	1	0.5	560.0	1.0			JENDL-1
342.5	1	0.5	560.0	0.6			ENDF-B-4
343.5 ± 2.5	■		585				BNL325(3)
344.74	0	0.5	560	1.0			73MOXON
343.5	0	0.5	585	±25			66FARRELL
344.1	1		222.0	±22.0			77SYME1
349.0	0	0.5	1500.0	2.0			JENDL-2
350.24	0	0.5	1500.0	2.0			JENDL-1
348.0	0	0.5	1500.0	2.14			ENDF-B-4
349.0	0	0.5	1500				BNL325(3)
350.24	0		1500	2.0			73MOXON
350.24	0	0.5	1500	{ 2.0 }			74MOXON
349.0	0					GT = 1500	66FARRELL
						CNO = 2.539	
357.59	1	0.5	260.0	{ 1.0 }			JENDL-2
358.74	1	0.5	426.0	1.0			JENDL-1
357.5	1	0.5	426.0	0.6			ENDF-B-4
357.5	■		443				BNL325(3)
358.74	0	0.5	426	1.0			73MOXON
357.5	0	0.5	443	±17			66FARRELL
357.59	1		260.0	±32.0			77SYME1
359.7	1	0.5	148.0	{ 1.0 }			JENDL-2
359.7	1		148.0	±32.0			77SYME1
367.0	0	0.5	250.0	2.0			JENDL-2
368.24	0	0.5	250.0	2.0			JENDL-1
367.0	0	0.5	250.0	2.14			ENDF-B-4
367.0	0	0.5	250				BNL325(3)
368.24	0		250	2.0			73MOXON
368.24	0	0.5	250	{ 2.0 }			74MOXON
367.0	0					GT = 250	66FARRELL
						CNO = 0.413	
378.78	1	0.5	200.0	{ 1.0 }			JENDL-2
379.74	1	0.5	426.0	1.0			JENDL-1
377.5	1	0.5	480.0	0.6			ENDF-B-4
378.5	■		443				BNL325(3)
379.74	0	0.5	426	1.0			73MOXON
378.5	0	0.5	500	±20			66FARRELL
378.78	1		200.0	±24.0			77SYME1
379.75	1	0.5	175.0	{ 1.0 }			JENDL-2
379.75	1		175.0	±24.5			77SYME1
389.74	1	0.5	480.0	1.0			JENDL-1
387.5	■		500				BNL325(3)
389.74	0	0.5	480	1.0			73MOXON
394.5	0	0.5	750.0	2.0			JENDL-2
395.24	0	0.5	750.0	2.0			JENDL-1
392.8	0	0.5	1900.0	2.14			ENDF-B-4
394.0	0	0.5	750				BNL325(3)
395.24	0		750	2.0			73MOXON
395.24	0	0.5	750	{ 2.0 }			74MOXON
394.0	0					GT = 750	66FARRELL
						CNO = 1.195	
397.362	1	0.5	483.0	{ 1.0 }			JENDL-2
397.74	1	0.5	50.0	1.0			JENDL-1
397.74	0	0.5	50	1.0			73MOXON
396.5	0						66FARRELL
397.362	1		483.0	±36.0			77SYME1
406.63	1	0.5	80.0	{ 1.0 }			JENDL-2
406.63	1		80.0	±25.0			77SYMF1
409.43	1	0.5	120.0	{ 1.0 }			JENDL-2
409.43	1		120.0	±30.0			77SYME1
413.18	1	0.5	145.0	{ 1.0 }			JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WNS (EV)	MISCELLANEOUS	REFERENCE
414.24	1	0.5	50.0	1.0		GT = 51.0	JENDL-1
410.0	1	1.5	100.0	0.6		GT = 100.6	ENDF-B-4
414.24	0	0.5	(50)	1.0			73MOXON
413.0	0						66FARRELL
413.18	1		145.0	±56.0			77SYME1
416.04	1	0.5	510.0	(1.0)		GT = 21.0	JENDL-2
417.24	1	0.5	20.0	1.0			JENDL-1
417.24	0	0.5	(20)	1.0			73MOXON
416.0	0						66FARRELL
416.04	1		510.0	±36.0			77SYME1
418.74	0	0.5	5000.0	2.0		GT = 5002.0	JENDL-2
418.74	0	0.5	5000.0	2.0		GT = 10502.0	JENDL-1
415.0	0	0.5	10500.0	2.14		WCH = 7.74	ENDF-B-4
417.0	0	0.5	*5000				BNL325(3)
418.74	0		5000	2.0			73MOXON
418.74	0	0.5	5000	(2.0)		GT = 5000	74MOXON
417.5	0					CNO = 7.738	66FARRELL
429.56	1	0.5	180.0	(1.0)			JENDL-2
427.24	1	1.5	900.0	1.0		GT = 901.0	JENDL-1
426.0	1	1.5	*1830	±400		WGI = 5.2	BNL325(3)
427.24	0	0.5	1800	1.0			73MOXON
426.0	0	1.5	*915	±15			66FARRELL
429.56	1		180.0	±31.0			77SYME1
425.0	0	0.5	8000.0	2.0		GT = 8002.0	JENDL-2
427.24	0	0.5	8000.0	2.0		GT = 8002.1	JENDL-1
423.6	0	0.5	8000.0	2.14		WCH = 12.25	ENDF-B-4
426.5	0	0.5	*8000				BNL325(3)
427.24	0		8000	2.0			73MOXON
427.24	0	0.5	8000	(2.0)		GT = 8000	74MOXON
426.5	0					CNO = 12.250	66FARRELL
432.7	1	0.5	195.0	1.0		GT = 21.0	JENDL-2
436.74	1	0.5	20.0	1.0			JENDL-1
436.74	0	0.5	(20)	1.0			73MOXON
435.5	0						66FARRELL
432.7	1		195.0	±30.0			77SYME1
446.59	1	0.5	248.0	1.0		GT = 21.0	JENDL-2
446.24	1	0.5	20.0	1.0			JENDL-1
446.24	0	0.5	(20)	1.0			73MOXON
446.0	0						66FARRELL
446.59	1		248.0	±36.0			77SYME1
451.85	1	0.5	370.0	1.0		GT = 21.0	JENDL-2
452.24	1	0.5	20.0	1.0			JENDL-1
452.24	0	0.5	(20)	1.0			73MOXON
451.0	0						66FARRELL
451.85	1		370.0	±36.0			77SYME1
454.74	0	0.5	3000.0	2.0		GT = 3002.0	JENDL-2
454.74	0	0.5	3000.0	2.0		GT = 2202.1	JENDL-1
455.25	0	0.5	2200.0	2.14		WCH = 4.45	ENDF-B-4
454.5	0	0.5	*3000				BNL325(3)
454.74	0		3000	2.0			73MOXON
455.74	0	0.5	3000	(2.0)		GT = 3000	74MOXON
454.5	0					CNO = 4.450	66FARRELL
458.74	1	0.5	75.0	1.0		GT = 76.0	JENDL-1
459.74	0	0.5	(75)	1.0			73MOXON
458.5	0						66FARRELL
462.1	0	0.5	750.0	2.0		GT = 752.0	JENDL-2
462.74	0	0.5	750.0	2.0		GT = 752.14	JENDL-1
461.5	0	0.5	750.0	2.14		WCH = 1.10	ENDF-B-4
461.5	0	0.5	*750				BNL325(3)
462.74	0		750	2.0			73MOXON
462.74	0	0.5	750	(2.0)		GT = 750	74MOXON
461.5	0					CNO = 1.104	66FARRELL
475.7	1	1.5	300.0	0.6		GT = 300.6	ENDF-B-4
480.0	1	0.5	100.0	0.6		GT = 100.6	ENDF-B-4
492.966	1	0.5	1675.0	(1.0)			
493.74	1	0.5	1987.0	1.0		GT = 1988.0	JENDL-2
492.2	1	0.5	1050.0	0.6		GT = 1050.6	JENDL-1
492.5	0		*2000				ENDF-B-4
493.74	0	0.5	1987	1.0			BNL325(3)
492.5	0						73MOXON
492.5	0	0.5	*2015.5	±29.5			66FARRELL
492.966	1		1675.0	±75.0			77SYME1
495.5	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-2
495.74	0	0.5	2000.0	2.0		GT = 2002.1	JENDL-1
495.2	0	0.5	*2000	2.14		WCH = 2.84	ENDF-B-4
495.5	0	0.5	*2000				BNL325(3)
496.74	0		2000	2.0			73MOXON
496.74	0	0.5	2000	(2.0)		GT = 2000	74MOXON
495.5	0					CNO = 2.841	66FARRELL

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WWS (EV)	MISCELLANEOUS	REFERENCE
507.2	0	0.5	2000.0	2.0			JENDL-2
508.24	0	0.5	2000.0	2.0			JENDL-1
507.0	0	0.5	2000.0	2.14		GT = 2002.0	ENDF-B-4
507.0	0	0.5	2000			GT = 2002.1	BNL325(3)
508.24	0	0.5	2000			WCH = 2.81	73MOXON
508.24	0	0.5	2000				74MOXON
507.0	0	0.5	2000				66FARRELL
						GT = 2000	
						GNO = 2.809	
509.24	1	0.5	75.0	1.0		GT = 75.0	JENDL-1
509.24	> 0	0.5	(75)	1.0			73MOXON
508.0	> 0						66FARRELL
513.74	1	0.5	100.0	1.0		GT = 101.0	JENDL-1
513.74	> 0	0.5	(100)	1.0			73MOXON
512.5	> 0						66FARRELL
523.0	0	0.5	750.0	2.0			JENDL-2
523.74	0	0.5	750.0	2.0		GT = 752.0	JENDL-1
522.5	0	0.5	750.0	2.14		GT = 752.14	ENDF-B-4
522.5	0	0.5	750			WCH = 1.04	BNL325(3)
523.74	0	0.5	750				73MOXON
523.74	0	0.5	750				74MOXON
522.5	0	0.5	750				66FARRELL
530.0	1	0.5	422.0	1.0			JENDL-2
531.24	1	0.5	422.0	1.0		GT = 423.0	JENDL-1
528.9	1	0.5	300.0	0.6		GT = 300.6	ENDF-B-4
530.0	*		(300)				BNL325(3)
531.24	> 0	0.5	422				73MOXON
530.0	> 0	0.5	(431)				66FARRELL
						GT = 431	
						± 9	
541.5	1	0.5	640.0	1.0			JENDL-2
545.24	1	0.5	640.0	1.0		GT = 641.0	JENDL-1
541.1	1	0.5	420.0	0.6		GT = 420.6	ENDF-B-4
544.0	*		(420)				BNL325(3)
545.24	> 0	0.5	640				73MOXON
544.0	> 0	0.5	640				66FARRELL
						GT = 640	
						± 2	
554.69	1	0.5	1325.0		(1.0)		JENDL-2
555.74	1	0.5	1600.0		1.0	GT = 1601.0	JENDL-1
553.9	1	0.5	400.0		0.6	GT = 400.6	ENDF-B-4
554.5	1	0.5	(1490)	± 300		WCH = 3.3	BNL325(3)
555.74	> 0	0.5	1600		1.0		73MOXON
554.5	> 0	0.5	(1615)	± 15			66FARRELL
554.69	1	0.5	1325.0	± 80.0			77SYME1
559.723	1	0.5	2025.0		(1.0)		JENDL-2
560.74	1	0.5	1260.0		1.0	GT = 1261.0	JENDL-1
559.2	1	1.5	140.0		0.6	GT = 140.6	ENDF-B-4
559.5	*		(1260)				BNL325(3)
560.74	> 0	0.5	1260		1.0		73MOXON
559.5	> 0	0.5	(1263)	± 3			66FARRELL
560.023	1	0.5	2025.0	± 95.0			77SYME1
568.0	0	0.5	10000.0	2.0			JENDL-2
572.24	0	0.5	10000.0	2.0		GT = 10002.0	JENDL-1
568.8	0	0.5	10500.0	2.14		GT = 10502.0	ENDF-B-4
571.0	0	0.5	(10000)			WCH = 13.23	BNL325(3)
572.24	0	0.5	10000				73MOXON
572.24	0	0.5	10000				74MOXON
571.0	0	0.5	(2025)				66FARRELL
						GT = 10000	
						GNO = 13.234	
575.5	1	0.5	300.0	0.6			JENDL-2
574.5	1	0.5	300.0	0.6		GT = 300.6	ENDF-B-4
588.5	0	0.5	2500.0	2.0			JENDL-2
588.74	0	0.5	2500.0	2.0		GT = 2502.0	JENDL-1
587.5	0	0.5	1900.0	2.14		GT = 1902.1	ENDF-B-4
588.5	0	0.5	(2500)			WCH = 3.26	BNL325(3)
588.74	0	0.5	2500				73MOXON
589.74	0	0.5	2500				74MOXON
588.5	0	0.5	(2500)				66FARRELL
						GT = 2500	
						GNO = 3.259	
600.0	0	0.5	6000.0	2.0			JENDL-2
601.24	0	0.5	6000.0	2.0		GT = 6002.0	JENDL-1
599.8	0	0.5	6000.0	2.14		GT = 6002.1	ENDF-B-4
600.0	0	0.5	(6000)			WCH = 7.75	BNL325(3)
601.24	0	0.5	6000				73MOXON
601.24	0	0.5	6000				74MOXON
600.0	0	0.5	(2025)				66FARRELL
						GT = 6000	
						GNO = 7.746	
603.975	1	0.5	925.0		(1.0)		JENDL-2
608.7	1	1.5	300.0		0.6	GT = 300.6	ENDF-B-4
603.975	1	0.5	925.0	± 82.0			77SYME1
612.0	1	1.5	200.0	0.6		GT = 200.6	ENDF-B-4
625.1	1	1.5	400.0	0.6		GT = 400.6	ENDF-B-4
629.0	1	0.5	130.0	0.6		GT = 130.6	ENDF-B-4
631.2	1	0.5	400.0	0.6		GT = 400.6	ENDF-B-4

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
635.3	0	0.5	3000.0	2.14		GT = 3002.1	ENDF-B-4
642.5 649.85	1 1	0.5	600.0 1900.0 ±300.0	0.6		GT = 600.6	ENDF-B-4 77SYME1

* A and B denote $g\Gamma_n$ and $g\Gamma_\gamma$, respectively

$$\begin{aligned} \text{** WW5} &= g\Gamma_n \Gamma_\gamma / \Gamma \quad (\text{eV}), & \text{GT} &= \Gamma \quad (\text{eV}) \\ \text{WGH} &= g\Gamma_n^{(0)} \quad (\text{eV}), & \text{GNO} &= \Gamma_n^{(0)} \quad (\text{eV}) \\ \text{WGI} &= g\Gamma_n^{(1)} \quad (\text{eV}), & \text{GGS} &= \sigma_0 \Gamma_\gamma \quad (\text{b} \cdot \text{eV}) \end{aligned}$$

References

- 66Farrell : Ref.(21)
- 69Hockenbury: Ref.(23)
- 71Garg : Ref.(49)
- 72Beer : Ref.(50)
- 73Moxon : Ref.(51) (Evaluation)
- 74Moxon : Ref.(3) (Evaluation)
- 75Fröhner : Ref.(52)
- 77Fröhner : Ref.(22)
- 77Perey : Ref.(19)
- 77Syme : Ref.(20)

Table 7 Resonance parameters of ^{60}Ni

ENERGY (KEV)	L	J	NEUTRON WIDTH *	GAMMA WIDTH * (EV)	WWS ** (EV)	MISCELLANEOUS **	REFERENCE
-5.5	0	0.5	222	2.85			JENDL-2
-5.50	0		52.5	5.5			JENDL-1
-5.50	0	0.5	52.5	5.5			73MOXON
							74MOXON
1.292	1	0.5	0.0003	1.0			JENDL-2
1.292	1	0.5	0.0003	1.0			JENDL-1
1.292	1	0.5	0.001	0.6			ENDF-B-4
1.293 \pm 0.009					0.0003 \pm 0.0001		BNL325(3)
1.292	0	0.5	0.0003	1.0			73MOXON
1.292 \pm 0.004	1	0.5	0.0003	(1.0)	0.0003 \pm 0.0001		74MOXON
1.294							69HOCKENBURY
1.292 \pm 0.004	1				0.0003 \pm 0.0001		70STIEGLITZ
2.257	1	0.5	0.073	1.0			JENDL-2
2.257	1	0.5	0.073	1.0			JENDL-1
2.257	1	1.5	0.034	0.6			ENDF-B-4
2.257 \pm 0.009					0.065 \pm 0.007		BNL325(3)
2.257	0	0.5	0.073	1.0			73MOXON
2.257	1	0.5	0.071	(1.0)			74MOXON
2.26					0.065 \pm 0.007		69HOCKENBURY
2.257 \pm 0.009	1				0.068 \pm 0.011		70STIEGLITZ
5.53	1	0.5	0.0593	1.0			JENDL-2
5.53	1	0.5	0.0593	1.0			JENDL-1
5.53	1	1.5	0.028	0.6			ENDF-B-4
5.53 \pm 0.02					0.056 \pm 0.009		BNL325(3)
5.53	0	0.5	0.059	1.0			73MOXON
5.53	1	0.5	0.059	(1.0)			74MOXON
5.52					0.055 \pm 0.006		69HOCKENBURY
5.53 \pm 0.02	1				0.055 \pm 0.009		70STIEGLITZ
12.23	1	0.5	0.28	(1.0)	0.22 \pm 0.05		JENDL-2
12.23	1	0.5	0.0706	1.0			JENDL-1
12.23 \pm 0.03					0.11 \pm 0.02		BNL325(3)
12.20	0	0.5	0.044	1.0			73MOXON
12.22	1	0.5	0.046	(1.0)			74MOXON
12.2					0.17 \pm 0.02		69HOCKENBURY
12.2 \pm 0.04	1				0.042 \pm 0.007		70STIEGLITZ
12.23 \pm 0.03					0.22 \pm 0.05		72BEER
12.23 \pm 0.03	1						77FROEHN
12.46	0	0.5	2353.5	2.73			JENDL-2
12.5	0	0.5	2660.0	3.3			JENDL-1
12.43	0	0.5	2500.0	2.14			ENDF-B-4
12.5 \pm 0.1	0		2660 \pm 100	3.3 \pm 0.3			BNL325(3)
12.47	0		2110	3.3			73MOXON
12.46	0	0.5	2112	3.33			74MOXON
14.5	0						66FARRELL
12.47 \pm 0.06	0		2660 \pm 100	3.30 \pm 0.30			70STIEGLITZ
12.4 \pm 0.1	0		1910 \pm 50				71QRG
12.5 \pm 0.1	0		2650 \pm 100	3.4 \pm 0.4			72BEER
12.3 \pm 0.1	0		2660 \pm 100	2.65 \pm 0.28			75FROEHN
12.3 \pm 0.2	0	0.5	2660 \pm 100	b 2.73 \pm 0.50			77FROEHN
12.2244 \pm 0.046	0		2353.5 \pm 0.6				77SYME
13.62	1	0.5	0.52	(1.0)	0.34 \pm 0.05		JENDL-2
13.62	1	0.5	0.13	1.0			JENDL-1
13.62 \pm 0.03					0.11 \pm 0.03		BNL325(3)
13.60	0	0.5	0.099	1.0			73MOXON
13.616	1	0.5	0.13	(1.0)			74MOXON
13.8					0.090 \pm 0.013		69HOCKENBURY
13.6 \pm 0.05	1						70STIEGLITZ
13.62 \pm 0.03					0.34 \pm 0.05		72BEER
13.62 \pm 0.03	1						77FROEHN
17.20	1	0.5	0.064	(1.0)	0.06 \pm 0.02		JENDL-2
17.20 \pm 0.05	1				0.06 \pm 0.02		77FROEHN
23.89	1	1.5	0.56	(1.0)	0.72 \pm 0.12		JENDL-2
23.8	1	1.5	0.613	1.0			JENDL-1
23.8	1	1.5	0.7	1.2			ENDF-B-4
23.88 \pm 0.06					0.78 \pm 0.10		BNL325(3)
23.8	0	1.5	0.85	1.0			73MOXON
23.86	1	1.5	0.58	(1.0)			74MOXON
23.8					0.78 \pm 0.10		69HOCKENBURY
23.8 \pm 0.10	1				0.921 \pm 0.140		70STIEGLITZ
23.88 \pm 0.06					0.72 \pm 0.12		72BEER
23.89 \pm 0.06	1						77FROEHN
28.47	1	0.5	0.11	(1.0)	0.10 \pm 0.03		JENDL-2
28.47	1	0.5	0.171	1.2			JENDL-1
28.47 \pm 0.07					0.15 \pm 0.05		BNL325(3)
28.47 \pm 0.07	1	0.5	0.087	(1.0)	0.08 \pm 0.04		74MOXON
28.5					0.26 \pm 0.05		69HOCKENBURY
28.47 \pm 0.07	1				0.10 \pm 0.03		72BEER
28.47 \pm 0.07	1						77FROEHN
28.650	0	0.5	681.6	0.60			JENDL-2
28.6	0	0.5	850.0	1.1			JENDL-1
28.7	0	0.5	650.0	2.14			ENDF-B-4
28.60 \pm 0.10	0	0.5	850 \pm 100	1.1 \pm 0.1			BNL325(3)
28.642	0		752	1.1			73MOXON
28.64	0	0.5	750	1.11			74MOXON
30.0	0						66FARRELL
							GT = 1100
							GNO = 6.380

ENERGY (EV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
28.64 ± 0.10	0		800 ± 50	1.10 ± 0.10			
28.65 ± 0.05	0		690 ± 40				
28.60 ± 0.1	0		900 ± 200	0.8 ± 0.3			
28.6 ± 0.1	0		800 ± 50	0.6 ± 0.15			
28.64 ± 0.10	0	0.5	800	0.60 ± 0.15			
28.650 ± 0.001	0		681.6 ± 0.23				
29.46	1	0.5	0.042	(1.0)	0.04 ± 0.01		JENDL-2
29.47	1	0.5	0.0989	1.0		GT = 1.0989	JENDL-1
29.47 ± 0.08	1	0.5	0.099	(1.0)	0.09 ± 0.02		BNL325(3)
29.47 ± 0.08	1	0.5	0.099	(1.0)	0.09 ± 0.03		74MOXON
29.47 ± 0.08	1	0.5			0.04 ± 0.01		72BEER
29.46 ± 0.08	1						77FROEHNERR
30.25	1	0.5	0.52	(1.0)	0.34 ± 0.05		JENDL-2
30.24	1	0.5	0.471	1.0		GT = 1.471	JENDL-1
30.1	1	1.5	0.22	0.6		GT = 0.82	ENDF-B-4
30.24 ± 0.08	1	0.5	0.475	1.0			BNL325(3)
30.1	0	0.5	0.475	(1.0)			73MOXON
30.20	1	0.5	0.5	(1.0)			74MOXON
30.2					0.38 ± 0.06	GGS = 33 ± 5	69HOCKENBURY
30.1 ± 0.12	1				0.321 ± 0.050		70STIEGLITZ
30.24 ± 0.08	1	1				WWC = 0.31 ± 0.06	72BEER
30.25 ± 0.08	1				0.34 ± 0.05		77FROEHNERR
33.03	1	0.5	8.5	0.42	0.40 ± 0.07		JENDL-2
32.9	1	1.5	0.205	1.0		GT = 1.205	JENDL-1
32.9	1	1.5	0.24	0.6		GT = 0.84	ENDF-B-4
33.03 ± 0.08	1	1.5	0.24	(1.0)			BNL325(3)
32.9	0	0.5	0.540	1.0			73MOXON
33.01	1	1.5	0.21	(1.0)			74MOXON
32.9 ± 0.13	1				0.351 ± 0.055	70STIEGLITZ	
33.03 ± 0.08	1	1			0.40 ± 0.07		72BEER
33.04 ± 0.08	1					WWC = 0.33 ± 0.07	77FROEHNERR
33.03	1		12.8 ± 3.1				77SYME2
33.03	1		7.1 ± 1.9				77SYME3
33.55	1	0.5	3.1	0.25	0.23 ± 0.04		JENDL-2
33.3	1	0.5	0.25	1.0		GT = 1.25	JENDL-1
33.3	1	0.5	0.3	0.6		GT = 0.9	ENDF-B-4
33.3 ± 0.1	1	1	0.3	(1.0)			BNL325(3)
33.3	0	0.5	0.235	1.0			73MOXON
33.37	1	0.5	0.24	(1.0)			74MOXON
33.4					0.190 ± 0.031	GGS = 69HOCKENBURY	
33.3 ± 0.13	1						70STIEGLITZ
33.40 ± 0.08	1	1			0.23 ± 0.04		72BEER
33.42 ± 0.08	1					WWC = 0.20 ± 0.05	77FROEHNERR
33.55	1		1.7 ± 2.3				77SYME2
33.55	1		3.6 ± 1.3				77SYME3
39.52	1	0.5	0.75	(1.0)	0.43 ± 0.07		JENDL-2
39.4	1	1.5	0.325	1.0		GT = 1.325	JENDL-1
39.4	1	1.5	0.27	1.0		GT = 1.27	ENDF-B-4
39.5 ± 0.1	0	0.5	1.30	1.0			BNL325(3)
39.4					0.49 ± 0.08	73MOXON	
39.5					0.565 ± 0.100	69HOCKENBURY	
39.4 ± 0.15	1						70STIEGLITZ
39.54 ± 0.10	0	1			0.43 ± 0.07		72BEER
39.52 ± 0.10	1					WWC = 0.41 ± 0.08	77FROEHNERR
43.050	0	0.5	84.09	0.96			JENDL-2
43.0	0	0.5	90.0	1.3		GT = 91.3	JENDL-1
43.08	0	0.5	77.0	2.14		GT = 79.14	ENDF-B-4
43.0 ± 0.1	0	0.5	90	*30	WCH = 0.43 ± 0.15	BNL325(3)	
43.08	0		77	1.73			73MOXON
42.9					0.77 ± 0.12	GGS = 47 ± 6	69HOCKENBURY
43.08 ± 0.23	0		77	*15	1.73 ± 0.18		70STIEGLITZ
43.1 ± 0.1	0		140	*30		GT = 0.67 ± 0.15	71GARG
42.93 ± 0.11	0		120	*30	1.0 ± 0.2		72BEER
42.9 ± 0.1	0		120	*30	0.92 ± 0.18		77FROEHNERR
42.92 ± 0.11	0	0.5	120		0.98 ± 0.16		77SYME
43.050 ± 0.0036	0		84.09	*13			
47.60	1	1.5	1.04	(1.0)	1.02 ± 0.16		JENDL-2
47.6	1	0.5	9.23	0.9		GT = 10.13	JENDL-1
47.4	1	1.5	0.7	1.2		GT = 1.9	ENDF-B-4
47.6 ± 0.1	1	0.5	10	0.9 ± 0.2		WGI = 0.49	BNL325(3)
47.4	0	1.5	0.76	1.0			73MOXON
47.55	0	0.5	1.41	(2.0)			74MOXON
47.4 ± 0.22	1		10		0.862 ± 0.130	70STIEGLITZ	
47.60 ± 0.12	0	1	10		1.0 ± 0.4		72BEER
47.50 ± 0.12	1		10		1.02 ± 0.16	WWC = 0.78 ± 0.16	77FROEHNERR
49.83	1	0.5	0.43	(1.0)	0.30 ± 0.05		JENDL-2
49.8	1	0.5	0.361	1.0		GT = 1.351	JENDL-1
49.6	1	0.5	0.45	0.6		GT = 1.05	ENDF-B-4
49.8 ± 0.1	1	1			0.26 ± 0.04		BNL325(3)
49.6	0	0.5	0.345	1.0			73MOXON
49.6 ± 0.25	1				0.257 ± 0.043	70STIEGLITZ	
49.80 ± 0.12	1	1			0.30 ± 0.05		72BEER
49.83 ± 0.12	1					WWC = 0.27 ± 0.05	77FROEHNERR
50.88	1	0.5	0.16	(1.0)	0.14 ± 0.03		JENDL-2
50.99	1	0.5	0.124	1.0		GT = 1.124	JENDL-1
50.9 ± 0.2	1				0.11 ± 0.02		BNL325(3)
50.8	0	0.5	0.011	1.0			73MOXON

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WWS (EV)	MISCELLANEOUS	REFERENCE
50.8 ± 0.26	1	1					70STIEGLITZ
50.99 ± 0.15	1	1			0.14 ± 0.03	WWS = 0.11 ± 0.02	72BEER
50.88 ± 0.15	1	1					77FROEHNERR
51.57	1	0.5	0.92	{ 1.0 }	0.48 ± 0.05		JENDL-2
51.64	1	1.5	0.266	1.0		GT = 1.266	JENDL-1
51.5	1	1.5	0.36	0.6		GT = 0.96	ENDF-B-4
51.5 ± 0.2	1	1.5					BNL325(3)
51.5	0	0.5	0.84	1.0			73MOXON
51.9							69HOCKENBURY
51.5 ± 0.26	1	1			0.456 ± 0.078		70STIEGLITZ
51.64 ± 0.15	1	1			0.48 ± 0.05	WWS = 0.38 ± 0.08	72BEER
51.57 ± 0.15	1	1					77FROEHNERR
52.7	0	0.5	{ 0.1 }	1.0			73MOXON
52.7 ± 0.27	1	1					70STIEGLITZ
56.29	1	0.5	2.6	0.35	0.31 ± 0.04		JENDL-2
56.0	1	0.5	0.266	1.0		GT = 1.266	JENDL-1
56.3	1	0.5	1.06	0.6		GT = 1.66	ENDF-B-4
56.0 ± 0.2	1	1			0.20 ± 0.06		BNL325(3)
55.3	0	0.5	0.60	1.0			73MOXON
56.3 ± 0.28	1	1			0.374 ± 0.063		70STIEGLITZ
56.00 ± 0.15	1	1			0.31 ± 0.04	WWS = 0.15 ± 0.03	72BEER
56.12 ± 0.15	1	1					77FROEHNERR
56.29	1	1	2.2 ± 2.8				77SYME2
56.29	1	1	2.8 ± 1.3				77SYME3
56.94	1	0.5	0.91	0.57	0.35 ± 0.05		JENDL-2
56.74	1	1.5	0.282	1.0		GT = 1.282	JENDL-1
56.9	1	1.5	0.32	0.6		GT = 0.92	ENDF-B-4
56.7 ± 0.2	1	1			0.44 ± 0.09		BNL325(3)
56.9	0	0.5	0.71	1.0			73MOXON
57.0							69HOCKENBURY
56.9 ± 0.29	1	1			0.416 ± 0.070		70STIEGLITZ
56.74 ± 0.15	1	1			0.35 ± 0.05	WWS = 0.45 ± 0.09	72BEER
56.78 ± 0.15	1	1					77FROEHNERR
56.94	1	1	0.94 ± 0.94				77SYME2
56.94	1	1	0.9 ± 0.39				77SYME3
65.1101	0	0.5	459.9	1.90			JENDL-2
65.42	0	0.5	500.0	2.1		GT = 502.1	JENDL-1
65.3	0	0.5	390.0	2.43		GT = 392.43	ENDF-B-4
65.42 ± 0.16	0	0.5	500 ± 150	2.1 ± 0.3		WWS = 1.96 ± 0.59	BNL325(3)
65.13	0	0	440	2.33			73MOXON
65.35	0	0.5	440	2.39			74MOXON
62.0	0	0				GT = 700	66FARRELL
65.2						CNO = 2.838	69HOCKENBURY
65.13 ± 0.40	0	0	390 ± 30	2.43 ± 0.25			70STIEGLITZ
65.3 ± 0.2	0	0	810 ± 140			CNO = 3.17 ± 0.78	71GARG
65.42 ± 0.16	0	0	500 ± 150	2.0 ± 0.4			72BEER
65.4 ± 0.2	0	0	500 ± 150	1.79 ± 0.26			75FROEHNERR
65.12 ± 0.16	0	0.5	600	1.90 ± 0.30			77FROEHNERR
65.1101 ± 0.0235	0	0	459.9 ± 0.75				77SYME
71.39	1	0.5	0.56	{ 1.0 }	0.36 ± 0.06		JENDL-2
71.51	1	1.5	0.22	1.0		GT = 1.22	JENDL-1
71.3	1	1.5	0.29	0.6		GT = 0.69	ENDF-B-4
71.5 ± 0.2	1	1			0.36 ± 0.07		BNL325(3)
71.3	0	0.5	0.66	1.0			73MOXON
71.3 ± 0.45	1	1			0.396 ± 0.066		70STIEGLITZ
71.51 ± 0.18	1	1			0.36 ± 0.06	WWS = 0.33 ± 0.07	72BEER
71.39 ± 0.18	1	1					77FROEHNERR
73.16	1	0.5	1.0	{ 1.0 }	0.50 ± 0.08		JENDL-2
73.25	1	1.5	0.351	1.0		GT = 1.351	JENDL-1
73.3 ± 0.2	1	1			0.48 ± 0.09		BNL325(3)
73.2	0	0.5	1.56	1.0			73MOXON
72.8							69HOCKENBURY
73.2 ± 0.50	1	1			0.610 ± 0.100		70STIEGLITZ
73.25 ± 0.18	1	1			0.50 ± 0.08	WWS = 0.44 ± 0.09	72BEER
73.16 ± 0.18	1	1					77FROEHNERR
78.08	1	0.5	0.28	{ 1.0 }	0.22 ± 0.04		JENDL-2
78.26	1	0.5	0.333	1.0		GT = 1.333	JENDL-1
78.3 ± 0.2	1	1			0.23 ± 0.04		BNL325(3)
78.2	0	0.5	0.875	1.0			73MOXON
78.2 ± 0.55	1	1			0.308 ± 0.051		70STIEGLITZ
78.26 ± 0.20	1	1			0.22 ± 0.04	WWS = 0.19 ± 0.04	72BEER
78.08 ± 0.20	1	1					77FROEHNERR
79.74	1	0.5	0.59	{ 1.0 }	0.37 ± 0.06		JENDL-2
79.98	1	1.5	0.19	1.0		GT = 1.19	JENDL-1
80.0 ± 0.2	1	1			0.33 ± 0.07		BNL325(3)
79.9	0	0.5	1.75	1.0			73MOXON
79.9 ± 0.58	1	1			0.447 ± 0.073		70STIEGLITZ
79.98 ± 0.20	1	1			0.37 ± 0.06	WWS = 0.33 ± 0.07	72BEER
79.74 ± 0.20	1	1					77FROEHNERR
81.61	1	0.5	0.33	{ 1.0 }	0.25 ± 0.04		JENDL-2
81.95	1	0.5	0.282	1.0		GT = 1.282	JENDL-1
82.0 ± 0.2	1	1	110 ± 40		0.22 ± 0.05		BNL325(3)
82.8 ± 0.3	0	0	110 ± 40			CNO = 0.39 ± 0.14	71GARG
81.95 ± 0.20	1	1				WWS = 0.22 ± 0.05	72BEER
81.61 ± 0.20	1	1					77FROEHNERR

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
83.41	1	0.5	7.15	0.51	0.48 ± 0.09		JENDL-2
84.94	1	1.5	80.0	0.2		GT = 80.2	JENDL-1
84.9 ± 0.2	1	1.5	80	1.40	0.20 ± 0.04	WCH = 1.61	BNL32S(3)
84.7 ± 0.59	1						7CSTIEGLITZ
83.8 ± 0.3	0		80	±40		GNO = 0.29 ± 0.14	71GARG
84.94 ± 0.20	1					WNC = 0.41 ± 0.08	72BEER
85.02 ± 0.20	1				0.48 ± 0.09		77FROEHN
83.41	1		7.15 ± 7.15				77SYME2
83.41	1		7.15 ± 1.9				77SYME3
86.6671	0	0.5	341.9	1.50			JENDL-2
86.3	0	0.5	330.0	2.0		GT = 332.0	JENDL-1
87.0	0	0.5	310.0	2.14		GT = 312.14	ENDF-B-4
86.3 ± 0.2	0	0.5	330	±25		WCH = 1.12 ± 0.09	BNL32S(3)
86.80	0		320				73MOXON
86.7	0	0.5	286				74MOXON
84.5	0					GT = 500	66FARRELL
87.0						GNO = 1.742	
86.8 ± 0.60	0		330	±25			69HOCKENBURY
86.7 ± 0.3	0		160	±40			70STIEGLITZ
86.33 ± 0.22	0		330	±25	1.4 ± 0.3		71GARG
86.3 ± 0.2	0		330	±25	1.51 ± 0.30		72BEER
86.35 ± 0.22	0	0.5	330		1.50 ± 0.30		75FROEHN
86.6671 ± 0.0067	0		341.9	± 2.3			77SYME
87.9	1	0.5	8.0	0.8	0.73 ± 0.07		JENDL-2
87.89	1	1.5	0.47	1.0		GT = 1.47	JENDL-1
87.9 ± 0.2	1				0.64 ± 0.13		BNL32S(3)
87.6 ± 0.61	1						70STIEGLITZ
87.89 ± 0.22	1				0.73 ± 0.07		72BEER
87.80 ± 0.22	1					WNC = 0.64 ± 0.13	77FROEHN
87.9	1		6.3 ± 4.6				77SYME2
87.9	1		9.0 ± 1.9				77SYME3
89.865	1	0.5	16.0	0.21	0.21		JENDL-2
89.93	1	0.5	0.205	1.0		GT = 1.205	JENDL-1
89.9 ± 0.3	1				0.17 ± 0.04		BNL32S(3)
89.93 ± 0.25	1				0.21 ± 0.05		72BEER
89.44 ± 0.25	1					WNC = 0.17 ± 0.04	75FROEHN
89.865	1		7.5 ± 5.2				77SYME2
89.865	1		19.0 ± 2.9				77SYME3
91.69	1	0.5	6.0	0.35	0.33		JENDL-2
91.6	1	0.5	0.333	1.0		GT = 1.333	JENDL-1
91.6 ± 0.3	1				0.25 ± 0.05		BNL32S(3)
91.60 ± 0.25	1				0.33 ± 0.05		72BEER
91.40 ± 0.25	1					WNC = 0.25 ± 0.05	77FROEHN
91.69	1		5.2 ± 4.8				77SYME2
91.69	1		6.6 ± 2.1				77SYME3
92.13	1	0.5	7.1	0.62	0.56		JENDL-2
93.94	1	1.5	0.316	1.0		GT = 1.316	JENDL-1
93.9 ± 0.3	1				0.48 ± 0.10		BNL32S(3)
93.3 ± 0.65	1						70STIEGLITZ
93.94 ± 0.25	1				0.56 ± 0.08		72BEER
93.39 ± 0.25	1					WNC = 0.48 ± 0.10	75FROEHN
92.13	1		8.7 ± 3.0				77SYME2
92.13	1		6.9 ± 1.1				77SYME3
96.5 ± 0.68	1						70STIEGLITZ
97.851	0	0.5	835.6	1.20			JENDL-2
97.5	0	0.5	1000.0	1.0		GT = 1001.0	JENDL-1
98.6	0	0.5	690.0	2.14		GT = 692.14	ENDF-B-4
97.5 ± 0.3	0	0.5	1000	±200	1.0 ± 0.2	WCH = 3.20 ± 0.64	BNL32S(3)
98.10	0		940		2.0		73MOXON
97.81	0	0.5	940		1.0		74MOXON
96.5	0					GT = 1250	66FARRELL
97.2						GNO = 4.084	
98.1 ± 0.70	0		870	±70			69HOCKENBURY
97.7 ± 0.4	0		1070	±160			70STIEGLITZ
97.20 ± 0.25	0		1000	±200	1.0 ± 0.2		71GARG
97.2 ± 0.3	0		1000	±200	1.13 ± 0.20		72BEER
95.79 ± 0.30	0	0.5	870		1.20 ± 0.25		75FROEHN
97.851 ± 0.011	0		835.6	± 0.23			77SYME
99.44	1	0.5	8.2	0.87	0.79		JENDL-2
99.24	1	1.5	0.852	1.0		GT = 1.852	JENDL-1
99.2 ± 0.3	1				0.92 ± 0.20		BNL32S(3)
99.24 ± 0.25	1						72BEER
99.94 ± 0.30	1				0.79 ± 0.09		77FROEHN
99.44	1		7.5 ± 4.0				77SYME2
99.44	1		8.5 ± 1.8				77SYME3
101.18	1	0.5	0.18	(1.0)	0.15 ± 0.04		JENDL-2
101.9	1	0.5	0.111	1.0		GT = 1.111	JENDL-1
101.9 ± 0.3	1				0.10 ± 0.05		BNL32S(3)
101.9 ± 0.25	1						72BEER
101.18 ± 0.30	1				0.15 ± 0.04		77FROEHN
107.479	0	0.5	265.3	1.35			JENDL-2
108.3	0	0.5	700.0	1.1		GT = 701.1	JENDL-1
106.8	0	0.5	840.0	2.14		GT = 842.14	ENDF-B-4
108.3 ± 0.3	0	0.5	700	±100	1.1 ± 0.3	WCH = 2.13 ± 0.31	BNL32S(3)

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
107.8	0		660	2.0			73MOXON
108.3	0	0.5	695	1.1			74MOXON
106.0	0					GT = 840	66FARRELL
107.8 ± 0.75	0		610 ± 60			CNO = 2.622	70STIEGLITZ
109.5 ± 0.5	0		1750			CNO = 5.29	71GARG
108.0 ± 0.25	0		700 ± 100	1.1 ± 0.3			72BEER
108.0 ± 0.3	0		700 ± 100	1.35 ± 0.20			75FROEHNERR
107.77 ± 0.30	0	0.5	610	1.35 ± 0.25			77FROEHNERR
107.479 ± 0.004	0		265.3 ± 0.53				77SYME
111.5	1	1.5	8.2	1.34	2.30 ± 0.38		JENDL-2
111.6	1	0.5	27.0	3.0			JENDL-1
111.6 ± 0.3	1	0.5		2.7 ± 0.6			BNL325(3)
111.3 ± 1.0	1				3.74 ± 0.80		70STIEGLITZ
111.6 ± 0.25	1				2.30 ± 0.38		72BEER
111.46 ± 0.30	1						77FROEHNERR
111.5	1		4.0 ± 8.0				77SYME2
111.5	1		9.25 ± 3.9				77SYME3
121.4	1	1.5	18.8	0.97	1.85 ± 0.15		JENDL-2
120.6 ± 1.1	1	1.5		1.3 ± 0.3			BNL325(3)
120.6 ± 1.1	1				2.31 ± 0.50		70STIEGLITZ
120.2 ± 0.35	1				1.85 ± 0.15		77FROEHNERR
121.4	1		27.4 ± 6.0				77SYME2
121.4	1		17.4 ± 2.5				77SYME3
123.2	1	0.5	1.94	[1.0]	0.66 ± 0.12		JENDL-2
123.6	1	1.5	0.538	1.0			JENDL-1
123.8 ± 1.2	1						BNL325(3)
123.8 ± 1.2	1						70STIEGLITZ
123.2 ± 0.4	1				0.66 ± 0.12		77FROEHNERR
127.7	1	0.5	61.0	[1.0]			JENDL-2
126.5	1	0.5	40.0	0.6			ENDF-B-4
127.74	0	0.5	40				73MOXON
126.5	0		43 ± 3				66FARRELL
127.7	1		61.1 ± 5.75				77SYME2
127.7	1		61.0 ± 2.5				77SYME3
129.0	1	0.5	32.0	[1.0]	0.97 ± 0.16		JENDL-2
129.2	1	1.5	1.0	1.0			JENDL-1
129.7 ± 1.3	1						BNL325(3)
129.7 ± 1.3	1						70STIEGLITZ
129.0 ± 0.4	1				0.97 ± 0.16		77FROEHNERR
135.7	1	1.5	16.7	[1.8]	3.25 ± 0.54		JENDL-2
136.5	1	1.5	3.7	3.7			JENDL-1
136.5 ± 1.4	1	0.5		4.3 ± 0.9			BNL325(3)
136.5 ± 1.4	1				4.31 ± 0.90		70STIEGLITZ
135.7 ± 0.5	1				3.25 ± 0.54		77FROEHNERR
141.9	1	1.5	41.0	1.6	3.0 ± 0.5		JENDL-2
139.6	1	1.5	3.5	3.5			JENDL-1
138.5	1	0.5	70.0	0.6			ENDF-B-4
139.6 ± 1.4	1	0.5		4.0 ± 0.9			BNL325(3)
139.74	0	0.5	70				73MOXON
138.5	0		77 ± 7				66FARRELL
139.6 ± 1.4	1				3.95 ± 0.90		70STIEGLITZ
138.95 ± 0.6	1				3.0 ± 0.5		77FROEHNERR
141.9	1		40.5 ± 6.3				77SYME2
141.9	1		41.5 ± 2.6				77SYME3
154.4	1	0.5	140.0	1.09	1.09 ± 0.18		JENDL-2
148.4	1	1.5	1.5	1.0			JENDL-1
153.5	1	0.5	200.0	0.6			ENDF-B-4
148.1 ± 0.4	1				1.09 ± 0.18		77FROEHNERR
154.4	1		165.0 ± 20.0				77SYME2
154.4	1		136.0 ± 8.0				77SYME3
156.0	0	0.5	1440	1	0.70		JENDL-2
156.4	0	0.5	440.0		2.0		JENDL-1
156.0	1	0.5	380.0		0.6		ENDF-B-4
156.4 ± 1.2	0		440 ± 50				BNL325(3)
157.24	0	0.5	380				73MOXON
155.4	0	0.5	440	[2.0]			74MOXON
156.0	0		419 ± 39				66FARRELL
156.4 ± 1.2	0	0.5	440 ± 50				70STIEGLITZ
155.4 ± 0.5	0	0.5	440 ± 50	0.85 ± 0.17			75FROEHNERR
154.8 ± 0.7	0	0.5	440	0.70 ± 0.12			77FROEHNERR
156	0		1100				77SYME
161.407	0	0.5	1009.5	2.2			JENDL-2
162.0	0	0.5	1400.0	2.2			JENDL-1
160.8	0	0.5	1800.0	2.14			ENDF-B-4
162.0 ± 0.4	0	0.5	1400 ± 200	2.2 ± 0.5			BNL325(3)
162.1	0		1330	2.0			73MOXON
161.62	0	0.5	1340	2.2			74MOXON
160.0	0						66FARRELL
162.1 ± 1.3	0		1250 ± 130				70STIEGLITZ
161	0		5300 ± 2000				71GARG
161.7 ± 0.4	0		1400 ± 200	2.2 ± 0.5			72BEER
161.7 ± 0.5	0		1400 ± 200	1.9 ± 0.4			75FROEHNERR
160.9 ± 0.9	0	0.5	1250	2.2 ± 0.4			77FROEHNERR
161.407 ± 0.017	0		1009.5 ± 1.0				77SYME

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WWS (EV)	MISCELLANEOUS	REFERENCE
167.05	I	1.5	65.0	1.54			JENDL-2
167.0	I	1.5	3.0	3.0		GT = 6.0	JENDL-1
167.0 ± 1.0					3.0 ± 0.8		77FROEHNERR
167.05	I		75.5 ± 11.0				77SYME2
167.05	I		60.5 ± 6.0				77SYME3
173.7	I	1.5	20.0	1.0	1.9 ± 0.5		JENDL-2
173.7	I	1.5	19.0	1.0		GT = 20.0	JENDL-1
173.7 ± 1.3					1.9 ± 0.5		77FROEHNERR
185.409	O	0.5	9150.4	3.2			JENDL-2
186.5	O	0.5	5850.0	2.0		GT = 5852.0	JENDL-1
186.2	O	0.5	6000.0	2.14		GT = 6002.1	ENDF-B-4
186.6 ± 1.5	O	0.5	5800 ± 800			GCH = 13.43 ± 1.85	BNL325(3)
186.5	O		5850	2.0			73MOXON
186.3	O	0.5	5870	(2.0)			74MOXON
186.2	O					GT = 6000	66FARRELL
186.5 ± 1.5	O		6000 ± 800			GNO = 14.303	70STIEGLITZ
186.5 ± 1	O		5700 ± 2300			GNO = 13.22 ± 5.34	71GARG
184.0 ± 1.5	O	0.5	5800	3.2 ± 0.8			77FROEHNERR
185.409 ± 0.108	O		9150.4 ± 2.2				77SYME
197.034	O	0.5	3692.3	4.1			JENDL-2
198.0	O	0.5	3100.0	2.0		GT = 3102.0	JENDL-1
197.0	O	0.5	3500.0	2.14		GT = 3502.1	ENDF-B-4
198.0 ± 1.8	O	0.5	3100 ± 350			GCH = 6.97 ± 0.79	BNL325(3)
198.0	O		3280	2.0			73MOXON
196.5	O	0.5	3290	(2.0)			74MOXON
197.0	O					GT = 3500	66FARRELL
198.0 ± 1.8	O		3100 ± 350			GNO = 8.125	70STIEGLITZ
196. ± 1	O		3500 ± 2300			GNO = 7.90 ± 5.20	71GARG
195.0 ± 2.0	O	0.5	3100	4.1 ± 1.0			77FROEHNERR
197.034 ± 0.046	O		3692.3 ± 1.7				77SYME
201.6	I	1.5	185.0	1.4	2.8		JENDL-2
202.0 ± 2.5	I		120		2.8 ± 0.7		77FROEHNERR
201.6	I		179.0 ± 19.7				77SYME2
201.6	I		186.0 ± 7.58				77SYME3
207.24	I	0.5	119.0	(1.0)			JENDL-2
207.24	I	0.5	110.0	1.0		GT = 111.0	JENDL-1
206.0	I	0.5	110.0	0.6		GT = 110.6	ENDF-B-4
206.0 ± 1.6	I		120				BNL325(3)
207.24	O	0.5	110				73MOXON
206.0	O		119 ± 9				66FARRELL
214.15	I	0.5	85.0	(1.0)			JENDL-2
215.24	I	0.5	94.0	0.6		GT = 94.6	JENDL-1
214.0	I	0.5	94.0	0.6		GT = 94.6	ENDF-B-4
214.0 ± 1.8	I		74				BNL325(3)
215.24	O	0.5	94				73MOXON
214.0	O		102 ± 8				66FARRELL
214.15	I		98.0 ± 17.0				77SYME2
214.15	I		83.7 ± 6.5				77SYME3
220.1	I	0.5	70.0	(1.0)			JENDL-2
221.24	I	0.5	98.0	0.6		GT = 98.6	JENDL-1
220.0	I	0.5	98.0	0.6		GT = 98.6	ENDF-B-4
220.0 ± 1.8	I		105				BNL325(3)
221.24	O	0.5	98				73MOXON
220.0	O		106 ± 8				66FARRELL
220.1	I		38.0 ± 26.0				77SYME2
220.1	I		83.0 ± 11.0				77SYME3
221.8	I	0.5	54.0	(1.0)			JENDL-2
221.8	I		57.0 ± 28.0				77SYME2
221.8	I		53.0 ± 10.5				77SYME3
230.2	I	0.5	46.0	(1.0)			JENDL-2
230.24	I	0.5	206.0	0.6		GT = 208.6	JENDL-1
229.0	I	0.5	208.0	0.6		GT = 208.6	ENDF-B-4
229.0 ± 1.8	I		224				BNL325(3)
230.24	O	0.5	208				73MOXON
229.0	O		224 ± 16				66FARRELL
230.2	I		60.0 ± 18.0				77SYME2
230.2	I		44.0 ± 7.0				77SYME3
231.05	I	0.5	82.5	(1.0)			JENDL-2
231.05	I		84.0 ± 20.0				77SYME2
231.05	I		82.5 ± 8.0				77SYME3
252.2	I	0.5	420.0	(1.0)			JENDL-2
253.24	I	0.5	870.0	0.6		GT = 870.6	JENDL-1
252.0	I	0.5	540.0	0.6		GT = 540.6	ENDF-B-4
252.0 ± 2.0	I		910				BNL325(3)
253.24	O	0.5	870				73MOXON
252.0	O		905 ± 35				66FARRELL
252.2	I		397.0 ± 40.0				77SYME2
252.2	I		425.0 ± 17.0				77SYME3
253.05	I	0.5	250.0	(1.0)			JENDL-2
253.05	I		260.0 ± 31.0				77SYME2
253.05	I		248.0 ± 12.0				77SYME3
256.3	I	0.5	470.0	(1.0)			JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
256.3	1		520.0 ±40.0				77SYME?
256.3	1		460.0 ±15.0				77SYME3
257.8	0	0.5	3500.0	2.0			JENDL-2
257.8	0	0.5	3500.0	2.0			JENDL-1
257.0	0	0.5	3750.0	2.14			ENDF-B-4
257.8 ± 2.1	0	0.5	3500 ±600			GT = 3502.0	BNL325(3)
257.8	0		3620	2.0		GT = 3752.1	73MOXON
257.8	0	0.5	3530	(2.0)		WGH = 6.89 ± 1.18	74MOXON
257.0	0					GT = 3750	66FARRELL
257.8 ± 2.1	0		3500 ±600			GNO = 7.690	70STIEGLITZ
273.0	1	0.5	150.0	0.6		GT = 150.6	ENDF-B-4
277.1	1	0.5	207.0	(1.0)			JENDL-2
274.0	1	0.5	150.0	0.6		GT = 150.6	ENDF-B-4
277.1	1		186.5 ±50.0				77SYME2
277.1	1		211.5 ±19.5				77SYME3
279.6	0	0.5	750.0	2.0			JENDL-2
279.6	0	0.5	750.0	2.0		GT = 752.0	JENDL-1
279.6	1	0.5	700.0	0.6		GT = 700.6	ENDF-B-4
279.6 ± 2.3	0	0.5	750 ±160			WGH = 1.42 ± 0.30	BNL325(3)
279.6	0		750	2.0			73MOXON
279.6 ± 2.3	0		750 ±160				70STIEGLITZ
283.0	1	0.5	620.0	1.0			JENDL-2
283.74	1	0.5	620.0	0.6		GT = 620.6	JENDL-1
282.5	1	0.5	620.0	0.6		GT = 620.6	ENDF-B-4
282.5 ± 2.4	0		647				BNL325(3)
283.74	0	0.5	620				73MOXON
282.5	0		647 ±27				66FARRELL
292.3	1	0.5	116.0	(1.0)		GT = 360.6	JENDL-2
293.74	1	0.5	360.0	0.6		GT = 360.6	JENDL-1
292.2	1	0.5	360.0	0.6			ENDF-B-4
292.5 ± 2.4	0		378				BNL325(3)
293.74	0	0.5	360				73MOXON
292.5	0		378 ±18				66FARRELL
292.3	1		124.0 ±25.0				77SYME2
292.3	1		114.0 ±10.0				77SYME3
292.8	1	0.5	170.0	(1.0)			JENDL-2
292.8	1		186.0 ±30.0				77SYME2
292.8	1		169.0 ±11.0				77SYME3
295.65	1	0.5	130.0	(1.0)			JENDL-2
295.5	1	0.5	600.0	0.6		GT = 600.6	ENDF-B-4
295.65	1		100.0 ±37.0				77SYME2
295.65	1		130.0 ±11.0				77SYME3
302.0	1	0.5	150.0	0.6		GT = 150.6	ENDF-B-4
303.7	1	0.5	90.0	0.6		GT = 90.6	ENDF-B-4
307.0	1	0.5	500.0	1.0			JENDL-2
307.24	1	0.5	500.0	0.6		GT = 500.6	JENDL-1
305.3	1	0.5	500.0	0.6		GT = 500.6	ENDF-B-4
305 ± 2.5	0		525				BNL325(3)
307.24	0	0.5	500				73MOXON
306.0	0		525 ±25				66FARRELL
316.8	0	0.5	3200.0	2.0		GT = 3202.0	JENDL-2
316.0	0	0.5	3200.0	2.0		GT = 3202.0	JENDL-1
316.0	0	0.5	3200.0	2.14		GT = 3202.1	ENDF-B-4
316.0 ± 2.5	0	0.5	3200 ±600			WGH = 5.69 ± 1.07	BNL325(3)
316.8	0		3200	2.0			73MOXON
316.8	0	0.5	3200	(2.0)			74MOXON
316.0	0					GT = 3200	66FARRELL
316.8 ± 3.1	0		3200 ±600			GNO = 5.969	70STIEGLITZ
326.3	0	0.5	5800.0	2.0		GT = 6802.0	JENDL-2
326.3	0		7000.0	2.0		GT = 7002.0	JENDL-1
325.0	0	0.5	8500.0	2.14		GT = 8502.1	ENDF-B-4
326.3 ± 2.5	0	0.5	7000 ±1100			WGH = 12.25 ± 1.93	BNL325(3)
326.3	0		7370	2.0			73MOXON
326.3	0	0.5	6900	(2.0)			74MOXON
325.0	0					GT = 8500	66FARRELL
326.3 ± 3.3	0		6800 ±1100			GNO = 15.655	70STIEGLITZ
339.5	0	0.5	7500.0	2.0		GT = 7502.0	JENDL-2
339.5	0	0.5	6500.0	2.0		GT = 6502.0	JENDL-1
338.3	0	0.5	4400.0	2.14		GT = 4402.1	ENDF-B-4
339.5 ± 2.5	0	0.5	6500 ±1500			WGH = 11.16 ± 2.57	BNL325(3)
339.5	0		6050	2.0			73MOXON
339.5	0	0.5	6950	(2.0)			74MOXON
338.0	0					GT = 5250	66FARRELL
339.5 ± 3.5	0		6800 ±1500			GNO = 9.500	70STIEGLITZ
347.24	0	0.5	250.0	2.0		GT = 252.0	JENDL-2
347.24	0	0.5	250.0	2.0		GT = 252.0	JENDL-1
346.0	0	0.5	250.0	2.14		GT = 252.14	ENDF-B-4

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WWS (EV)	MISCELLANEOUS	REFERENCE
347.24	0		250	2.0			73MOXON
347.24	0	0.5	250	[2.0]			74MOXON
346.0	0					GT = 250 CNO = 0.448	66FARRELL
357.2	0	0.5	1000.0	2.0		GT = 1002.0	JENDL-2
357.2	0	0.5	1000.0	2.0		GT = 1002.0	JENDL-1
357.2	0	0.5	1000.0	2.14		GT = 1002.1	ENDF-B-4
357.2 ± 2.6	0	0.5	^a 1000			WGH = 1.67	BNL325(3)
358.44	0		1000	2.0			73MOXON
358.44	0	0.5	1000	[2.0]			74MOXON
357.2	0					GT = 1000 CNO = 1.765	66FARRELL
359.4	1	0.5	1076.0	[1.0]		GT = 1077.0	JENDL-2
359.74	1	0.5	1076.0	0.6		GT = 1076.6	JENDL-1
358.5	1	0.5	1076.0	0.6		GT = 1076.6	ENDF-B-4
358.5 ± 2.6	■		^a 1113				BNL325(3)
359.74	0	0.5	1076				73MOXON
358.5	0		^a 1113 ± 37				66FARRELL
376.74	0	0.5	4000.0	2.0		GT = 4002.0	JENDL-2
375.5	0	0.5	4000.0	2.0		GT = 4002.0	JENDL-1
375.5	0	0.5	4000.0	2.14		GT = 4002.1	ENDF-B-4
375.5	0		^a 4000				BNL325(3)
376.74	0		4000	2.0			73MOXON
376.74	0	0.5	4000	[2.0]			74MOXON
375.5	0					GT = 4000 CNO = 6.905	66FARRELL
379.0	1	0.5	220.0	[1.0]		GT = 221.00	JENDL-2
379.74	1	0.5	220.0	0.6		GT = 220.6	JENDL-1
378.5	1	0.5	220.0	0.6		GT = 220.6	ENDF-B-4
378.5	■		^a 226				BNL325(3)
379.74	0	0.5	220				73MOXON
378.5	0		^a 226 ± 6				66FARRELL
387.74	1	0.5	280.0	[1.0]		GT = 281.0	JENDL-2
387.74	1	0.5	280.0	0.6		GT = 280.6	JENDL-1
387.5	1	0.5	280.0	0.6		GT = 280.6	ENDF-B-4
387.5	■		^a 290				BNL325(3)
387.74	0	0.5	280				73MOXON
387.5	0		^a 290 ± 10				66FARRELL
393.0	1	0.5	266.0	[1.0]		GT = 267.0	JENDL-2
393.24	1	0.5	266.0	0.6		GT = 266.6	JENDL-1
392.0	1	0.5	266.0	0.6		GT = 266.6	ENDF-B-4
392.0	■		^a 225				BNL325(3)
393.24	0	0.5	266				73MOXON
392.0	0		^a 275 ± 9				66FARRELL
397.4	1	0.5	312.0	[1.0]		GT = 313.0	JENDL-2
398.24	1	0.5	312.0	0.6		GT = 312.6	JENDL-1
397.0	1	0.5	312.0	0.6		GT = 312.6	ENDF-B-4
397.0	■		^a 321				BNL325(3)
398.24	0	0.5	312				73MOXON
397.0	0		^a 321 ± 9				66FARRELL
402.74	1	0.5	390.0	[1.0]		GT = 391.0	JENDL-2
402.74	1	0.5	390.0	0.6		GT = 390.6	JENDL-1
401.5	1	0.5	390.0	0.6		GT = 390.6	ENDF-B-4
401.5	■		^a 400				BNL325(3)
402.74	0	0.5	390				73MOXON
401.5	0		^a 400 ± 10				66FARRELL
406.4	1	0.5	200.0	0.6		GT = 200.6	ENDF-B-4
408.5	1	0.5	200.0	0.6		GT = 200.6	ENDF-B-4
412.3	0	0.5	750.0	2.0		GT = 752.0	JENDL-2
412.3	0	0.5	750.0	2.0		GT = 752.0	JENDL-1
412.3	0	0.5	750.0	2.14		GT = 752.14	ENDF-B-4
412.3	0		^a 750			WGH = 1.17	BNL325(3)
413.54	0		750	2.0			73MOXON
413.54	0	0.5	750	[2.0]			74MOXON
412.3	0					GT = 750 CNO = 1.242	66FARRELL
422.24	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-2
421.0	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-1
421.7	0	0.5	2000.0	2.14		GT = 2002.1	ENDF-B-4
421.0	0	0.5	^a 2000			WGH = 3.08	BNL325(3)
422.24	0		2000	2.0			73MOXON
422.24	0	0.5	2000	[2.0]			74MOXON
421.0	0					GT = 2000 CNO = 3.282	66FARRELL
426.5	0	0.5	500.0	2.0		GT = 502.0	JENDL-2
426.5	0	0.5	500.0	2.0		GT = 502.0	JENDL-1
427.0	0	0.5	500.0	2.14		GT = 502.14	ENDF-B-4
426.5	0	0.5	^a 500			WGH = 0.77	BNL325(3)
427.74	0		500	2.0			73MOXON
427.74	0	0.5	500	[2.0]			74MOXON
426.5	0					GT = 500 CNO = 0.816	66FARRELL
432.74	1	0.5	220.0	[1.0]		GT = 221.0	JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
432.74	1	0.5	220.0	0.6		GT = 220.6	JENDL-1
431.5	1	0.5	220.0	0.6		GT = 220.6	ENDF-B-4
431.5	■		230				BNL325(3)
432.74	1	0.5	220				73MOXON
431.5	1		230	±10			66FARRELL
434.0	1	0.5	80.0	0.6		GT = 80.6	ENDF-B-4
437.24	0	0.5	1000.0	2.0		GT = 1002.0	JENDL-2
437.24	0	0.5	1000.0	2.0		GT = 1002.0	JENDL-1
436.5	0	0.5	1000.0	2.14		GT = 1002.1	ENDF-B-4
437.24	0		1000	2.0			73MOXON
437.24	0	0.5	1000	(2.0)		GT = 1000	74MOXON
436.0	0					GNO = 1.616	66FARRELL
447.24	0	0.5	3000.0	2.0		GT = 3002.0	JENDL-2
447.24	0	0.5	3000.0	2.0		GT = 3002.0	JENDL-1
447.5	0	0.5	3000.0	2.14		GT = 3002.1	ENDF-B-4
446.0	0	0.5	3000			WCH = 4.49	BNL325(3)
447.24	0		3000	2.0			73MOXON
447.24	0	0.5	3000	(2.0)		GT = 3000	74MOXON
446.0	0					GNO = 4.800	66FARRELL
454.24	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-2
453.0	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-1
453.0	0	0.5	1500.0	2.14		GT = 1502.1	ENDF-B-4
453.0	0		1500			WCH = 2.23	BNL325(3)
454.24	0		1500	2.0			73MOXON
454.24	0	0.5	1500	(2.0)		GT = 1500	74MOXON
453.0	0					GNO = 2.384	66FARRELL
463.24	0	0.5	1000.0	2.0		GT = 1002.0	JENDL-2
462.0	0	0.5	1000.0	2.0		GT = 1002.0	JENDL-1
462.0	0	0.5	1000.0	2.14		GT = 1002.1	ENDF-B-4
462.0	0		1000			WCH = 1.47	BNL325(3)
463.24	0		1000	2.0			73MOXON
463.24	0	0.5	1000	(2.0)		GT = 1000	74MOXON
462.0	0					GNO = 1.576	66FARRELL
479.1	1	1.5	100.0	0.6		GT = 100.6	ENDF-B-4
474.24	0	0.5	500.0	2.0		GT = 502.0	JENDL-2
474.24	0	0.5	500.0	2.0		GT = 502.0	JENDL-1
474.7	0	0.5	300.0	2.14		GT = 302.14	ENDF-B-4
473.0	0	0.5	500			WCH = 0.73	BNL325(3)
474.24	0		500	2.0			73MOXON
474.24	0	0.5	500	(2.0)		GT = 500	74MOXON
473.0	0					GNO = 0.780	66FARRELL
485.84	0	0.5	3750.0	2.0		GT = 3752.0	JENDL-2
484.6	0	0.5	3750.0	2.0		GT = 3752.0	JENDL-1
484.5	0	0.5	6600.0	2.14		GT = 6602.1	ENDF-B-4
484.6	0	0.5	3750			WCH = 5.39	BNL325(3)
485.84	0		3750	2.0			73MOXON
485.84	0	0.5	3750	(2.0)		GT = 3750	74MOXON
484.6	0					GNO = 5.788	66FARRELL
498.74	1	0.5	565.0	(1.0)		GT = 566.0	JENDL-2
498.74	1	0.5	565.0	0.6		GT = 565.6	JENDL-1
497.8	1	0.5	565.0	0.6		GT = 565.6	ENDF-B-4
497.5	■		578				BNL325(3)
498.74	0	0.5	565				73MOXON
497.5	0		577.5	±12.5			66FARRELL
498.0	0	0.5	5000.0	2.0		GT = 5002.0	JENDL-2
498.0	0	0.5	5000.0	2.0		GT = 5002.0	JENDL-1
498.0	0	0.5	5000.0	2.14		GT = 5002.1	ENDF-B-4
498.0	0		5000			WCH = 7.63	BNL325(3)
499.24	0		5000	2.0			73MOXON
499.24	0	0.5	5000	(2.0)		GT = 5000	74MOXON
498.0	0					GNO = 7.628	66FARRELL
503.74	1	0.5	325.0	(1.0)		GT = 326.0	JENDL-2
503.74	1	0.5	325.0	0.6		GT = 325.6	JENDL-1
502.5	1	0.5	325.0	0.6		GT = 325.6	ENDF-B-4
502.5	■		333				BNL325(3)
503.74	0	0.5	325				73MOXON
502.5	0		332.5	± 7.5			66FARRELL
512.74	1	1.5	1270.0	(1.0)		GT = 1271.0	JENDL-2
512.74	1	1.5	2565.0	0.6		GT = 2565.6	JENDL-1
511.5	1	1.5	850.0	0.6		GT = 850.6	ENDF-B-4
511.5	1	1.5	2420				BNL325(3)
512.74	0	0.5	2565				73MOXON
511.5	0	1.5	1276	± 6			66FARRELL
516.0	0	0.5	2250.0	2.0		GT = 2252.0	JENDL-2
514.74	0	0.5	2250.0	2.0		GT = 2252.0	JENDL-1
514.5	0	0.5	1350.0	2.14		GT = 1352.1	ENDF-B-4

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
513.5	0	0.5	2250			WCH = 3.14 ± 6.3	BNL325(3) 73MOXON 74MOXON 66FARRELL
514.74	0		2250	2.0			
514.74	0	0.5	2250		{ 2.0 }		
513.5	0					GT = 2250 CNO = 3.388	
520.3	0	0.5	5000.0	2.0		GT = 5002.0	JENDL-2
521.54	0	0.5	5000.0	2.0		GT = 5002.0	JENDL-1
520.8	0	0.5	2950.0	2.14		GT = 2952.1	ENDF-B-4
520.3	0	0.5	5000			WCH = 6.93	BNL325(3) 73MOXON 74MOXON 66FARRELL
521.54	0		5000	2.0		GT = 5000	
521.54	0	0.5	5000		{ 2.0 }	CNO = 7.486	
520.3	0						
527.0	0	0.5	3000.0	2.0		GT = 3002.0	JENDL-2
526.24	0	0.5	3000.0	2.0		GT = 3002.0	JENDL-1
526.5	0	0.5	3000.0	2.14		GT = 3002.1	ENDF-B-4
525.5	0	0.5	3000			WCH = 4.14	BNL325(3) 73MOXON 74MOXON 66FARRELL
526.24	0		3000	2.0		GT = 3000	
526.74	0	0.5	3000		{ 2.0 }	CNO = 4.473	
525.5	0						
528.0	1	0.5	300.0	0.6		GT = 300.6	ENDF-B-4
534.24	0	0.5	500.0	2.0		GT = 502.0	JENDL-2
534.24	0	0.5	500.0	2.0		GT = 502.0	JENDL-1
533.0	0	0.5	500.0	2.14		GT = 502.14	ENDF-B-4
533.0	0	0.5	500			WCH = 0.69	BNL325(3) 73MOXON 74MOXON 66FARRELL
534.24	0		500	2.0		GT = 500	
534.24	0	0.5	500		{ 2.0 }	CNO = 0.741	
533.0	0						
553.74	1	0.5	700.0		{ 1.0 }	GT = 701.0	JENDL-2
553.74	1	0.5	700.0			GT = 700.6	JENDL-1
553.4	1	0.5	700.0			GT = 700.6	ENDF-B-4
552.5	■		710				BNL325(3) 73MOXON 74MOXON 66FARRELL
553.74	0	0.5	700				
552.5	0	0.5	710	±10			
557.74	0	0.5	500.0	2.0		GT = 502.0	JENDL-2
557.74	0	0.5	500.0	2.0		GT = 502.0	JENDL-1
557.4	0	0.5	800.0	2.14		GT = 802.14	ENDF-B-4
556.5	0	0.5	500			WCH = 0.67	BNL325(3) 73MOXON 74MOXON 66FARRELL
557.74	0		500	2.0		GT = 500	
557.74	0	0.5	500		{ 2.0 }	CNO = 0.728	
556.5	0						
567.24	1	0.5	260.0		1.0	GT = 261.0	JENDL-2
567.24	1	0.5	260.0		1.0	GT = 261.0	JENDL-1
566.5	1	0.5	260.0		0.6	GT = 260.6	ENDF-B-4
566.0	*		260				BNL325(3) 73MOXON 74MOXON 66FARRELL
557.24	0	0.5	260				
566.0	0	0.5	260				
581.74	0	0.5	250.0	2.0		GT = 252.0	JENDL-2
581.74	0	0.5	250.0	2.0		GT = 252.0	JENDL-1
581.3	0	0.5	500.0	2.14		GT = 502.14	ENDF-B-4
580.3	0	0.5	250			WCH = 0.33	BNL325(3) 73MOXON 74MOXON 66FARRELL
581.74	0		250	2.0		GT = 250	
581.74	0	0.5	250		{ 2.0 }	CNO = 0.357	
580.3	0						
589.74	0	0.5	500.0	2.0		GT = 502.0	JENDL-2
589.74	0	0.5	500.0	2.0		GT = 502.0	JENDL-1
590.0	0	0.5	500.0	2.14		GT = 502.14	ENDF-B-4
588.5	0	0.5	500			WCH = 0.65	BNL325(3) 73MOXON 74MOXON 66FARRELL
589.74	0		500	2.0		GT = 500	
589.74	0	0.5	500		{ 2.0 }	CNO = 0.711	
588.5	0						
594.8	0	0.5	2500.0	2.0		GT = 2502.0	JENDL-2
596.04	0	0.5	2500.0	2.0		GT = 2502.0	JENDL-1
594.8	0	0.5	2800.0	2.14		GT = 2802.1	ENDF-B-4
594.8	0	0.5	2500			WCH = 3.24	BNL325(3) 73MOXON 74MOXON 66FARRELL
596.04	0		2500	2.0		GT = 250	
596.04	0	0.5	2500		{ 2.0 }	CNO = 3.538	
594.8	0						
604.0	0	0.5	5500.0	2.14		GT = 5502.1	ENDF-B-4
617.0	0	0.5	4500.0	2.14		GT = 4502.1	ENDF-B-4
627.0	0	0.5	7500.0	2.14		GT = 7502.1	ENDF-B-4
637.0	0	0.5	2500.0	2.14		GT = 2502.1	ENDF-B-4
652.0	0	0.5	7000.0	2.14		GT = 7002.1	ENDF-B-4

* A and B denote $g\Gamma_n$ and $g\Gamma_\gamma$, respectively

$$\begin{aligned} \text{** WW5} &= g\Gamma_n \Gamma_\gamma / \Gamma \quad (\text{eV}), & \text{GT} &= \Gamma \quad (\text{eV}) \\ \text{GGS} &= \sigma_0 \Gamma_\gamma \quad (\text{b} \cdot \text{eV}), & \text{WWC} &= \Gamma_n \Gamma_\gamma / \Gamma \quad (\text{eV}) \\ \text{WGH} &= g\Gamma_n^{(0)} \quad (\text{eV}), & \text{GNO} &= \Gamma_n^{(0)} \quad (\text{eV}) \\ \text{WGI} &= g\Gamma_n^{(1)} \quad (\text{eV}) \end{aligned}$$

References

- 66Farrell : Ref.(21)
- 69Hockenbury: Ref.(23)
- 70Stieglitz : Ref.(24)
- 71Garg : Ref.(49)
- 72Beer : Ref.(50)
- 73Moxon : Ref.(51) (Evaluation)
- 74Moxon : Ref.(3) (Evaluation)
- 75Fröhner : Ref.(52)
- 77Fröhner : Ref.(22)
- 77Syme : Ref.(20)

Table 8 Resonance parameters of ^{61}Ni

ENERGY (KEV)	L	J	NEUTRON WIDTH *	GAMMA WIDTH (EV)	WWS **	MISCELLANEOUS ***	REFERENCE
-1.8	0	1.5	212	0.62			JENDL-2
1.354	1	1.5	0.92	(1.0)	0.24 ± 0.03		JENDL-2
1.354	1	1.5	0.315	1.0		GT = 1.315	JENDL-1
1.35 ± 0.01						WWD = 0.48 ± 0.060	BNL325(3)
1.354	0	0.5	0.315	1.0	0.24 ± 0.03	GGS = 478 ± 60	73MOXON
1.354	1	1.5	0.32	(1.0)	0.24 ± 0.03		74MOXON
1.354							69HOCKENBURY
1.35							72HOCKENBURY
2.35	1	1.5	0.01	1.0		GT = 1.01	JENDL-1
2.35 ± 0.01							BNL325(3)
2.35	0	0.5	0.01	1.0			73MOXON
2.35	1	1.5	(0.01)	(1.0)			74MOXON
2.35							69HOCKENBURY
2.35							72HOCKENBURY
3.14	1	1.5	0.20	(1.0)	0.084 ± 0.018		JENDL-2
3.14	1	1.5	0.092	1.0		GT = 1.092	JENDL-1
3.14 ± 0.01						WWD = 0.17 ± 0.04	BNL325(3)
3.14	0	0.5	0.092	1.0			73MOXON
3.14	1	1.5	0.092	(1.0)	0.084 ± 0.018	GGS = 71 ± 14	74MOXON
3.14					0.084 ± 0.018		69HOCKENBURY
3.14							72HOCKENBURY
3.30	1	1.5	24.0	(1.0)	0.48 ± 0.06		JENDL-2
3.3	1	1.5	0.92	1.0		GT = 1.92	JENDL-1
3.30 ± 0.01						WWD = 0.96 ± 0.12	BNL325(3)
3.30	0	0.5	0.92	1.0			73MOXON
3.30	1	1.5	0.92	(1.0)	0.48 ± 0.06	GGS = 341 ± 44	74MOXON
3.30					0.48 ± 0.06		69HOCKENBURY
3.30							72HOCKENBURY
4.391							72HOCKENBURY
4.591							72HOCKENBURY
6.38	1	1.5	0.52	(1.0)	0.17 ± 0.04		JENDL-2
6.36 ± 0.01					0.17 ± 0.04		BNL325(3)
6.36							72HOCKENBURY
6.38 ± 0.02	1						77FROEHNERR
6.47	1	1.5	15.7	(1.0)	0.47 ± 0.10		JENDL-2
6.47	1	1.5	0.54	1.0		GT = 1.54	JENDL-1
6.47 ± 0.01						WWD = 0.70 ± 0.20	BNL325(3)
6.47	0	0.5	0.54	1.0			73MOXON
6.47	1	1.5	0.54	(1.0)	0.35 ± 0.10	GGS = 145 ± 43	74MOXON
6.47					0.35 ± 0.10		69HOCKENBURY
6.46							72HOCKENBURY
6.47 ± 0.01	1				0.47 ± 0.10		77FROEHNERR
7.12	0	0.5	3.54	1.0	0.78 ± 0.12	GGS = 285 ± 42	73MOXON
7.12							69HOCKENBURY
7.11							72HOCKENBURY
7.15	0	1	(74)		2.53		JENDL-2
7.15	0	1.0	74.0		2.5	GT = 76.5	JENDL-1
7.15 ± 0.02	0	1	R 50 ± 6		2.5 ± 0.4	WWD = 0.59 ± 0.07	BNL325(3)
7.15	0	1	74		2.5		73MOXON
7.152	0	1	74		2.5		74MOXON
6.97			23			GNO = 0.28	66GOOD
7.15			74				70CHO
7.15 ± 0.02	0	0			2.5 ± 0.5	WGM = 74 ± 8	72BEER ***
7.15 ± 0.02	0	1	74	* 8	2.55 ± 0.35	WGM = 74 ± 8	75FROEHNERR
7.15 ± 0.01	0	1				WMI = 2.53 ± 0.42	77FROEHNERR ***
7.53	0	0.5	R 0.1		1.0		73MOXON
7.53							69HOCKENBURY
7.52							72HOCKENBURY
7.57	0	2	(177)		2.19		JENDL-2
7.55	0	2.0	177.0		2.3	GT = 179.3	JENDL-1
7.55 ± 0.02	0	2	R 225 ± 20		2.3 ± 0.6	WWD = 2.59 ± 0.23	BNL325(3)
7.55	0	2	177		2.3		73MOXON
7.545	0	2	177		2.3		74MOXON
7.37			238			GNO = 2.79	66GOOD
7.55			177				70CHO
7.54 ± 0.02	0	2			2.3 ± 0.6	WGM = 177 ± 16	72BEER
7.58 ± 0.02	0	2	177	* 16	2.23 ± 0.35	WGM = (177)	75FROEHNERR
7.57 ± 0.01	0	2				WMI = 2.19 ± 0.14	77FROEHNERR
8.71	0	0.5	R 1.86		1.0		73MOXON
8.71							69HOCKENBURY
8.70							72HOCKENBURY
8.75	0	2	(6)		2.20		JENDL-2
8.74	0	2.0	6.0		2.6	GT = 8.6	JENDL-1
8.73 ± 0.02	0	2	R 7.5 ± 2.5		2.6 ± 0.8	WWD = 0.080 ± 0.027	BNL325(3)
8.74	0	2	6		2.6		73MOXON
8.745	0	2	6		2.6		74MOXON
8.74			6			GNO = 6 ± 2	66GOOD
8.74 ± 0.20	0	2			2.6 ± 0.8		70CHO
8.75 ± 0.02	0	2	6	* 2	2.6 ± 0.8	WGM = 6 ± 2	72BEER
8.75 ± 0.02	0	2				WMI = 2.20 ± 0.30	75FROEHNERR
8.75							77FROEHNERR

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
9.94	1	1.5	0.22	(1.0)	0.09 ± 0.02		JENDL-2
9.91	1	1.5	0.22	1.0		GT = 1.22	JENDL-1
9.91 ± 0.02	1	1.5	0.099	(1.0)	0.09 ± 0.03	WWD = 0.18 ± 0.06	BNL325(3) 74MOXON 69HOCKENBURY 72BEER
9.93 ± 0.02	1	1.5	0.099	(1.0)	0.09 ± 0.03	WWD = 0.09 ± 0.03	72HOCKENBURY 77FROEHNER ***
9.90					0.09 ± 0.02		
9.93 ± 0.02	1	1.5					
9.87	1	1.5					
9.94 ± 0.06	1						
10.17	1	1.5	0.47	(1.0)	0.16 ± 0.02		JENDL-2
10.2	1	1.5	0.613	1.0		GT = 1.613	JENDL-1
10.20 ± 0.03	1	1.5	0.24	(1.0)	0.19 ± 0.05	WWD = 0.38 ± 0.10	BNL325(3) 74MOXON 69HOCKENBURY 72BEER
10.18 ± 0.03	1	1.5	0.24	(1.0)	0.19 ± 0.05	WWD = 0.19 ± 0.05	72HOCKENBURY 77FROEHNER
10.2							
10.18 ± 0.03	1	1.5			0.16 ± 0.02		
10.1							
10.17 ± 0.06	1						
10.90 ± 0.03							BNL325(3) 72HOCKENBURY
10.9							
11.4 ± 0.03							BNL325(3) 72HOCKENBURY
11.4							
11.8 ± 0.03							BNL325(3) 72HOCKENBURY
11.8							
12.67	0	2	(75)	1.75			JENDL-2
12.64	0	2.0	75.0	1.7			JENDL-1
12.64 ± 0.03	0	2	R 90 ± 10	1.7 ± 0.4		WWD = 0.80 ± 0.09	BNL325(3) 73MOXON 74MOXON
12.64	0	2	75	1.7			
12.64	0	2	75	1.7			
12.4			67.7				GNO = 0.61
12.6							66G000
12.64			75				69HOCKENBURY 70CHO
12.64 ± 0.03	0	2		1.7 ± 0.4			72BEER
12.6							72HOCKENBURY
12.67 ± 0.03	0	2	75 ± 4	1.72 ± 0.25			75FROEHNER
12.67 ± 0.03	0	2				WWD = (75)	77FROEHNER
						WWI = 1.75 ± 0.30	
13.42	1	1.5	1.38	(1.0)	0.29 ± 0.04		JENDL-2
13.43 ± 0.03	1	1.5	0.525	(1.0)	0.31 ± 0.08		74MOXON 72BEER
13.43 ± 0.03	1						72HOCKENBURY
13.3							77FROEHNER
13.42 ± 0.03	1				0.29 ± 0.04		
13.67	0	2	(61)	1.70			JENDL-2
13.63	0	2.0	61.0	1.6			JENDL-1
13.60 ± 0.03	0	2	R 76 ± 5	1.6 ± 0.4		WWD = 0.65 ± 0.04	BNL325(3) 73MOXON 74MOXON
13.63	0	2	61	1.6			
13.63	0	2	61	1.6			
13.3			75.6				66G000
13.63	0	2	61	1.6 ± 0.4			70CHO
13.63 ± 0.03	0	2					72BEER
13.5							72HOCKENBURY
13.68 ± 0.05	0	2	61 ± 4	1.65 ± 0.25			75FROEHNER
13.67 ± 0.03	0	2				WWD = (61)	77FROEHNER
						WWI = 1.70 ± 0.28	
14.06	0	1	(17)	3.1			JENDL-2
14.02	0	1.0	17.0	3.1			JENDL-1
14.02 ± 0.03	0	1	R 13 ± 3	3.1 ± 0.5		WWD = 0.11 ± 0.03	BNL325(3) 73MOXON 74MOXON
14.02	0	1	17	3.1			
14.02	0	1	17	3.1			
13.7			13.0				66G000
14.0							69HOCKENBURY
14.02			17				70CHO
14.02 ± 0.03	0	1		3.1 ± 0.5			72BEER
13.9							72HOCKENBURY
14.06 ± 0.05	0	1	17 ± 4	3.20 ± 0.45			75FROEHNER
14.06 ± 0.03	0	1				WWD = (17)	77FROEHNER
						WWI = 3.1 ± 0.5	
14.45	1	1.5	1.63	(1.0)	0.31 ± 0.05		JENDL-2
14.45	1	1.5	1.5	1.0			JENDL-1
14.45 ± 0.04	1	1				GT = 2.5	BNL325(3) 74MOXON
14.45 ± 0.04	1	1.5	0.4	(1.0)	0.30 ± 0.08	WWD = 0.60 ± 0.60	69HOCKENBURY 72BEER
14.3							72HOCKENBURY
14.45 ± 0.04	1	1				WWD = 0.30 ± 0.08	77FROEHNER
14.3							
14.45 ± 0.04	1				0.31 ± 0.05		
15.44	1	1.5	0.56	(1.0)	0.18 ± 0.03		JENDL-2
15.38	1	1.5	0.515	1.0			JENDL-1
15.38 ± 0.04	1	1				GT = 1.515	BNL325(3) 74MOXON
15.38 ± 0.04	1	1.5	0.2	(1.0)	0.17 ± 0.04	WWD = 0.34 ± 0.08	69HOCKENBURY 72BEER
15.3							72HOCKENBURY
15.38 ± 0.04	1	1				WWD = 0.17 ± 0.04	77FROEHNER
15.3							
15.44 ± 0.04	1				0.18 ± 0.03		
15.72	1	1.5	0.28	(1.0)	0.11 ± 0.02		JENDL-2
15.72 ± 0.04	1				0.11 ± 0.02		77FROEHNER
16.61	0	1	(1817)	2.3			JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
16.7	0	1.0	817.0	2.2		GT = 819.2	JENDL-1
16.70 ± 0.05	0	1	8600 ± 20	2.2 ± 0.4		WGO = 4.64 ± 0.15	BNL325(3)
16.70	0	1	817	2.2			73MOXON
16.7	0	1	814	2.2			74MOXON
16.3			411			GND = 3.21	66G000
16.7							69HOCKENBURY
16.70		1	817				70CHO
16.70 ± 0.05	0	1		2.2 ± 0.4		WGM = 817 ± 16	72BEER
16.7							72HOCKENBURY
16.61 ± 0.10	0	1	817 ± 16	2.07 ± 0.30		WGM = 817	75FROEHNERR
16.61 ± 0.05	0	1				WWI = 2.3 ± 0.4	77FROEHNERR
16.82	1	1.5	0.56	(1.0)	0.18 ± 0.03	GT = 1.389	JENDL-2
16.8	1	1.5	0.389	1.0		WHD = 0.28 ± 0.08	JENDL-1
16.80 ± 0.05	1	1.5	0.16	(1.0)	0.14 ± 0.04	WHC = 0.14 ± 0.04	BNL325(3)
16.80 ± 0.05	1	1.5			0.18 ± 0.03		74MOXON
16.82 ± 0.05	1						72BEER
17.93	0	1	(177)	4.1		GT = 178.6	JENDL-2
17.86	0	1.0	177.0	1.6		WGO = 1.05 ± 0.08	JENDL-1
17.83 ± 0.05	0	1	8140 ± 10	1.6 ± 0.5			BNL325(3)
17.86	0	1	177	1.6			73MOXON
17.86	0	1	177	1.6			74MOXON
17.5			174			GND = 1.32	66G000
17.8							69HOCKENBURY
17.86	0	1	177	1.6 ± 0.5			70CHO
17.86 ± 0.05	0	1			WGM = 177 ± 8	72BEER	
17.7							72HOCKENBURY
17.99 ± 0.10	0	1	177 ± 8	1.4 ± 0.4		WGM = (177)	75FROEHNERR
17.93 ± 0.05	0	1				WWI = 4.1 ± 0.7	77FROEHNERR
18.97	0	2	(69)	0.78		GT = 69.9	JENDL-2
18.87	0	2.0	69.0	0.9		WHD = 0.66 ± 0.07	JENDL-1
18.83 ± 0.05	0	2	890 ± 10	0.9 ± 0.3			BNL325(3)
18.87	0	2	69	0.9			73MOXON
18.87	0	2	69	0.9			74MOXON
18.3			181			GND = 1.34	66G000
19.0							69HOCKENBURY
18.87	0	2	69				70CHO
18.87 ± 0.05	0	2		0.9 ± 0.3		WGM = 69 ± 4	72BEER
18.8							72HOCKENBURY
18.97 ± 0.10	0	2	69 ± 4	0.78 ± 0.11		WGM = (69)	75FROEHNERR
18.97 ± 0.06	0	2				WWI = 0.78 ± 0.13	77FROEHNERR
20.35	1	1.5	0.22	(1.0)	0.09 ± 0.02	GT = 1.22	JENDL-2
20.25	1	1.5	0.22	1.0		WHD = 0.18 ± 0.06	JENDL-1
20.25 ± 0.05	1	1	0.099	(1.0)	0.09 ± 0.3		BNL325(3)
20.25 ± 0.05	0	1.5					74MOXON
20.4							69HOCKENBURY
20.25 ± 0.05	1	1					72BEER
20.25					0.09 ± 0.02		72HOCKENBURY
20.35 ± 0.06	1						77FROEHNERR
20.67	1	1.5	0.22	(1.0)	0.09 ± 0.02	GT = 1.282	JENDL-2
20.5	1	1.5	0.282	1.0		WHD = 0.22 ± 0.06	JENDL-1
20.50 ± 0.05	1	1	0.125	(1.0)	0.11 ± 0.3		BNL325(3)
20.55 ± 0.05	1	1	0.125				74MOXON
20.55 ± 0.05	1	1					72BEER
20.4					0.09 ± 0.02		72HOCKENBURY
20.67 ± 0.06	1						77FROEHNERR
21.41	1	1.5	1.63	(1.0)	0.31 ± 0.10	GT = 16.7	JENDL-2
21.35	1	1.5	14.7	2.0		WHD = 1.76 ± 0.40	JENDL-1
21.35 ± 0.05	0	1					BNL325(3)
21.40 ± 0.05	1	1	7.3	(1.0)	0.86 ± 0.2		74MOXON
21.40 ± 0.05	0	1					72BEER
21.3					0.31 ± 0.10		72HOCKENBURY
21.41 ± 0.10	1						77FROEHNERR
21.61	1	1.5	4.42	(1.5)	0.56 ± 0.19		JENDL-2
21.61 ± 0.10	1				0.56 ± 0.19		77FROEHNERR
24.17	1	1.5	1.94	(1.0)	0.33 ± 0.05	GT = 3.57	JENDL-2
24.12	1	1.5	2.57	1.0		WHD = 0.72 ± 0.18	JENDL-1
24.12 ± 0.05	1	1	0.56	(1.0)	0.36 ± 0.09		BNL325(3)
24.12 ± 0.05	1	1					74MOXON
24.12 ± 0.05	1	1			0.33 ± 0.05		72BEER
24.17 ± 0.07	1						77FROEHNERR
24.62	0	1	(129)	1.4		GT = 130.4	JENDL-2
24.62	0	1.0	129.0	1.4		WGO = 0.62 ± 0.05	JENDL-1
24.62 ± 0.06	0	1	897 ± 8	1.4 ± 0.3			BNL325(3)
24.62	0	1	129	1.4			73MOXON
24.62	0	1	129	1.4			74MOXON
23.8			100			GND = 0.64	66G000
24.8					3.98 ± 1.3		69HOCKENBURY
24.62	1	1	129				70CHO
24.62 ± 0.06	0	1		1.4 ± 0.3			72BEER
24.73 ± 0.07	0	1	129 ± 10	1.41 ± 0.20			75FROEHNERR
24.62 ± 0.07	0	1					77FROEHNERR
25.28	1	1.5	1.0	(1.0)	0.25 ± 0.05		JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WWS (EV)	MISCELLANEOUS	REFERENCE
25.12 ± 0.06	1	1	1.5	0.33	(1.0)	WWD = 0.50 ± 0.12	BNL325(3) 74MOXON 72BEER 77FROEHNER
25.12 ± 0.06	1	1	1.5	0.33	(1.0)	WWD = 0.25 ± 0.06	
25.12 ± 0.06	1	1	1.5	0.33	(1.0)	WWD = 0.25 ± 0.06	
25.28 ± 0.07	1					0.25 ± 0.05	
26.10	1	1.5	0.92	(1.0)	0.24 ± 0.06		
25.96	1	1.5	0.923	1.0		GT = 1.923	JENDL-2 JENDL-1
25.96 ± 0.06	1	1	1.5	0.31	(1.0)	WWD = 0.48 ± 0.12	BNL325(3) 74MOXON 72BEER 77FROEHNER
25.96 ± 0.06	1	1	1.5	0.31	(1.0)	WWD = 0.24 ± 0.06	
25.96 ± 0.06	1	1	1.5	0.31	(1.0)	WWD = 0.24 ± 0.06	
25.10 ± 0.10	1					0.24 ± 0.06	
26.58	1	1.5	0.56	(1.0)	0.18 ± 0.05		
26.45	1	1.5	0.563	1.0		GT = 1.563	JENDL-2 JENDL-1
26.45 ± 0.05	1	1	1.5	0.22	(1.0)	WWD = 0.36 ± 0.10	BNL325(3) 74MOXON 72BEER 77FROEHNER
26.45 ± 0.06	1	1	1.5	0.22	(1.0)	WWD = 0.18 ± 0.05	
26.45 ± 0.06	1	1	1.5	0.22	(1.0)	WWD = 0.18 ± 0.05	
26.58 ± 0.07	1						
27.22	1	1.5	0.67	(1.0)	0.20 ± 0.05		
27.1	1	1.5	0.667	1.0		GT = 1.667	JENDL-2 JENDL-1
27.10 ± 0.07	1	1	1.5	0.25	(1.0)	WWD = 0.40 ± 0.10	BNL325(3) 74MOXON 72BEER 77FROEHNER
27.10 ± 0.07	1	1	1.5	0.25	(1.0)	WWD = 0.20 ± 0.05	
27.10 ± 0.07	1	1	1.5	0.25	(1.0)	WWD = 0.20 ± 0.05	
27.22 ± 0.07	1						
27.78	1	1.5	10.0	(1.0)	0.45 ± 0.09		
27.65	1	1.5	4.0	1.0		GT = 5.0	JENDL-2 JENDL-1
27.65 ± 0.07	1	1	1.5	0.67	(1.0)	WWD = 0.80 ± 0.20	BNL325(3) 74MOXON 69HOCKENBURY
27.65	1	1	1.5	0.67	(1.0)	WWD = 1.74 ± 0.85	
27.65 ± 0.07	1	1	1.5	0.67	(1.0)	WWD = 0.40 ± 0.10	72BEER 77FROEHNER
27.78 ± 0.08	1					0.45 ± 0.09	
28.15	1	1.5	2.33	(1.0)	0.35 ± 0.13		
28.15 ± 0.08	1					0.35 ± 0.13	JENDL-2 77FROEHNER
28.40	0	2	(5)	1.09	0.56 ± 0.20		
28.21	0	2.0	3.0	3.0		GT = 6.0	JENDL-2 JENDL-1
28.21 ± 0.07	0	2	5.3 ± 5.0	3.0 ± 1.0		WWD = 0.038 ± 0.030	BNL325(3) 73MOXON 74MOXON 70CHO
28.21	0	2	3	3.0			
28.21	0	2	5	3.0			
28.21	0	2	3	3.0			
28.21 ± 0.07	0	2		3.0 ± 1		WWD = 5 ± 4	72BEER 75FROEHNER
28.35 ± 0.07	0	2	5 ± 4	2.2 ± 0.8		WWD = (5)	77FROEHNER
28.40 ± 0.08	0	2			0.56 ± 0.20		
29.08	0	1	(409)	1.7			
29.11	0	1.0	409.0	2.4		GT = 411.4	JENDL-2 JENDL-1
29.11 ± 0.07	0	1	310 ± 20	2.4 ± 0.4		WWD = 1.82 ± 0.12	BNL325(3) 73MOXON 74MOXON
29.11	0	1	409	2.4			
29.11	0	1	409	2.4			
29.2			236			CNO = 1.40	69G00D 69HOCKENBURY
29.0							70CHO
29.11		1	409				72BEER
29.11 ± 0.07	0	1		2.4 ± 0.4		WWD = 409 ± 22	
29.3 ± 0.1	0	1	409 ± 22	1.6 ± 0.3			75FROEHNER
29.08 ± 0.08	0	1				WWD = (409)	77FROEHNER
29.08 ± 0.08	0	1				WWD = 1.7 ± 0.4	
29.35	1	1.5	3.0	(1.5)	0.50 ± 0.20		
29.35 ± 0.08	1				0.50 ± 0.20		JENDL-2 77FROEHNER
30.10	1	1.5	0.28	(1.0)	0.11 ± 0.04		
30.10 ± 0.08	1				0.11 ± 0.04		JENDL-2 77FROEHNER
30.64	0	2	(15)	2			
30.64	0	2.0	13.0	2.0		GT = 15.0	JENDL-2 JENDL-1
30.64 ± 0.08	0	2	19 ± 10			WWD = 0.11 ± 0.06	BNL325(3) 73MOXON 74MOXON
30.64	0	2	13	2.0			
30.64 ± 0.08	0	2	15 ± 8	(2.0)			
30.2			423			CNO = 2.43	69G00D 69HOCKENBURY
30.8							70CHO
30.64		2	13				72BEER
30.64 ± 0.08	0	2				WWD = 15 ± 8	
30.64 ± 0.08	0	2				WWD = 15 ± 8	77FROEHNER
30.64 ± 0.08	0	2				WWD = 2 ± 1	
31.13	0	1.0	788.0	2.0		GT = 780.0	JENDL-2
31.13	0	1.0	788.0	2.0		GT = 790.0	JENDL-1
31.13	0	1	788	2.0			73MOXON
31.13 ± 0.08	0	1	788 ± 28	(2.0)			74MOXON
31.13	0	1	788				70CHO
31.13 ± 0.08	0	1					72BEER
31.13	0	1				WWD = 788 ± 28	
31.83	0	2.0	10.0	2.0		GT = 12.0	
31.83	0	2.0	10.0	2.0		GT = 12.0	JENDL-2 JENDL-1
31.83 ± 0.08	0	2	12.5 ± 7.5			WWD = 0.070 ± 0.042	BNL325(3) 73MOXON 74MOXON
31.83	0	2	8	2.0			
31.83 ± 0.08	0	2	10 ± 6	(2.0)			
31.6			392			CNO = 2.20	69G00D 69HOCKENBURY
31.7							70CHO
31.83		2	8				72BEER
31.83 ± 0.08	0	2				WWD = 10 ± 6	
32.7	0	2.0	220.0	2.0		GT = 222.0	JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
32.7	0	2.0	220.0	2.0			
32.70 ± 0.08	0	2	^a 265 ± 25			GT = 222.0 WGO = 1.47 ± 0.14	JENDL-1 BNL325(3) 73MOXON
32.70	0	2	220	2.0			74MOXON
32.70 ± 0.08	0	2	220 ± 10	(2.0)		GNO = 0.66	66G00D
32.7			120				70CHO
32.70		2	220			WGM = 220 ± 10	72BEER
32.70 ± 0.08	0	2					
33.68	0	1.0	58.0	2.8		GT = 60.8	JENDL-2
33.68	0	1.0	58.0	2.8		GT = 60.8	JENDL-1
33.68 ± 0.08	0	1	^a 50 ± 10	2.8 ± 0.5		WGO = 0.27 ± 0.05	BNL325(3) 73MOXON
33.68	0	1	58	2.8		74MOXON	
33.68	0	1	58	2.8		66G00D	
33.68			123			69HOCKENBURY	
33.8						70CHO	
33.68		1	58			72BEER	
33.68 ± 0.08	0	1		2.8 ± 0.5		WGM = 58 ± 10	77FROEHNERR
33.68 ± 0.08	0	1				WHA = 2.8 ± 0.5	
34.65 ± 0.1	1	1.5	(0.1)	(1.0)			74MOXON
34.65 ± 0.10	1	1					72BEER
36.02 ± 0.1	1	1.5	(0.1)	(1.0)			74MOXON
36.02 ± 0.10	1	1					72BEER
37.13	0	2.0	133.0	3.0		GT = 136.0	JENDL-2
37.13	0	2.0	133.0	3.0		GT = 136.0	JENDL-1
37.13 ± 0.09	0	2	^a 180 ± 20	3.0 ± 0.5		WGO = 0.93 ± 0.10	BNL325(3) 73MOXON
37.13	0	2	133	3.0		74MOXON	
37.13	0	2	133	3.0		66G00D	
36.0			294			69HOCKENBURY	
37.3						70CHO	
37.13		2	133			72BEER	
37.13 ± 0.09	0	2		3.0 ± 0.5		WGM = 133 ± 12	77FROEHNERR
37.13 ± 0.09	0	2				WHA = 3.0 ± 0.5	
39.77 ± 0.11	1	1					72BEER
41.34	0	1.0	176.0	2.0		GT = 178.0	JENDL-2
41.34	0	1.0	176.0	2.0		GT = 178.0	JENDL-1
41.34 ± 0.10	0	1	^a 150 ± 20			WGO = 0.74 ± 0.10	BNL325(3) 73MOXON
41.34	0	1	176	2.0		74MOXON	
41.34	0	1	176	(2.0)		66G00D	
40.0			243			69HOCKENBURY	
41.3						70CHO	
41.34	1	176				72BEER	
41.34 ± 0.10	0	1				WGM = 176 ± 22	
43.25	0	2.0	10.0	2.0		GT = 12.0	JENDL-2
43.25	0	2.0	10.0	2.0		GT = 12.0	JENDL-1
43.25 ± 0.11	0	2	^a 12.5 ± 10.0			WGO = 0.060 ± 0.048	BNL325(3) 73MOXON
43.25	0	2	10	2.0		66G00D	
42.2			133			70CHO	
43.25	2	10				72BEER	
43.25 ± 0.11	0	2				WGM = 10 ± 8	
43.61	0	2.0	30.0	2.0		GT = 32.0	JENDL-2
43.6	0	2.0	30.0	2.0		GT = 32.0	JENDL-1
43.61 ± 0.11	0	2	^a 37.5 ± 17.5			WGO = 0.18 ± 0.08	BNL325(3) 73MOXON
43.60	0	2	30	2.0		74MOXON	
43.60	0	2	30	(2.0)		70CHO	
43.61	0	2	30			72BEER	
43.61 ± 0.11	0	2				WGM = 30 ± 14	
45.49	0	1.0	66.0	2.0		GT = 68.0	JENDL-2
45.49	0	1.0	66.0	2.0		GT = 68.0	JENDL-1
45.49 ± 0.11	0	1	^a 50 ± 6			WGO = 0.23 ± 0.03	BNL325(3) 73MOXON
45.49	0	1	66	2.0		74MOXON	
44.0	0	1	169	(2.0)		66G00D	
45.49	1	66				70CHO	
45.49 ± 0.11	0	1				72BEER	
45.49			WGM = 66 ± 8				
46.16	0	1.0	54.0	2.0		GT = 56.0	JENDL-2
46.16	0	1.0	54.0	2.0		GT = 56.0	JENDL-1
46.16 ± 0.12	0	1	^a 40.5 ± 6.0			WGO = 0.19 ± 0.03	BNL325(3) 73MOXON
46.16	0	1	54	2.0		74MOXON	
46.16	0	1	54	(2.0)		69HOCKENBURY	
46.1						70CHO	
46.16	1	54				72BEER	
46.16 ± 0.12	0	1				WGM = 54 ± 8	
50.51	0	1.0	133.0	2.0		GT = 135.0	JENDL-2
50.51	0	1.0	133.0	2.0		GT = 135.0	JENDL-1
50.51 ± 0.12	0	1	^a 100 ± 9			WGO = 0.45 ± 0.04	BNL325(3) 73MOXON
50.51	0	1	133	2.0		74MOXON	
50.51	0	1	133	(2.0)		66G00D	
48.4			83			69HOCKENBURY	
50.7						70CHO	
50.51	1	133				72BEER	
50.51 ± 0.12	0	1				WGM = 133 ± 12	
53.3	0	2.0	141.0	2.0		GT = 143.0	JENDL-2
53.3	0	2.0	141.0	2.0		GT = 143.0	JENDL-1

JAERI-M 85-101

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WWS (EV)	MISCELLANEOUS	REFERENCE
53.30 ± 0.13	0	2	^a 176 ±13			WGO = 0.76 ± 0.06	BNL325(3) 73MOXON 74MOXON 70CHO 72BEER
53.30	0	2	141	2.0			
53.30	0	2	141	[2.0]			
53.30		2	141				
53.30 ± 0.13	0	2				WGM = 141 ±10	
54.81	0	1.0	189.0	2.0		GT = 191.0	JENDL-2
54.81	0	1.0	189.0	2.0		GT = 191.0	JENDL-1
54.81 ± 0.14	0	1	^a 142 ±14			WGO = 0.61 ± 0.06	BNL325(3) 73MOXON 74MOXON 70CHO 72BEER
54.81	0	1	189	2.0			
54.81	0	1	189	[2.0]			
54.81		1	189				
54.81 ± 0.14	0	1				WGM = 189 ±18	
55.49	0	2.0	119.0	2.0		GT = 121.0	JENDL-2
56.49	0	2.0	119.0	2.0		GT = 121.0	JENDL-1
56.49 ± 0.14	0	2	^a 149 ±13			WGO = 0.63 ± 0.06	BNL325(3) 73MOXON 74MOXON 70CHO 72BEER
56.49	0	2	119	2.0			
56.49	0	2	119	[2.0]			
56.49	0	2	119				
56.49 ± 0.14	0	2				WGM = 119 ±10	
58.16	0	1.0	178.0	2.0		GT = 180.0	JENDL-2
58.16	0	1.0	178.0	2.0		GT = 180.0	JENDL-1
58.16 ± 0.15	0	1	^a 133 ±15			WGO = 0.55 ± 0.06	BNL325(3) 73MOXON 74MOXON 69HOCKENBURY 70CHO 72BEER
58.16	0	1	178	2.0			
58.16	0	1	178	[2.0]			
58.16		1	178				
58.16 ± 0.15	0	1				WGM = 178 ±20	
64.07	0	2.0	54.0	2.0		GT = 56.0	JENDL-2
64.07	0	2.0	54.0	2.0		GT = 56.0	JENDL-1
64.07 ± 0.16	0	2	^a 68 ± 6			WGO = 0.27 ± 0.02	BNL325(3) 73MOXON 74MOXON 70CHO 72BEER
64.07	0	2	535	2.0			
64.07	0	2	535	[2.0]			
64.07	0	2	535				
64.07 ± 0.16	0	2				WGM = 54 ± 2	
65.87	0	2.0	1430.0	2.0		GT = 1432.0	JENDL-2
65.87	0	2.0	1430.0	2.0		GT = 1432.0	JENDL-1
65.87 ± 0.16	0	2	^a 1790 ±225			WGO = 6.97 ± 0.09	BNL325(3) 73MOXON 74MOXON 70CHO 72BEER
65.87	0	2	1430	2.0			
65.87	0	2	1430	[2.0]			
65.87	0	2	1430				
65.87 ± 0.16	0	2				WGM = 1430 ±180	
68.77	0	2.0	1100.0	2.0		GT = 1102.0	JENDL-2
68.77	0	2.0	1100.0	2.0		GT = 1102.0	JENDL-1
68.77 ± 0.17	0	2	^a 1375 ±625			WGO = 5.24 ± 2.38	BNL325(3) 73MOXON 74MOXON 70CHO 72BEER
68.77	0	2	1100	2.0			
68.77	0	2	1100	[2.0]			
68.77	0	2	1100				
68.77 ± 0.17	0	2				WGM = 1100 ±500	
70.8							69HOCKENBURY
89.6							69HOCKENBURY

* A denotes $2g\Gamma_n$

** WW5 = $g\Gamma_n\Gamma_\gamma/\Gamma$ (eV), GT = Γ (eV)

WWD = $2g\Gamma_n\Gamma_\gamma/\Gamma$ (eV), GGS = $\sigma_0\Gamma_\gamma$ (b·eV)

WGO = $2g\Gamma_n^{(0)}$ (eV), WGM = $g\Gamma_n$ (eV)

WWI = $g\Gamma_\gamma$ (eV), WWC = $\Gamma_n\Gamma_\gamma/\Gamma$ (eV)

GNO = $\Gamma_n^{(0)}$ (eV)

*** 1) $g\Gamma_n$ reported in 72 Beer and 77 Fröhner should probably be read as Γ_n .

Γ_n .

2) $g\Gamma_\gamma$ reported in 77 Fröhner should probably be read as Γ_γ .

3) $\Gamma_n\Gamma_\gamma/\Gamma$ reported in 72 Beer should probably be read as $g\Gamma_n\Gamma_\gamma/\Gamma$.

References

66Good : Ref.(53)

69Hockenbury: Ref.(23)

70Cho : Ref.(25)

72Beer : Ref.(50)

72Hockenbury: Ref.(54)

73Moxon : Ref.(51)

74Moxon : Ref.(3)

75Fröhner : Ref.(52)

77Fröhner : Ref.(22)

Table 9 Resonance parameters of ^{62}Ni

ENERGY (KEV)	L	J	NEUTRON WIDTH*	GAMMA WIDTH*	WW5**	MISCELLANEOUS**	REFERENCE
							69HOCKENBURY
2.34							
4.6	0	0.5	2026	2.375			JENDL-2
4.6	0	0.5	1820.0	2.31			JENDL-1
4.6	0	0.5	1700.0	2.14			ENDF-B-4
4.54 ± 0.05	0	0.5	1600 ± 160	0.76 ± 0.12		WGH = 23.75 ± 2.29	BNL325(3)
4.599	0		2075	2.31			73MOXON
4.599	0	0.5	2075	2.31			74MOXON
6.0	0					CNO = 1130.0	66FARRELL
4.6			{1300}	0.76 ± 0.12			69HOCKENBURY
4.54 ± 0.05	0		1340 ± 90			CNO = 19.89 ± 1.34	71GARG
8.91	1	0.5	0.089	1.0		GT = 1.089	JENDL-2
8.91	1	0.5	0.089	1.0		GT = 1.089	JENDL-1
8.91 ± 0.1	1	0.5	0.089	{1.0}	0.082 ± 0.016		74MOXON
8.91 ± 0.10	> 0				0.082 ± 0.016		75BEER
9.42	1	0.5	0.315	1.0		GT = 1.315	JENDL-2
9.42	1	0.5	0.315	1.0		GT = 1.315	JENDL-1
9.42 ± 0.1	1	0.5	0.32	{1.0}	0.24 ± 0.04		74MOXON
9.42 ± 0.10	> 0				0.24 ± 0.04		75BEER
17.69	1	0.5	0.15	1.0		GT = 1.15	JENDL-2
17.69	1	0.5	0.113	1.0		GT = 1.113	JENDL-1
17.69 ± 0.07	1	0.5	0.15	{1.0}	0.13 ± 0.03		74MOXON
17.69 ± 0.07	> 0				0.13 ± 0.03		75BEER
24.46	1	0.5	0.205	1.0		GT = 1.205	JENDL-2
24.46	1	0.5	0.205	1.0		GT = 1.205	JENDL-1
24.46 ± 0.11	1	0.5	0.21	{1.0}	0.17 ± 0.03		74MOXON
24.46 ± 0.11	> 0				0.17 ± 0.03		75BEER
28.22	1	0.5	0.351	1.0		GT = 1.351	JENDL-2
28.22	1	0.5	0.351	1.0		GT = 1.351	JENDL-1
28.22 ± 0.14	1	0.5	0.35	{1.0}	0.26 ± 0.04		74MOXON
28.22 ± 0.14	> 0				0.26 ± 0.04		75BEER
29.29	1	0.5	1.0	1.0		GT = 2.0	JENDL-2
29.29	1	0.5	1.0	1.0		GT = 2.0	JENDL-1
29.29 ± 0.14	1	0.5	1.0	{1.0}	0.50 ± 0.07		74MOXON
29.29 ± 0.14	> 0				0.50 ± 0.07		75BEER
34.28	1	0.5	0.563	1.0		GT = 1.563	JENDL-2
34.28	1	0.5	0.563	1.0		GT = 1.563	JENDL-1
34.28 ± 0.18	1	0.5	0.56	{1.0}	0.36 ± 0.06		74MOXON
34.28 ± 0.18	> 0				0.36 ± 0.06		75BEER
38.04	1	0.5	3.35	1.0		GT = 4.35	JENDL-2
38.04	1	0.5	3.35	1.0		GT = 4.35	JENDL-1
38.04 ± 0.20	1	0.5	3.35	{1.0}	0.77 ± 0.11		74MOXON
38.04 ± 0.20	> 0				0.77 ± 0.11		75BEER
40.3	1	0.5	0.135	1.0		GT = 1.136	JENDL-2
40.3	1	0.5	0.136	1.0		GT = 1.136	JENDL-1
40.3 ± 0.3	> 0				0.12 ± 0.03		75BEER
41.0	1	0.5	0.235	1.0		GT = 1.235	JENDL-2
41.0	1	0.5	0.235	1.0		GT = 1.235	JENDL-1
41.0 ± 0.3	> 0				0.19 ± 0.04		75BEER
42.87	0	0.5	340.0	0.36		GT = 340.36	JENDL-2
42.87	0	0.5	340.0	0.35		GT = 340.36	JENDL-1
38.5	0	0.5	250.0	2.14		GT = 252.14	ENDF-B-4
42.87 ± 0.01	0	0.5	340 ± 10			WGH = 1.64 ± 0.05	BNL325(3)
42.872	0		340	2.0			73MOXON
42.87	0	0.5	340	0.36			74MOXON
42.5	0					CNO = 1000	66FARRELL
42.87 ± 0.14	0		340 ± 15	0.36 ± 0.07			75BEER
44.8	1	0.5	0.515	1.0		GT = 1.515	JENDL-2
44.8	1	0.5	0.515	1.0		GT = 1.515	JENDL-1
44.8 ± 0.3	> 0				0.34 ± 0.07		75BEER
53.1	1	0.5	0.176	1.0		GT = 1.176	JENDL-2
53.1	1	0.5	0.176	1.0		GT = 1.176	JENDL-1
53.1 ± 0.3	> 0				0.15 ± 0.05		75BEER
56.91	1	0.5	56.0	0.28		GT = 56.28	JENDL-2
56.91	1	0.5	56.0	0.28		GT = 56.28	JENDL-1
53.8	0	0.5	100.0	2.14		GT = 102.14	ENDF-B-4
56.91 ± 0.02	1	0.5	56 ± 4			WGH = 2.5	BNL325(3)
56.5	0					GT = 1200	66FARRELL
56.91 ± 0.19	> 0		56 ± 17	0.28 ± 0.11	0.28 ± 0.07	CNO = 0.83	75BEER
63.1	1	0.5	0.37	1.0		GT = 1.37	JENDL-2
63.1	1	0.5	0.37	1.0		GT = 1.37	JENDL-1
63.1 ± 0.4	> 0				0.27 ± 0.07		75BEER
74.0	1	0.5	0.887	1.0		GT = 1.887	JENDL-2
74.0	1	0.5	0.887	1.0		GT = 1.887	JENDL-1
74.0 ± 0.5	> 0				0.47 ± 0.09		75BEER
77.2	0	0.5	70.0	2.0		GT = 72.0	JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	MWS (EV)	MISCELLANEOUS	REFERENCE
77.23	0	0.5	70.0	2.0			
76.0	1	0.5	175.0	0.6			
77.23 ± 0.03	0	0.5	70 ± 7			WCH = 0.25 ± 0.03	JENDL-1 ENDF-B-4 BNL32S(3) 73MOXON 74MOXON 75BEER
77.23	0	0.5	70	2.0			
77.2 ± 0.3	0	0.5	70 ± 30	(2.0)			
77.2 ± 0.3	0	0.5	70 ± 30				
78.4	1	0.5	48.0	0.14			
78.42	1	0.5	48.0	0.14			
78.42 ± 0.04	1	0.5	48 ± 7				
78.4 ± 0.3	0	0.5	48 ± 7		0.14 ± 0.04		JENDL-2 JENDL-1 BNL32S(3) 75BEER
89.3 ± 0.35	0		250 ± 120			GNO = 2.64 ± 0.40	71GARG
93.4	1	0.5	1.22	1.0			
93.4	1	0.5	1.22	1.0			
93.4 ± 0.8	> 0	0.5			0.55 ± 0.12		JENDL-2 JENDL-1 75BEER
94.7	0	0.5	2500.0	0.56			
94.7	0	0.5	2500.0	0.56			
93.5	0	0.5	2250.0	2.14			
94.7 ± 0.02	0	0.5	2500 ± 100			WCH = 8.12 ± 0.33	JENDL-2 JENDL-1 ENDF-B-4 BNL32S(3) 73MOXON 74MOXON 66FARRELL
94.74	0	0.5	2500	2.0			
94.74	0	0.5	2440	0.56			
93.5	0	0.5					
95.5 ± 0.40	0		1620 ± 400			GNO = 7.483	71GARG
94.7 ± 0.4	0		2500 ± 100	0.56 ± 0.13		GNO = 5.29 ± 1.31	75BEER
103.3	1	1.5	1.86	1.0			
103.3	1	1.5	1.86	1.0			
103.3 ± 0.8	> 0	0.5			1.30 ± 0.22		JENDL-2 JENDL-1 75BEER
105.6	0	0.5	4600.0	1.4			
105.6	0	0.5	4600.0	1.4			
104.7	0	0.5	3700.0	2.14			
105.65 ± 0.03	0	0.5	4600 ± 200			WCH = 14.15 ± 0.62	JENDL-2 JENDL-1 ENDF-B-4 BNL32S(3) 73MOXON 74MOXON 66FARRELL
105.65	0	0.5	4600	2.0			
105.6	0	0.5	4600	1.4			
104.5	0	0.5					
104.5 ± 0.5	0		3850				
105.6 ± 0.4	0		4600 ± 200	1.40 ± 0.31		GNO = 14.183	71GARG
105.6	0					GNO = 11.90	75BEER
112.0	1	0.5	4.26	1.0			
112.0	1	0.5	4.26	1.0			
112.0 ± 0.9	> 0	0.5			0.81 ± 0.20		JENDL-2 JENDL-1 75BEER
118.5	1	1.5	3.17	1.0			
118.5	1	1.5	3.17	1.0			
118.5 ± 1.1	> 0	0.5			1.52 ± 0.33		JENDL-2 JENDL-1 75BEER
137.4	1	0.5	127.0	1.8			
137.4	1	0.5	113.0	1.8			
137.5	1	0.5	113.0	0.6			
137.5	1	0.5	127				
138.74	> 0	0.5	113	1.0			
137.5	> 0	0.5	126.5 ± 13.5				
137.4 ± 1.4	> 0	0.5			1.77 ± 0.44		66FARRELL 75BEER
145.0	1	1.5	309.0	1.55			
145.0	1	1.5	309.0	1.55			
145.0 ± 1.5	> 0	0.5			3.09 ± 0.62		JENDL-2 JENDL-1 75BEER
149.3	0	0.5	140.0	2.0			
149.3	0	0.5	140.0	2.0			
148.5	0	0.5	200.0	2.14			
149.3 ± 0.1	0	0.5	140 ± 20			WCH = 0.36 ± 0.05	JENDL-2 JENDL-1 ENDF-B-4 BNL32S(3) 73MOXON 74MOXON 66FARRELL
149.3	0	0.5	140	2.0			
149.3	0	0.5	140	(2.0)			
148.5	0	0.5					
148.5 ± 0.8	0		200				
149.3 ± 0.7	0		140 ± 70			GNO = 0.533	71GARG
149.3	0					GNO = 0.53	75BEER
160.5	1	1.5	21.9	1.45			
160.5	1	1.5	15.1	1.45			
160.5 ± 1.8	> 0	0.5			2.72 ± 0.60		JENDL-2 JENDL-1 75BEER
188.2	0	0.5	90.0	2.0			
188.2	0	0.5	90.0	2.0			
188.2 ± 0.2	0	0.5	90 ± 20			WCH = 0.21 ± 0.05	JENDL-2 JENDL-1 BNL32S(3) 73MOXON 74MOXON 75BEER
188.2	0	0.5	90	2.0			
188.2	0	0.5	90	(2.0)			
188.2 ± 0.9	0	0.5	(90)				
190.74	1	0.5	125.0	1.0			
190.74	1	0.5	125.0	1.0			
189.5	1	0.5	125.0	0.6			
190.74	> 0	0.5	125	1.0			
189.5	> 0	0.5	137.5 ± 12.5				
214.7	0	0.5	190.0	2.0			
214.7	0	0.5	190.0	2.0			
214.7 ± 0.2	0	0.5	190 ± 20			WCH = 0.41 ± 0.04	JENDL-2 JENDL-1 BNL32S(3) 73MOXON
214.7	0	0.5	190	2.0			

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
214.7	0	0.5	190	(2.0)			74MOXON
214.7 ± 1.1	0		190 ± 130				75BEER
217.74	1	0.5	175.0	1.0		GT = 176.0	JENDL-2
217.74	1	0.5	175.0	1.0		GT = 176.0	JENDL-1
216.5	1	0.5	175.0	0.6		GT = 175.6	ENDF-B-4
217.74	0	0.5	175	1.0			73MOXON
216.5	0		*190.5 ± 15.5				66FARRELL
229.5	0	0.5	6180.0	2.0		GT = 6182.0	JENDL-2
229.5	0	0.5	6180.0	2.0		GT = 6182.0	JENDL-1
230.2	0	0.5	7250.0	2.14		GT = 7252.1	ENDF-B-4
229.5 ± 0.04	0	0.5	*6250 ± 80			WGH = 13.05 ± 0.17	BNL325(3)
229.5	0		6180	2.0			73MOXON
229.5	0	0.5	6180	(2.0)			74MOXON
229.5	0					GT = 7250	66FARRELL
229.5 ± 1.2	0		6180 ± 160			GNO = 15.761	
229.5 ± 1.2	0						75BEER
243.23	0	0.5	780.0	2.0		GT = 782.0	JENDL-2
243.23	0	0.5	780.0	2.0		GT = 782.0	JENDL-1
243.0	0	0.5	1250.0	2.14		GT = 1252.1	ENDF-B-4
242.2 ± 0.08	0	0.5	*780 ± 40			WGH = 1.58 ± 0.08	BNL325(3)
243.23	0		780	2.0			73MOXON
243.2	0	0.5	780	(2.0)			74MOXON
242.2	0					GT = 750	66FARRELL
243.2 ± 1.3	0		780 ± 150			GNO = 1.591	
243.2 ± 1.3	0						75BEER
260.74	1	0.5	105.0	1.0		GT = 106.0	JENDL-2
260.74	1	0.5	105.0	1.0		GT = 106.0	JENDL-1
259.5	1	0.5	105.0	0.6		GT = 105.6	ENDF-B-4
259.5	■		*113				BNL325(3)
260.74	0	0.5	105	1.0			73MOXON
259.5	0		*112.5 ± 7.5				66FARRELL
273.74	1	0.5	315.0	1.0		GT = 316.0	JENDL-2
273.74	1	0.5	315.0	1.0		GT = 316.0	JENDL-1
273.1	1	0.5	315.0	0.6		GT = 315.6	ENDF-B-4
272.5	■		*333				BNL325(3)
273.74	0	0.5	315	1.0			73MOXON
272.5	0		*332.5 ± 7.5				66FARRELL
281.1	0	0.5	4800.0	2.0		GT = 4802.0	JENDL-2
281.1	0	0.5	4800.0	2.0		GT = 4802.0	JENDL-1
282.1	0	0.5	5200.0	2.14		GT = 5202.1	ENDF-B-4
280.5	0	0.5	*4800 ± 200			WGH = 9.06 ± 0.38	BNL325(3)
281.1	0		4800	2.0			73MOXON
281.1	0	0.5	4800	(2.0)			74MOXON
280.5	0					GT = 5500	66FARRELL
281.1 ± 1.6	0		4800 ± 400			GNO = 10.911	
281.1 ± 1.6	0						75BEER
287.24	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-2
287.24	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-1
286.4	0	0.5	1500.0	2.14		GT = 1502.1	ENDF-B-4
286.0	0	0.5	*1500 ± 500			WGH = 2.81 ± 0.93	BNL325(3)
287.24	0		1500	2.0			73MOXON
287.0	0	0.5	1250	(2.0)			74MOXON
286.0	0					GT = 1500	66FARRELL
(288.0)	0		(1000)			GNO = 2.950	
296.24	1	0.5	190.0	1.0		GT = 191.0	JENDL-2
296.24	1	0.5	190.0	1.0		GT = 191.0	JENDL-1
297.0	1	0.5	190.0	0.6		GT = 190.6	ENDF-B-4
297.0	■		*200				BNL325(3)
296.24	0	0.5	190	1.0			73MOXON
297.0	0		*200 ± 10				66FARRELL
300.74	1	0.5	470.0	1.0		GT = 471.0	JENDL-2
300.74	1	0.5	470.0	1.0		GT = 471.0	JENDL-1
299.5	1	0.5	470.0	0.6		GT = 470.6	ENDF-B-4
299.5	■		*500				BNL325(3)
300.74	0	0.5	470	1.0			73MOXON
299.5	0		*495 ± 25				66FARRELL
305.24	0	0.5	800.0	2.0		GT = 802.0	JENDL-2
305.24	0	0.5	800.0	2.0		GT = 802.0	JENDL-1
304.5	0	0.5	1000.0	2.14		GT = 1002.1	ENDF-B-4
304.0	0	0.5	*800			WGH = 1.45	BNL325(3)
305.24	0		800	2.0			73MOXON
305.24	0	0.5	800	(2.0)			74MOXON
304.0	0					GT = 800	66FARRELL
304.0	0					GNO = 1.531	
316.74	1	0.5	225.0	1.0		GT = 226.0	JENDL-2
316.74	1	0.5	225.0	1.0		GT = 226.0	JENDL-1
314.8	1	0.5	225.0	0.6		GT = 225.6	ENDF-B-4
315.5	■		*238				BNL325(3)
316.74	0	0.5	225	1.0			73MOXON
315.5	0		*237.5 ± 12.5				66FARRELL
320.24	1	0.5	356.0	1.0		GT = 357.0	JENDL-2
320.24	1	0.5	356.0	1.0		GT = 357.0	JENDL-1
319.5	1	0.5	356.0	0.6		GT = 356.6	ENDF-B-4

ENERGY (MEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
319.0	*		^a 375				
320.24	0	0.5	356	1.0			
319.0	0	0.5	^a 375	± 19			
324.24	1	0.5	560.0	1.0		GT = 561.0	JENDL-2
324.24	1	0.5	560.0	1.0		GT = 561.0	JENDL-1
323.4	1	0.5	560.0	0.6		GT = 560.6	ENDF-B-4
323.0	*		^a 560				BNL325(3)
324.24	0	0.5	560	1.0			73MOXON
323.0	0	0.5	^a 600	± 40			66FARRELL
328.24	0	0.5	5500.0	2.0		GT = 5502.0	JENDL-2
328.24	0	0.5	5500.0	2.0		GT = 5502.0	JENDL-1
328.0	0	0.5	5500.0	2.14		GT = 5502.1	ENDF-B-4
327.0	0	0.5	^a 5500			WCH = 9.62	BNL325(3)
328.24	0	0.5	5500	2.0			73MOXON
328.24	0	0.5	5500	(2.0)		GT = 5500	74MOXON
327.0	0					GNO = 10.186	66FARRELL
345.44	0	0.5	7500.0	2.0		GT = 7502.0	JENDL-2
345.44	0	0.5	7500.0	2.0		GT = 7502.0	JENDL-1
345.0	0	0.5	7500.0	2.14		GT = 7502.1	ENDF-B-4
344.2	0	0.5	^a 7500			WCH = 12.78	BNL325(3)
345.44	0		7500	2.0			73MOXON
345.44	0	0.5	7500	(2.0)		GT = 7500	74MOXON
344.2	0					GNO = 13.578	66FARRELL
353.24	1	0.5	267.0	1.0		GT = 268.0	JENDL-2
353.24	1	0.5	267.0	1.0		GT = 268.0	JENDL-1
352.5	1	0.5	267.0	0.6		GT = 267.6	ENDF-B-4
352.0	*		^a 279				BNL325(3)
353.24	0	0.5	267	1.0			73MOXON
352.0	0	0.5	^a 278.5	± 11.5			66FARRELL
357.44	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-2
357.44	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-1
357.7	0	0.5	2000.0	2.14		GT = 2002.1	ENDF-B-4
356.2	0	0.5	^a 2000			WCH = 3.35	BNL325(3)
357.44	0	0.5	2000	2.0			73MOXON
357.44	0	0.5	2000	(2.0)		GT = 2000	74MOXON
356.2	0					GNO = 3.567	66FARRELL
365.27	1	0.5	187.0	1.0		GT = 188.0	JENDL-2
365.27	1	0.5	187.0	1.0		GT = 188.0	JENDL-1
364.7	1	0.5	187.0	0.6		GT = 187.6	ENDF-B-4
364.0	*		^a 194				BNL325(3)
365.24	0	0.5	187	1.0			73MOXON
364.0	0	0.5	^a 193.5	± 6.5			66FARRELL
375.74	0	0.5	250.0	2.0		GT = 252.0	JENDL-2
375.74	0	0.5	250.0	2.0		GT = 252.0	JENDL-1
375.5	0	0.5	250.0	2.14		GT = 252.14	ENDF-B-4
374.5	0	0.5	^a 250			WCH = 0.41	BNL325(3)
375.74	0	0.5	250	2.0			73MOXON
375.74	0	0.5	250	(2.0)		GT = 250	74MOXON
374.5	0					GNO = 0.436	66FARRELL
383.74	0	0.5	1250.0	2.0		GT = 1252.0	JENDL-2
383.74	0	0.5	1250.0	2.0		GT = 1252.0	JENDL-1
383.5	0	0.5	1150.0	2.14		GT = 1152.1	ENDF-B-4
382.5	0	0.5	^a 1250			WCH = 2.02	BNL325(3)
383.74	0		1250	2.0			73MOXON
383.74	0	0.5	1250	(2.0)		GT = 1250	74MOXON
382.5	0					GNO = 2.161	66FARRELL
389.74	0	0.5	4500.0	2.0		GT = 4502.0	JENDL-2
389.74	0	0.5	4500.0	2.0		GT = 4502.0	JENDL-1
389.5	0	0.5	4500.0	2.14		GT = 4502.1	ENDF-B-4
388.5	0	0.5	^a 4500			WCH = 7.22	BNL325(3)
389.74	0		4500	2.0			73MOXON
389.74	0	0.5	4500	(2.0)		GT = 4500	74MOXON
388.5	0					GNO = 7.726	66FARRELL
402.44	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-2
402.44	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-1
402.2	0	0.5	1750.0	2.14		GT = 1752.1	ENDF-B-4
401.5	0	0.5	^a 1500			WCH = 2.37	BNL325(3)
402.44	0		1500	2.0			73MOXON
402.44	0	0.5	1500	(2.0)		GT = 1500	74MOXON
401.2	0					GNO = 2.540	66FARRELL
404.54	1	0.5	435.0	1.0		GT = 436.0	JENDL-2
404.5	1	0.5	392.0	1.0		GT = 393.0	JENDL-1
403.9	1	0.5	404.0	0.6		GT = 404.6	ENDF-B-4
403.3	*		^a 392				BNL325(3)
404.54	0	0.5	4035	1.0			73MOXON
403.3	0	0.5	^a 407.5	± 27.5			66FARRELL
421.54	1	0.5	800.0	1.0		GT = 801.0	JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
421-54	1	0.5	800.0	1.0		GT = 801.0	JENDL-1
420-7	1	0.5	800.0	0.6		GT = 800.6	ENDF-B-4
420-3	■		813				BNL325131
421-54	► 0	0.5	800		1.0		73MOXON
420-3	► 0		813	±13			66FARRELL
424-24	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-2
424-24	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-1
423-8	0	0.5	1500.0	2.14		GT = 1502.1	ENDF-B-4
423-0	0	0.5	1500			WGH = 2.31	BNL325131
424-24	0	0.5	1500	2.0			73MOXON
424-24	0	0.5	1500	(2.0)		GT = 1500	74MOXON
423-0	0					CNO = 2.4B3	66FARRELL
434-24	0	0.5	6500.0	2.0		GT = 6502.0	JENDL-2
434-24	0	0.5	6500.0	2.0		GT = 6502.0	JENDL-1
434-7	0	0.5	6500.0	2.14		GT = 6502.1	ENDF-B-4
435-0	0	0.5	6500			WGH = 9.88	BNL325131
434-24	0		6500	2.0			73MOXON
434-24	0	0.5	6500	(2.0)		GT = 6500	74MOXON
433-0	0					CNO = 10.650	66FARRELL
445-24	0	0.5	350.0	2.0		GT = 352.0	JENDL-2
445-24	0	0.5	350.0	2.0		GT = 352.0	JENDL-1
445-0	0	0.5	350.0	2.14		GT = 352.14	ENDF-B-4
444-0	0	0.5	350			WGH = 0.53	BNL325131
445-24	0		350	2.0			73MOXON
445-24	0	0.5	350	(2.0)		GT = 350	74MOXON
444-0	0					CNO = 0.567	66FARRELL
447-74	1	0.5	150.0	1.0		GT = 151.0	JENDL-2
447-74	1	0.5	150.0	1.0		GT = 151.0	JENDL-1
446-5	1	0.5	350.0	0.6		GT = 350.6	ENDF-B-4
447-74	► 0	0.5	(150)	1.0			73MOXON
446-5	► 0						66FARRELL
451-04	1	0.5	248.0	1.0		GT = 248.0	JENDL-2
451-04	1	0.5	248.0	1.0		GT = 248.0	JENDL-1
449-8	■		250				BNL325131
451-04	► 0	0.5	248		1.0		73MOXON
449-8	► 0		248	± 1			66FARRELL
451-24	1	0.5	231.0	1.0		GT = 232.0	JENDL-2
451-24	1	0.5	231.0	1.0		GT = 232.0	JENDL-1
451-2	1	0.5	600.0	0.6		GT = 600.6	ENDF-B-4
450-0	■		236				BNL325131
451-24	► 0	0.5	231		1.0		73MOXON
450-0	► 0		235.5	± 4.5			66FARRELL
459-24	0	0.5	500.0	2.0		GT = 502.0	JENDL-2
459-24	0	0.5	500.0	2.0		GT = 502.0	JENDL-1
458-9	0	0.5	500.0	2.14		GT = 502.14	ENDF-B-4
458-0	0	0.5	500			WGH = 0.74	BNL325131
459-24	0		500	2.0			73MOXON
459-24	0	0.5	500	(2.0)		GT = 500	74MOXON
458-0	0					CNO = 0.800	66FARRELL
463-04	1	0.5	540.0	1.0		GT = 541.0	JENDL-2
463-04	1	0.5	540.0	1.0		GT = 541.0	JENDL-1
462-52	1	0.5	540.0	0.6		GT = 540.6	ENDF-B-4
461-8	■		550				BNL325131
463-04	► 0	0.5	540		1.0		73MOXON
461-8	► 0		550	±10			66FARRELL
476-24	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-2
476-24	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-1
475-5	0	0.5	1900.0	2.14		GT = 1902.1	ENDF-B-4
475-0	0	0.5	1500			WGH = 2.18	BNL325131
476-24	0		1500	2.0			73MOXON
476-24	0	0.5	1500	(2.0)		GT = 1500	74MOXON
475-0	0					CNO = 2.363	66FARRELL
481-24	1	0.5	318.0	1.0		GT = 319.0	JENDL-2
481-24	1	0.5	318.0	1.0		GT = 319.0	JENDL-1
481-2	1	0.5	318.0	0.6		GT = 318.6	ENDF-B-4
480-0	■		324				BNL325131
481-24	► 0	0.5	318		1.0		73MOXON
480-0	► 0		339	±2			66FARRELL
489-74	0	0.5	4000.0	2.0		GT = 4002.0	JENDL-2
489-74	0	0.5	4000.0	2.0		GT = 4002.0	JENDL-1
488-5	0	0.5	4000.0	2.14		GT = 4002.1	ENDF-B-4
488-5	0	0.5	4000			WGH = 5.72	BNL325131
489-74	0		4000	2.0			73MOXON
489-74	0	0.5	4000	(2.0)		GT = 4000	74MOXON
488-5	0					CNO = 6.228	66FARRELL
494-24	1	0.5	890.0	1.0		GT = 891.0	JENDL-2
494-24	1	0.5	890.0	1.0		GT = 891.0	JENDL-1
493-5	1	0.5	890.0	0.6		GT = 890.6	ENDF-B-4

ENERGY [KEV]	L	J	NEUTRON WIDTH [EV]	GAMMA WIDTH [EV]	WHS [EV]	MISCELLANEOUS	REFERENCE
493.5	I	0.5	⁸ 934			WCI = -2.8	BNL325(3) 73MOXON 66FARRELL
494.24	> O	0.5	890	1.0			
493.5	> O	0.5	⁸ 901	± 11			
499.24	O	0.5	1500.0	2.0		GT = 1502.0	JENDL-2
499.24	O	0.5	1500.0	2.0		GT = 1502.0	JENDL-1
499.2	O	0.5	1500.0	2.14		GT = 1502.1	ENDF-B-4
498.0	O	0.5	⁸ 1500			WCH = -2.13	BNL325(3)
499.24	O		1500	2.0			73MOXON
499.24	O	0.5	1500	1 2.0 1			74MOXON
498.0	O					GT = 1500	66FARRELL
						GNO = -2.317	
509.74	O	0.5	500.0	2.0		GT = 502.0	JENDL-2
509.74	O	0.5	500.0	2.0		GT = 502.0	JENDL-1
509.1	O	0.5	1500.0	2.14		GT = 1502.1	ENDF-B-4
508.5	O	0.5	⁸ 500			WCH = -0.70	BNL325(3)
509.74	O		500	2.0			73MOXON
509.74	O	0.5	500	1 2.0 1			74MOXON
508.5	O					GT = 500	66FARRELL
						GNO = -0.766	
513.2	I	0.5	170.0	0.6		GT = 170.6	ENDF-B-4
516.74	I	0.5	140.0	1.0		GT = 141.0	JENDL-2
516.74	I	0.5	140.0	1.0		GT = 141.0	JENDL-1
515.5	I	0.5	140.0	0.6		GT = 140.6	ENDF-B-4
515.5	I		⁸ 145				BNL325(3)
516.74	> O	0.5	140	1.0			73MOXON
515.5	> O	0.5	⁸ 145	± 5			66FARRELL
523.24	I	0.5	380.0	1.0		GT = 381.0	JENDL-2
523.24	I	0.5	380.0	1.0		GT = 381.0	JENDL-1
522.45	I	0.5	380.0	0.6		GT = 380.6	ENDF-B-4
522.C	I		⁸ 390				BNL325(3)
523.24	> O	0.5	380	1.0			73MOXON
522.0	> O	0.5	⁸ 390	± 10			66FARRELL
530.24	I	1.5	925.0	1.0		GT = 926.0	JENDL-2
530.24	I	1.5	1725.0	1.0		GT = 1726.0	JENDL-1
529.0	I	1.5	925.0	0.6		GT = 925.6	ENDF-B-4
529.0	I	1.5	⁸ 1690			WCH = -4.7	BNL325(3)
530.24	> O	0.5	1725	1.0			73MOXON
529.0	> O	1.5	⁸ 894	± 31			66FARRELL
536.74	I	0.5	1600.0	1.0		GT = 1601.0	JENDL-2
536.74	I	0.5	1600.0	1.0		GT = 1601.0	JENDL-1
535.5	I	0.5	1600.0	0.6		GT = 1600.6	ENDF-B-4
535.5	I		⁸ 1390				BNL325(3)
536.74	> O	0.5	1600	1.0			73MOXON
535.5	> O	0.5	⁸ 1630	± 30			66FARRELL
540.24	O	0.5	2000.0	2.0		GT = 2002.0	JENDL-2
540.24	O	0.5	2000.0	2.0		GT = 2002.0	JENDL-1
539.0	O	0.5	2000.0	2.14		GT = 2002.1	ENDF-B-4
539.0	O	0.5	⁸ 2000			WCH = -2.72	BNL325(3)
540.24	O		2000	2.0			73MOXON
540.24	O	0.5	2000	1 2.0 1			74MOXON
539.0	O					GT = 2000	66FARRELL
						GNO = -2.989	
543.0	I	0.5	150.0	0.6		GT = 150.6	ENDF-B-4
544.4	I	0.5	150.0	0.6		GT = 150.6	ENDF-B-4
555.24	I	0.5	655.0	1.0		GT = 656.0	JENDL-2
555.24	I	0.5	655.0	1.0		GT = 656.0	JENDL-1
554.0	I	0.5	655.0	0.6		GT = 655.6	ENDF-B-4
554.0	I		⁸ 675				BNL325(3)
555.24	> O	0.5	655	1.0			73MOXON
554.0	> O	0.5	⁸ 668	± 13			66FARRELL
569.74	I	0.5	825.0	1.0		GT = 826.0	JENDL-2
569.74	I	0.5	825.0	1.0		GT = 826.0	JENDL-1
568.2	I	0.5	500.0	0.6		GT = 500.6	ENDF-B-4
568.5	I		⁸ 643				BNL325(3)
569.74	> O	0.5	825	1.0			73MOXON
568.5	> O	0.5	⁸ 643	± 18			66FARRELL
573.34	O	0.5	4000.0	2.0		GT = 4002.0	JENDL-2
573.34	O	0.5	4000.0	2.0		GT = 4002.0	JENDL-1
571.8	O	0.5	3300.0	2.14		GT = 3302.1	ENDF-B-4
571.8	O	0.5	⁸ 4000			WCH = -5.29	BNL325(3)
573.34	O		4000	2.0			73MOXON
573.34	O	0.5	4000	1 2.0 1			74MOXON
571.8	O					GT = 4000	66FARRELL
						GNO = -5.836	
582.24	O	0.5	500.0	2.0		GT = 502.0	JENDL-2
582.24	O	0.5	500.0	2.0		GT = 502.0	JENDL-1
581.6	O	0.5	1100.0	2.14		GT = 1102.1	ENDF-B-4
581.0	O	0.5	⁸ 600			WCH = -0.66	BNL325(3)
582.24	O		500	2.0			73MOXON
582.24	O	0.5	500	1 2.0 1			74MOXON
581.0	O					GT = 500	66FARRELL
						GNO = -0.725	

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
584.74	0	0.5	10000.0	2.0		GT = 10002.0	JENDL-2
584.74	0	0.5	10000.0	2.0		GT = 10002.0	JENDL-1
583.9	0	0.5	10000.0	2.14		GT = 10002.0	ENDF-B-4
583.5	0	0.5	10000			WGH = 13.09	BNL325(3)
584.74	0		10000	2.0			73MOXON
584.74	0	0.5	10000	2.0			74MOXON
583.5	0		10000	2.0			66FARRELL
591.74	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-2
591.74	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-1
593.5	0	0.5	2000.0	2.14		GT = 2002.1	ENDF-B-4
593.5	0	0.5	2000			WGH = 2.60	BNL325(3)
591.74	0		2000	2.0			73MOXON
591.74	0	0.5	2000	2.0			74MOXON
593.5	0		2000	2.0			66FARRELL
630.74	1	0.5	810.0	1.0		GT = 811.0	JENDL-2
600.74	1	0.5	810.0	1.0		GT = 811.0	JENDL-1
599.5	1	0.5	375.0	0.6		GT = 375.6	ENDF-B-4
599.5	1	0.5	375			WGI = 2.2	BNL325(3)
600.74	0	0.5	810	1.0			73MOXON
599.5	0	0.5	825	1.0			66FARRELL
605.5	1	0.5	300.0	0.6		GT = 300.6	ENDF-B-4
611.0	0	0.5	7000.0	2.14		GT = 7002.1	ENDF-B-4
621.5	1	0.5	300.0	0.6		GT = 300.6	ENDF-B-4
628.5	0	0.5	1300.0	2.14		GT = 1302.1	ENDF-B-4
631.0	1	1.5	500.0	0.6		GT = 500.6	ENDF-B-4
635.5	1	1.5	600.0	0.6		GT = 600.6	ENDF-B-4
640.3	0	0.5	400.0	2.14		GT = 402.14	ENDF-B-4
642.5	1	0.5	300.0	0.6		GT = 300.6	ENDF-B-4
653.0	0	0.5	9000.0	2.14		GT = 9002.1	ENDF-B-4

* A and B denote $g\Gamma_n$ and $g\Gamma_\gamma$, respectively

** WW5 = $g\Gamma_n \Gamma_\gamma / \Gamma$ (eV), GT = Γ (eV)

WGH = $g\Gamma_n^{(0)}$ (eV), GNO = $\Gamma_n^{(0)}$ (eV)

WGI = $g\Gamma_n^{(1)}$ (eV)

References

- 66Farrell : Ref.(21)
- 69Hockenbury: Ref.(23)
- 71Garg : Ref.(49)
- 73Moxon : Ref.(51) (Evaluation)
- 74Moxon : Ref.(3) (Evaluation)
- 75Beer : Ref.(26)

Table 10 Resonance parameters of ^{64}Ni

ENERGY (KEV)	L	J.	NEUTRON WIDTH *	GAMMA WIDTH (EV)	HW5 ** (EV)	MISCELLANEOUS **	REFERENCE
9.52	I	1.5	6.9	(1.0)	1.73 ± 0.2		JENDL-2
9.52	I	1.5	6.18	1.0	1.7 ± 0.2	GT = 7.18	ENDF-B-4 BNL325(3) 73MOXON 74MOXON 69HOCKENBURY
9.52	> 0	1.5	6.41	1.0	1.73 ± 0.2		
9.52	I	1.5	6.9	(1.0)	1.73 ± 0.2	CCS = 473 ± 70	
9.52					1.73 ± 0.2		
14.3	0	0.5	2900.0	1.9		GT = 2901.9	JENDL-2
14.3	0	0.5	2900.0	1.9		GT = 2901.9	JENDL-1
13.8	0	0.5	800.0	2.14		GT = 802.14	ENDF-B-4 BNL325(3) 73MOXON 74MOXON 66FARRELL
14.3 ± 0.2	0	0.5	2900	± 500	0.76 ± 0.15	WGH = 24.25 ± 4.18	
14.3	0		2900		2.0		
14.3	0	0.5	2900		1.9	GT = 3000 GND = 25.7	
13.8	0		2900				
14.3	0		2900	± 500	1.9 ± 0.4		
14.8	I	0.5	0.316	1.0		GT = 1.316	JENDL-2
14.8	I	0.5	0.316	1.0		GT = 1.316	JENDL-1
14.8 ± 0.1	I	0.5	0.32	(1.0)	0.24 ± 0.05		74MOXON 75BEER
14.8 ± 0.1	> 0				0.24 ± 0.05		
25.8 ± 0.1							BNL325(3) 69HOCKENBURY 72HOCKENBURY
26.0							
25.8							
31.85	I	0.5	3.0	1.0		GT = 4.0	JENDL-2
31.85	I	0.5	3.0	1.0		GT = 4.0	JENDL-1
31.85 ± 0.15	I	0.5	3.0	(1.0)	0.75 ± 0.11		74MOXON 72HOCKENBURY 75BEER
31.85 ± 0.15	> 0				0.75 ± 0.11		
33.82	0	0.5	8900.0	2.9		GT = 8902.9	JENDL-2
33.82	0	0.5	8900.0	2.9		GT = 8902.9	JENDL-1
33.2	0	0.5	950.0	2.14		GT = 952.14	ENDF-B-4 BNL325(3) 73MOXON 74MOXON 66FARRELL
33.81 ± 0.04	0	0.5	8900	± 500		WGH = 48.40 ± 2.72	
33.81	0		8900		2.0		
33.82	0	0.5	8900		2.9		
33.2	0		8900			GT = 9500 GND = 52.5	
33.5 ± 1.5							
33.82 ± 0.10	0		8900	± 50	2.9 ± 0.6		
39.2							69HOCKENBURY 72HOCKENBURY
39.2							
46.1							69HOCKENBURY 72HOCKENBURY
45.8							
53.9							69HOCKENBURY
60.0							72HOCKENBURY
62.8	I	0.5	5.7	1.0		GT = 6.7	JENDL-2
62.8	I	0.5	5.7	1.0		GT = 6.7	JENDL-1
64.0							69HOCKENBURY 72HOCKENBURY 75BEER
62.4							
62.8 ± 0.4	> 0				0.85 ± 0.18		
83.4							69HOCKENBURY 72HOCKENBURY
82.8							
105.5	I	0.5	110.0	1.0		GT = 111.0	JENDL-2
105.52	I	0.5	110.0	1.0		GT = 111.0	JENDL-1
105.0	I	0.5	115.0	0.5		GT = 115.6	ENDF-B-4 BNL325(3) 73MOXON 66FARRELL 75BEER
105.52 ± 0.08	*		110	± 30			
105.52	> 0	0.5	110		1.0		
105.0	> 0		122.5	± 7.5			
106.5 ± 0.4	> 0		110	± 50			
129.3	0	0.5	1340.0	2.0		GT = 1342.0	JENDL-2
129.32	0	0.5	1340.0	2.0		GT = 1342.0	JENDL-1
128.8	0	0.5	1700.0	2.14		GT = 1702.1	ENDF-B-4 BNL325(3) 73MOXON 74MOXON 66FARRELL
129.32 ± 0.03	0	0.5	1400	± 50		WGH = 3.89 ± 0.14	
129.32	0		1340		2.0		
129.3	0	0.5	1340	(2.0)			
128.8	0		1340			GT = 1700 GND = 4.847	
129.3 ± 0.5	0		1340	± 70			
142.0	I	0.5	170.0	1.0		GT = 171.0	JENDL-2
141.97	I	0.5	170.0	1.0		GT = 171.0	JENDL-1
141.5	I	0.5	140.0	0.6		GT = 140.6	ENDF-B-4 BNL325(3) 73MOXON 66FARRELL 75BEER
141.5 ± 0.1	I		170	± 20			
141.97	> 0	0.5	170		1.0		
141.5	> 0		150	± 10			
142.0 ± 0.6	> 0		170	± 70			
148.8	0	0.5	80.0	2.0		GT = 82.0	JENDL-2
148.8	0	0.5	80.0	2.0		GT = 82.0	JENDL-1
148.8 ± 0.1	0	0.5	80	± 20		WGH = 0.21 ± 0.05	
148.8	0		80		2.0		
148.8 ± 0.7	0	0.5	80	± 70	(2.0)		
148.8 ± 0.7	0		80	± 70			
195.0	0	0.5	3900.0	2.0		GT = 3902.0	JENDL-2

JAERI-M 85-101

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	MWS (EV)	MISCELLANEOUS	REFERENCE
154.96	0	0.5	3890.0	2.0		GT = 3892.0	JENDL-1
155.1	0	0.5	5000.0	2.14		GT = 5002.1	ENDF-B-4
155.0 ± 0.1	0	0.5	^a 3950 ± 100			WCH = 10.04 ± 0.25	BNL325(3) 73MOXON 74MOXON 66FARRELL
154.96	0		3890	2.0			
155.0	0	0.5	3900	2.0		GT = 5000	
154.5	0					CNO = 13.076	
155.0 ± 0.7	0		3900 ± 100				75BEER
163.2	0	0.5	140.0	2.0		GT = 142.0	JENDL-2
163.2	0	0.5	140.0	2.0		GT = 142.0	JENDL-1
163.0	0	0.5	300.0	2.14		GT = 302.14	ENDF-B-4
163.2 ± 0.1	0	0.5	^a 160 ± 20			WCH = 0.40 ± 0.05	BNL325(3) 73MOXON 74MOXON 66FARRELL
163.2	0		140	2.0			
163.2	0	0.5	140	2.0		GT = 300	
163.0	0					CNO = 0.765	
163.2 ± 0.8	0		140 ± 80				75BEER
177.7	0	0.5	470.0	2.0		GT = 472.0	JENDL-2
177.7	0	0.5	470.0	2.0		GT = 472.0	JENDL-1
177.5	0	0.5	500.0	2.14		GT = 502.14	ENDF-B-4
177.7 ± 0.1	0	0.5	^a 470 ± 30			WCH = 1.12 ± 0.07	BNL325(3) 73MOXON 74MOXON 66FARRELL
177.7	0		470	2.0			
177.7	0	0.5	470	2.0		GT = 500	
177.5	0					CNO = 1.225	
177.7 ± 0.8	0		470 ± 90				75BEER
191.5	1	0.5	160.0	1.0		GT = 161.0	JENDL-2
191.5	1	0.5	160.0	1.0		GT = 161.0	JENDL-1
190.0	1	0.5	105.0	0.6		GT = 105.6	ENDF-B-4
191.0 ± 0.2	1	1	^a 140 ± 30				BNL325(3) 73MOXON 66FARRELL
191.5	0	0.5	160	1.0			
191.0	0	0.5	^a 105.5 ± 4.5				75BEER
191.5 ± 0.9	0	0	^a 160 ± 110				
205.3	0	0.5	60.0	2.0		GT = 62.0	JENDL-2
205.3	0	0.5	60.0	2.0		GT = 62.0	JENDL-1
205.3 ± 0.2	0	0.5	^a 60 ± 20			WCH = 0.13 ± 0.04	BNL325(3) 73MOXON 74MOXON 75BEER
205.3	0		60	2.0			
205.3 ± 1.1	0	0.5	60	2.0			
205.3 ± 1.1	0		(60)				
214.7	1	0.5	80.0	1.0		GT = 81.0	JENDL-2
214.7	1	0.5	80.0	1.0		GT = 81.0	JENDL-1
213.7	1	0.5	150.0	0.6		GT = 150.6	ENDF-B-4
214.7 ± 0.3	1	0.5	^a 90 ± 20				BNL325(3) 73MOXON 66FARRELL
214.7	0	0.5	80	1.0			
213.7	0	0.5	^a 155 ± 5				
214.7 ± 1.1	0	0	^a 80 ;				75BEER
219.8	0	0.5	30.0	2.0		GT = 32.0	JENDL-2
219.8	0	0.5	30.0	2.0		GT = 32.0	JENDL-1
221.0	1	0.5	400.0	0.6		GT = 400.6	ENDF-B-4
219.8 ± 0.1	0	0.5	^a 30 ± 20			WCH = 0.064 ± 0.043	BNL325(3) 73MOXON 74MOXON 75BEER
219.8 ± 1.1	0	0.5	30	2.0			
219.8 ± 1.1	0	0.5	(30)	2.0			
226.9	0	0.5	120.0	2.0		GT = 122.0	JENDL-2
226.9	0	0.5	120.0	2.0		GT = 122.0	JENDL-1
226.9 ± 0.3	0	0.5	^a 120 ± 30			WCH = 0.25 ± 0.06	BNL325(3) 73MOXON 74MOXON 75BEER
226.9	0		120	2.0			
226.9 ± 1.2	0	0.5	120	2.0			
226.9 ± 1.2	0		(120)				
231.9	0	0.5	3770.0	2.0		GT = 3772.0	JENDL-2
231.95	0	0.5	3770.0	2.0		GT = 3772.0	JENDL-1
231.0	0	0.5	4000.0	2.14		GT = 4002.1	ENDF-B-4
231.95 ± 0.04	0	0.5	^a 3770 ± 90			WCH = 7.63 ± 0.19	BNL325(3) 73MOXON 74MOXON 66FARRELL
231.95	0		3770	2.0			
231.9	0	0.5	3770	2.0		GT = 4000	
231.0	0					CNO = 8.670	
231.9 ± 1.2	0		3770 ± 160				75BEER
237.9	1	0.5	320.0	1.0		GT = 321.0	JENDL-2
237.9	1	0.5	320.0	1.0		GT = 321.0	JENDL-1
236.2	1	0.5	395.0	0.6		GT = 395.6	ENDF-B-4
237.9 ± 0.1	1	0.5	^a 320 ± 40				BNL325(3) 73MOXON 74MOXON 66FARRELL
237.9	0	0.5	320	1.0			
235.7	0	0.5	^a 402.5 ± 7.5				
237.9 ± 1.3	0	0	^a 320 ± 130				
238.7	1	1.5	400.0	0.6		GT = 400.6	ENDF-B-4
255.7	1	0.5	170.0	1.0		GT = 171.0	JENDL-2
255.7	1	0.5	170.0	1.0		GT = 171.0	JENDL-1
257.9	1	0.5	570.0	0.6		GT = 570.6	ENDF-B-4
255.7 ± 0.3	1	0.5	^a 170 ± 40				BNL325(3) 73MOXON 66FARRELL
255.7	0	0.5	170	1.0			
254.0	0	0	^a 589 ± 19				
255.7 ± 1.4	0	0	^a 170 ± 150				
255.7	0	0.5	2200.0	2.0		GT = 2202.0	JENDL-2

JAERI-M 85-101

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WNS (EV)	MISCELLANEOUS	REFERENCE
269.58	0	0.5	2210.0	2.0		GT = 2212.0	JENDL-1
272.2	0	0.5	2200.0	2.14		GT = 2202.1	ENDF-B-4
269.7 ± 0.1	0	0.5	2210 ± 90			WGN = 4.26 ± 0.17	BNL325131
269.58	0		2210	2.0			73MOXON
269.7	0	0.5	2200	(2.0)			74MOXON
268.0	0					GT = 3000	66FARRELL
269.7 ± 1.5	0		2200	±200		GNO = 6.076	75BEER
275.24	1	0.5	310.0	1.0		GT = 311.0	JENDL-2
275.24	1	0.5	310.0	1.0		GT = 311.0	JENDL-1
278.2	1	0.5	310.0	0.6		GT = 310.6	ENDF-B-4
275.24	0	0.5	310	1.0			73MOXON
274.0	0		320	±10			66FARRELL
283.5	0	0.5	350.0	2.0		GT = 352.0	JENDL-2
283.5	0	0.5	350.0	2.0		GT = 352.0	JENDL-1
283.5 ± 0.4	0	0.5	350 ± 70			WGN = 0.66 ± 0.13	BNL325131
283.5	0		350	2.0			73MOXON
283.5 ± 1.6	0	0.5	350 ± 190	(2.0)			74MOXON
283.5 ± 1.6	0		350 ± 190				75BEER
290.24	1	0.5	105.0	1.0		GT = 106.0	JENDL-2
290.24	1	0.5	105.0	1.0		GT = 106.0	JENDL-1
288.4	1	0.5	105.0	0.6		GT = 105.6	ENDF-B-4
290.24	0	0.5	105	1.0			73MOXON
289.0	0		107.5	± 2.5			66FARRELL
299.24	0	0.5	1000.0	2.0		GT = 1002.0	JENDL-2
299.24	0	0.5	1000.0	2.0		GT = 1002.0	JENDL-1
298.0	0	0.5	1000.0	2.14		GT = 1002.1	ENDF-B-4
298.0 ± 2.5	0	0.5	1000			WGN = 1.83	BNL325131
299.24	0		1000	2.0			73MOXON
299.24	0	0.5	1000	(2.0)		GT = 1000	74MOXON
298.0	0					GNO = 1.930	66FARRELL
309.74	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-2
309.74	0	0.5	1500.0	2.0		GT = 1502.0	JENDL-1
309.1	0	0.5	1500.0	2.14		GT = 1502.1	ENDF-B-4
308.5 ± 2.5	0	0.5	1500			WGN = 2.70	BNL325131
309.74	0		1500	2.0			73MOXON
309.74	0	0.5	1500	(2.0)		GT = 1500	74MOXON
308.5	0					GNO = 2.851	66FARRELL
321.24	1	0.5	50.0	1.0		GT = 51.0	JENDL-2
321.24	1	0.5	50.0	1.0		GT = 51.0	JENDL-1
320.0	1	1.5	100.0	0.6		GT = 100.6	ENDF-B-4
321.24	0	0.5	(50)	1.0			73MOXON
320.0	0						66FARRELL
327.74	1	0.5	585.0	1.0		GT = 586.0	JENDL-2
327.74	1	0.5	585.0	1.0		GT = 586.0	JENDL-1
326.5	1	0.5	585.0	0.6		GT = 585.6	ENDF-B-4
327.0 ± 2.5	0		597				BNL325131
327.74	0	0.5	585	1.0			73MOXON
326.5	0		596	±11			66FARRELL
334.24	0	0.5	250.0	2.0		GT = 252.0	JENDL-2
333.0	0	0.5	250.0	2.14		GT = 252.14	ENDF-B-4
333.0 ± 2.5	0	0.5	250			WGN = 0.43	BNL325131
334.24	0		250	2.0			73MOXON
334.24	0	0.5	250	(2.0)		GT = 250	74MOXON
333.0	0					GNO = 0.459	66FARRELL
335.24	1	0.5	50.0	1.0		GT = 51.0	JENDL-2
335.24	1	0.5	50.0	1.0		GT = 51.0	JENDL-1
334.0	1	1.5	100.0	0.6		GT = 100.6	ENDF-B-4
335.24	0	0.5	(50)	1.0			73MOXON
334.0	0						66FARRELL
341.44	0	0.5	500.0	2.0		GT = 502.0	JENDL-2
341.44	0	0.5	500.0	2.0		GT = 502.0	JENDL-1
340.2	0	0.5	500.0	2.14		GT = 502.14	ENDF-B-4
340.2	0	0.5	500			WGN = 0.86	BNL325131
341.44	0		500	2.0			73MOXON
341.44	0	0.5	500	(2.0)		GT = 500	74MOXON
340.2	0					GNO = 0.910	66FARRELL
353.24	1	0.5	200.0	1.0		GT = 201.0	JENDL-2
353.24	1	0.5	200.0	1.0		GT = 201.0	JENDL-1
352.0	1	1.5	100.0	0.6		GT = 100.6	ENDF-B-4
353.24	0	0.5	(200)	1.0			73MOXON
352.0	0						66FARRELL
360.54	1	0.5	715.0	1.0		GT = 716.0	JENDL-2
360.54	1	0.5	715.0	1.0		GT = 716.0	JENDL-1
360.3	1	0.5	725.0	0.6		GT = 725.6	ENDF-B-4
360.3	0		728				BNL325131
360.54	0	0.5	715	1.0			73MOXON
360.3	0		728	±13			66FARRELL
361.74	1	1.5	950.0	1.0		GT = 951.0	JENDL-2

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WHS (EV)	MISCELLANEOUS	REFERENCE
366.24	1	0.5	1870.0	1.0		GT = 1871.0	JENDL-1
365.0	1	0.5	960.0	0.6		GT = 960.6	ENDF-B-4
365.0	1	1.5	1857			WGI = 7.6	BNL325(3)
366.24	> 0	0.5	1870	1.0			73MOXON
365.0	> 0	1.5	* 943	± 7			66FARRELL
369.24	1	0.5	200.0	1.0		GT = 201.0	JENDL-2
369.24	1	0.5	200.0	1.0		GT = 201.0	JENDL-1
367.6	1	1.5	100.0	0.6		GT = 100.6	ENDF-B-4
369.24	> 0	0.5	(200 ± 1)	1.0			73MOXON
368.0	> 0						66FARRELL
372.74	1	1.5	695.0	1.0		GT = 696.0	JENDL-2
372.74	1	0.5	1365.0	1.0		GT = 1366.0	JENDL-1
371.0	1	1.5	* 465.0	0.6		GT = 465.6	ENDF-B-4
371.5	1	1.5	1318			WGI = 5.3	BNL325(3)
372.74	> 0	0.5	* 365	1.0			73MOXON
371.5	> 0	1.5	* 689	± 6			66FARRELL
377.24	1	0.5	270.0	1.0		GT = 271.0	JENDL-2
377.24	1	0.5	270.0	1.0		GT = 271.0	JENDL-1
376.35	1	0.5	270.0	0.6		GT = 270.6	ENDF-B-4
377.24	> 0	0.5	270	1.0			73MOXON
376.0	> 0		* 275	± 5			66FARRELL
380.15	1	0.5	150.0	0.6		GT = 150.6	ENDF-B-4
384.24	1	1.5	875.0	1.0		GT = 876.0	JENDL-2
384.24	1	0.5	1730.0	1.0		GT = 1731.0	JENDL-1
383.0	1	1.5	600.0	0.6		GT = 600.6	ENDF-B-4
383.0	1	1.5	* 1597				BNL325(3)
384.24	> 0	0.5	1730	1.0			73MOXON
383.0	> 0	1.5	* 870	± 5			66FARRELL
390.24	0	0.5	6000.0	2.0		GT = 6002.0	JENDL-2
390.24	0	0.5	6000.0	2.0		GT = 6002.0	JENDL-1
388.4	0	0.5	7200.0	2.14		GT = 7202.1	ENDF-B-4
389.0	0	0.5	* 6000			WGI = 9.52	BNL325(3)
390.24	0		6000	2.0			73MOXON
390.24	0	0.5	6000	(2.0)		GT = 6000	74MOXON
389.0	0					GNO = 10.296	66FARRELL
393.74	1	0.5	230.0	1.0		GT = 231.0	JENDL-2
393.74	1	0.5	230.0	1.0		GT = 231.0	JENDL-1
392.75	1	0.5	130.0	0.6		GT = 130.6	ENDF-B-4
392.5	*		* 235				BNL325(3)
393.74	> 0	0.5	230	1.0			73MOXON
392.5	> 0	0.5	* 235	± 5			66FARRELL
396.74	1	0.5	810.0	1.0		GT = 811.0	JENDL-2
396.74	1	0.5	810.0	1.0		GT = 811.0	JENDL-1
395.5	1	0.5	810.0	0.6		GT = 810.6	ENDF-B-4
395.5	*		* 815				BNL325(3)
396.74	> 0	0.5	810	1.0			73MOXON
395.5	> 0	0.5	* 815	± 5			66FARRELL
408.24	1	1.5	1010.0	1.0		GT = 1011.0	JENDL-2
408.24	1	0.5	2030.0	1.0		GT = 2031.0	JENDL-1
406.6	1	1.5	1020.0	0.6		GT = 1020.6	ENDF-B-4
407.0	1	1.5	* 2020			WGI = 7.4	BNL325(3)
408.24	> 0	0.5	2030	1.0			73MOXON
407.0	> 0	1.5	* 1013	± 3			66FARRELL
415.24	1	0.5	750.0	1.0		GT = 751.0	JENDL-2
415.24	1	0.5	750.0	1.0		GT = 751.0	JENDL-1
413.6	1	0.5	300.0	0.6		GT = 300.6	ENDF-B-4
414.0	*		* 759				BNL325(3)
415.24	> 0	0.5	750	1.0			73MOXON
414.0	> 0	0.5	* 759	± 9			66FARRELL
422.04	0	0.5	8000.0	2.0		GT = 8002.0	JENDL-2
422.04	0	0.5	8000.0	2.0		GT = 8002.0	JENDL-1
420.8	0	0.5	8000.0	2.14		GT = 8002.1	ENDF-B-4
420.8	0	0.5	* 8000			WGI = 12.33	BNL325(3)
422.04	0		8000	2.0			73MOXON
422.04	0	0.5	8000	(2.0)		GT = 8000	74MOXON
420.8	0					GNO = 13.270	66FARRELL
445.3	1	0.5	800.0	0.6		GT = 800.6	ENDF-B-4
456.74	1	0.5	470.0	1.0		GT = 471.0	JENDL-2
456.74	1	0.5	470.0	1.0		GT = 471.0	JENDL-1
455.5	1	0.5	800.0	0.6		GT = 800.6	ENDF-B-4
455.5	1	0.5	* 560				BNL325(3)
456.74	> 0	0.5	470	1.0			73MOXON
455.5	> 0	0.5	* 465	± 5			66FARRELL
460.74	1	1.5	580.0	1.0		GT = 581.0	JENDL-2
460.74	1	0.5	1160.0	1.0		GT = 1161.0	JENDL-1
459.5	1	1.5	750.0	0.6		GT = 750.6	ENDF-B-4
459.5	1	1.5	* 1100				BNL325(3)
460.74	> 0	0.5	1160	1.0			73MOXON
459.5	> 0	1.5	* 580	± 0			66FARRELL
467.74	1	0.5	985.0	1.0		GT = 986.0	JENDL-2

JAERI-M 85-101

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WNS (EV)	MISCELLANEOUS	REFERENCE
467.74	I	0.5	985.0	1.0		GT = 986.0	JENDL-1
466.5	I	0.5	985.0	0.6		GT = 985.6	ENDF-B-4
466.5	I	0.5	995				BNL325(3)
467.74	I	0.5	985	1.0			73MOXON
466.5	I	0.5	993 ± 8				66FARRELL
471.24	I	0.5	530.0	1.0		GT = 531.0	JENDL-2
471.24	I	0.5	530.0	1.0		GT = 531.0	JENDL-1
470.0	I	0.5	530.0	0.6		GT = 530.6	ENDF-B-4
470.0	I	0.5	535				BNL325(3)
471.24	I	0.5	530	1.0			73MOXON
470.0	I	0.5	535 ± 5				66FARRELL
472.0	I	0.5	130.0	0.6		GT = 130.6	ENDF-B-4
473.1	I	0.5	110.0	0.6		GT = 110.6	ENDF-B-4
480.24	I	0.5	1130.0	1.0		GT = 1131.0	JENDL-2
480.24	I	0.5	1130.0	1.0		GT = 1131.0	JENDL-1
479.0	I	0.5	1130.0	0.6		GT = 1130.6	ENDF-B-4
479.0	I	0.5	1090				BNL325(3)
480.24	I	0.5	1130	1.0			73MOXON
479.0	I	0.5	1135 ± 135				66FARRELL
484.24	O	0.5	5000.0	2.0		GT = 5002.0	JENDL-2
484.24	O	0.5	5000.0	2.0		GT = 5002.0	JENDL-1
483.0	O	0.5	5000.0	2.14		GT = 5002.1	ENDF-B-4
483.0	O	0.5	5000			WGN = 7.19	BNL325(3)
484.24	O	0.5	5000	2.0			73MOXON
484.24	O	0.5	5000	2.0		GT = 5000	74MOXON
483.0	O	0.5	5000			GNO = 7.822	66FARRELL
489.04	I	0.5	430.0	1.0		GT = 431.0	JENDL-2
489.04	I	0.5	430.0	1.0		GT = 431.0	JENDL-1
487.8	I	0.5	430.0	0.6		GT = 430.6	ENDF-B-4
489.04	I	0.5	430	1.0			73MOXON
487.8	I	0.5	435 ± 5				66FARRELL
500.74	I	0.5	530.0	1.0		GT = 531.0	JENDL-2
500.74	I	0.5	530.0	1.0		GT = 531.0	JENDL-1
499.5	I	0.5	530.0	0.6		GT = 530.6	ENDF-B-4
499.5	I	0.5	535				BNL325(3)
500.74	I	0.5	530	1.0			73MOXON
499.5	I	0.5	535 ± 5				66FARRELL
504.24	I	0.5	760.0	1.0		GT = 761.0	JENDL-2
504.24	I	0.5	760.0	1.0		GT = 761.0	JENDL-1
503.0	I	0.5	760.0	0.6		GT = 760.6	ENDF-B-4
503.0	I	0.5	766				BNL325(3)
504.24	I	0.5	760	1.0			73MOXON
503.0	I	0.5	766 ± 6				66FARRELL
512.9	I	0.5	200.0	0.6		GT = 200.6	ENDF-B-4
514.8	I	0.5	100.0	0.6		GT = 100.6	ENDF-B-4
520.24	I	0.5	475.0	1.0		GT = 476.0	JENDL-2
520.24	I	0.5	475.0	1.0		GT = 476.0	JENDL-1
519.0	I	0.5	700.0	0.6		GT = 700.6	ENDF-B-4
519.0	I	0.5	477				BNL325(3)
520.24	I	0.5	475	1.0			73MOXON
519.0	I	0.5	478 ± 3				66FARRELL
524.24	O	0.5	1000.0	2.0		GT = 1002.0	JENDL-2
524.24	O	0.5	1000.0	2.0		GT = 1002.0	JENDL-1
522.9	O	0.5	2100.0	2.14		GT = 2102.1	ENDF-B-4
523.0	O	0.5	1900			WGN = 1.38	BNL325(3)
524.24	O	0.5	1000	2.0			73MOXON
524.24	O	0.5	1000	2.0		GT = 1000	74MOXON
523.0	O	0.5	1000			GNO = 1.513	66FARRELL
530.54	O	0.5	750.0	2.0		GT = 752.0	JENDL-2
530.54	O	0.5	750.0	2.0		GT = 752.0	JENDL-1
529.3	O	0.5	2300.0	2.14		GT = 2302.1	ENDF-B-4
530.54	O	0.5	750	2.0			73MOXON
530.54	O	0.5	750	2.0			74MOXON
529.3	O	0.5	750			GNO = 1.129	66FARRELL
537.74	O	0.5	10000.0	2.0		GT = 10002.0	JENDL-2
537.74	O	0.5	10000.0	2.0		GT = 10002.0	JENDL-1
536.5	O	0.5	10000.0	2.14		GT = 10002.0	ENDF-B-4
536.5	O	0.5	10000			WGN = 13.65	BNL325(3)
537.74	O	0.5	10000	2.0			73MOXON
537.74	O	0.5	10000	2.0		GT = 10000	74MOXON
536.5	O	0.5	10000			GNO = 14.976	66FARRELL
542.74	I	1.5	870.0	1.0		GT = 871.0	JENDL-2
542.74	I	0.5	1700.0	1.0		GT = 1701.0	JENDL-1
541.5	I	0.5	1700.0	0.6		GT = 1700.6	ENDF-B-4
541.5	I	1.5	1670			WGN = 4.5	BNL325(3)
542.74	I	0.5	1700	1.0			73MOXON
541.5	I	1.5	1660 ± 10				66FARRELL

ENERGY (KEV)	L	J	NEUTRON WIDTH (EV)	GAMMA WIDTH (EV)	WW5 (EV)	MISCELLANEOUS	REFERENCE
553.24	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-2
553.24	0	0.5	2000.0	2.0		GT = 2002.0	JENDL-1
552.0	0	0.5	2000.0	2.14		GT = 2002.1	ENDF-B-4
552.0	0	0.5	2000			WGH = 2.69	BNL325(3)
553.24	0		2000		2.0		73MOXON
553.24	0	0.5	2000		(2.0)		74MOXON
552.0	0					GT = 2000	66FARRELL
						GNO = 2.960	
566.24	1	0.5	890.0	1.0		GT = 891.0	JENDL-2
566.24	1	0.5	890.0	1.0		GT = 891.0	JENDL-1
565.0	1	0.5	890.0	0.6		GT = 890.6	ENDF-B-4
565.0	*		890				BNL325(3)
566.24	0	0.5	890		1.0		73MOXON
565.0	0		8901	#11			66FARRELL
577.24	0	0.5	4000.0	2.0		GT = 4002.0	JENDL-2
577.24	0	0.5	4000.0	2.0		GT = 4002.0	JENDL-1
576.9	0	0.5	3300.0	2.14		GT = 3802.1	ENDF-B-4
576.0	0	0.5	4000			WGH = 5.27	BNL325(3)
577.24	0		4000		2.0		73MOXON
577.24	0	0.5	4000		(2.0)		74MOXON
576.0	0					GT = 4000	66FARRELL
						GNO = 5.819	
584.24	0	0.5	300.0	2.0		GT = 302.0	JENDL-2
584.24	0	0.5	300.0	2.0		GT = 302.0	JENDL-1
583.0	0	0.5	300.0	2.14		GT = 302.14	ENDF-B-4
583.0	0	0.5	300			WGH = 0.39	BNL325(3)
584.24	0		300		2.0		73MOXON
584.24	0	0.5	300		(2.0)		74MOXON
583.0	0					GT = 300	66FARRELL
						GNO = 0.434	
603.8	0	0.5	11500.0	2.14		GT = 11502.0	ENDF-B-4
620.4	0	0.5	9400.0	2.14		GT = 9402.1	ENDF-B-4
528.8	0	0.5	3600.0	2.14		GT = 3602.1	ENDF-B-4
633.3	1	1.5	800.0	0.6		GT = 800.6	ENDF-B-4
538.2	1	1.5	800.0	0.6		GT = 800.6	ENDF-B-4

* A denotes $g\Gamma_n$

$$** WW5 = g\Gamma_n \Gamma_\gamma / \Gamma \text{ (eV)}, \quad GT = \Gamma \text{ (eV)}$$

$$GGS = \sigma_0 \Gamma_\gamma \text{ (b·eV)}, \quad WGH = g\Gamma_n^{(0)} \text{ (eV)}$$

$$GNO = \Gamma_n^{(0)} \text{ (eV)}, \quad WGI = g\Gamma_n^{(1)} \text{ (eV)}$$

References

66Farrell : Ref.(21)

69Hockenbury: Ref.(23)

72Hockenbury: Ref.(54)

73Moxon : Ref.(51) (Evaluation)

74Moxon : Ref.(3) (Evaluation)

75Beer : Ref.(26)

Table 11 Capture resonance integrals with
cut-off energy of 0.5 eV

	Calculated	BNL-325 (3rd) ¹⁸⁾ (barns)
natural	2.2	2.2 ± 0.2
⁵⁸ Ni	2.2	2.2 ± 0.2
⁶⁰ Ni	1.5	1.5 ± 0.2
⁶¹ Ni	2.4	1.6 ± 0.4
⁶² Ni	6.9	6.8 ± 0.2
⁶⁴ Ni	0.82	1.1 ± 0.2

Table 12 The γ -ray strength functions used for natural nickel files and for isotope nickel files.

Isotope	Natural nickel file	Isotope nickel file ($\times 10^{-4}$)
⁵⁸ Ni	0.462	0.462
⁶⁰ Ni	0.277	0.231
⁶¹ Ni	1.94	4.65
⁶² Ni	0.0952	0.138
⁶⁴ Ni	0.0587	0.0767

Table 13 Profile function of giant resonance.

$$\sigma(E) = \sum_{i=1}^2 \frac{\sigma_{mi}}{1 + [(E^2 - E_{mi}^2)^2 / E^2 \Gamma_i^2]}$$

$$E_{m1} = 16.30 \text{ MeV}$$

$$E_{m2} = 18.51 \text{ MeV}$$

$$\sigma_{m1} = 34.10$$

$$\sigma_{m2} = 55.2$$

$$\Gamma_1 = 2.44 \text{ MeV}$$

$$\Gamma_2 = 6.37 \text{ MeV}$$

Table 14 Discrete levels of isotopic nickel and
level grouping in natural nickel file.

MT	Natural	^{58}Ni		^{60}Ni		^{61}Ni		^{62}Ni		^{64}Ni	
		-Q(MeV)*	I ^π	-Q(MeV)	I ^π	-Q(MeV)	I ^π	-Q(MeV)	I ^π	-Q(MeV)	I ^π
51	0.0674					0.0674	5/2 ⁻				
52	0.2828					0.2830	1/2 ⁻				
53	0.6556					0.6560	3/2 ⁻				
54	0.9082					0.9088	5/2 ⁻				
55	1.0144					1.0150	7/2 ⁻				
56	1.0993					1.1000	3/2 ⁻				
						1.1323	5/2 ⁻				
57	1.1719					1.1857	3/2 ⁻	1.1729	2 ⁺		
58	1.3320			1.3325	2 ⁺					1.3459	2 ⁺
59	1.4549	1.4545	2 ⁺			1.4580	7/2 ⁻				
						1.6100	5/2 ⁻				
						1.7298	3/2 ⁻				
						1.808	7/2 ⁻				
60	1.9768					1.978	9/2 ⁺	2.0486	0 ⁺		
						1.997	3/2 ⁻				
						2.003	7/2 ⁻				
						2.019	7/2 ⁻				
						2.114	9/2 ⁺				
						2.123	1/2 ⁻				
61	2.1582			2.1589	2 ⁺					2.275	0 ⁺
62	2.2840			2.2848	0 ⁺	2.410	5/2 ⁻	2.3018	2 ⁺	2.3364	4 ⁺
63	2.4601	2.4595	4 ⁺			2.466	7/2 ⁻				
64	2.5049			2.5058	4 ⁺					2.608	4 ⁺
65	2.6253			2.6262	3 ⁺					2.750	2 ⁺
66	2.7763	2.7757	2 ⁺					2.8912	0 ⁺	2.865	0 ⁺
										2.885	2 ⁺
67	2.9033	2.9026	1 ⁺							2.971	2 ⁺
68	2.9435	2.9428	0 ⁺							3.028	0 ⁺
69	3.0390	3.0383	2 ⁺					3.0582	2 ⁺		
70	3.1179			3.1190	4 ⁺			3.1580	2 ⁺	3.165	4 ⁺
				3.1241	2 ⁺			3.1765	4 ⁺		
				3.1300	4 ⁺						

to be continued

Table 14 (continued)

MT	Natural -Q(MeV)*	^{58}Ni		^{60}Ni		^{61}Ni		^{62}Ni		^{64}Ni	
		-Q(MeV)	I ^π								
71	3.1853			3.1864	3 ⁺			3.2577	2 ⁺		
				3.1941	1 ⁺			3.2620	4 ⁺		
72	3.2652	3.2645	2 ⁺	3.2694	2 ⁺			3.2699	2 ⁺	3.273	2 ⁺
								3.2774	4 ⁺		
73	3.3172			3.3183	0 ⁺			3.3703	1 ⁺		
74	3.3798			3.3810	4 ⁺					3.393	3 ⁺
				3.3936	2 ⁺						
75	3.4216	3.4208	3 ⁺					3.4620	4 ⁺	3.459	1 ⁺
								3.4860	0 ⁺	3.483	4 ⁺
								3.5185	2 ⁺		
								3.5229	3 ⁺		
76	3.5248	3.5240	4 ⁺	3.5300	0 ⁺					3.560	3 ⁻
		3.5313	0 ⁺								
77	3.5878	3.5942	1 ⁺	3.5890	3 ⁺					3.647	2 ⁺
78	3.6185	3.6202	4 ⁺	3.6197	3 ⁺						
79	3.6697			3.6710	4 ⁺						
80	3.7277			3.7290	3 ⁺			3.7570	3 ⁻	3.748	4 ⁺
				3.7355	1 ⁺						
				3.7410	0 ⁺						
81	3.7761	3.7752	4 ⁺					3.8493	1 ⁺	3.795	1 ⁺
								3.8530	2 ⁺	3.808	3 ⁺
								3.8600	2 ⁺	3.848	5 ⁻
82	3.8701			3.8714	2 ⁺					3.965	4 ⁺
83	3.8998	3.8989	2 ⁺								
84	4.1089	4.108	2 ⁺								
85	4.2910	4.290	3 ⁺								
86	4.3440	4.343	6 ⁺								
		4.349	4 ⁺								
87	4.3810	4.380	5 ⁻								
88	4.4020	4.401	4 ⁺								
89	4.4508	4.4498	0 ⁺								
90	4.4730	4.472	3 ⁻								
91	2.5264		4.517		3.895		2.528		3.967		4.084

* The Q-values of natural nickel file was recalculated with the effective mass of natural nickel so as to keep the threshold values.

Table 15 Level density parameters of Nickel isotopes.

Isotope	57	58	59	60	61	62	63	64	65
a (MeV ⁻¹)	5.00*	6.45	6.97	7.55	8.14	8.77	9.37	9.98	10.57
σ_M^2/\sqrt{U} (MeV ^{-1/2})	4.78	5.557	5.841	6.145	6.455	6.773	7.076	7.380	7.673
Δ (MeV)	1.2*	2.47	1.20	2.47	1.20	2.60	1.20	2.70	1.20
E_x (MeV)	6.33*	7.30	8.00	10.00	7.00	9.00	3.00	4.32	4.00
T_c (MeV)	1.44*	1.49	1.35	1.26	1.17	1.08	1.36	1.15	0.947
σ_{exp}^2 **	-	5.95	6.58	4.47	5.17	4.26	4.34	5.03	4.30

* Values of Gilbert and Cameron⁴¹⁾

** The spin cut off parameters of the present work are given as

$$\sigma_M^2 = 0.146 \sqrt{a(E - \Delta)} A^{2/3} \quad E > E_x$$

$$= \sigma_{\text{exp}}^2 + (\sigma_M^2(E_x) - \sigma_{\text{exp}}^2) \frac{E}{E_x} \quad E < E_x$$

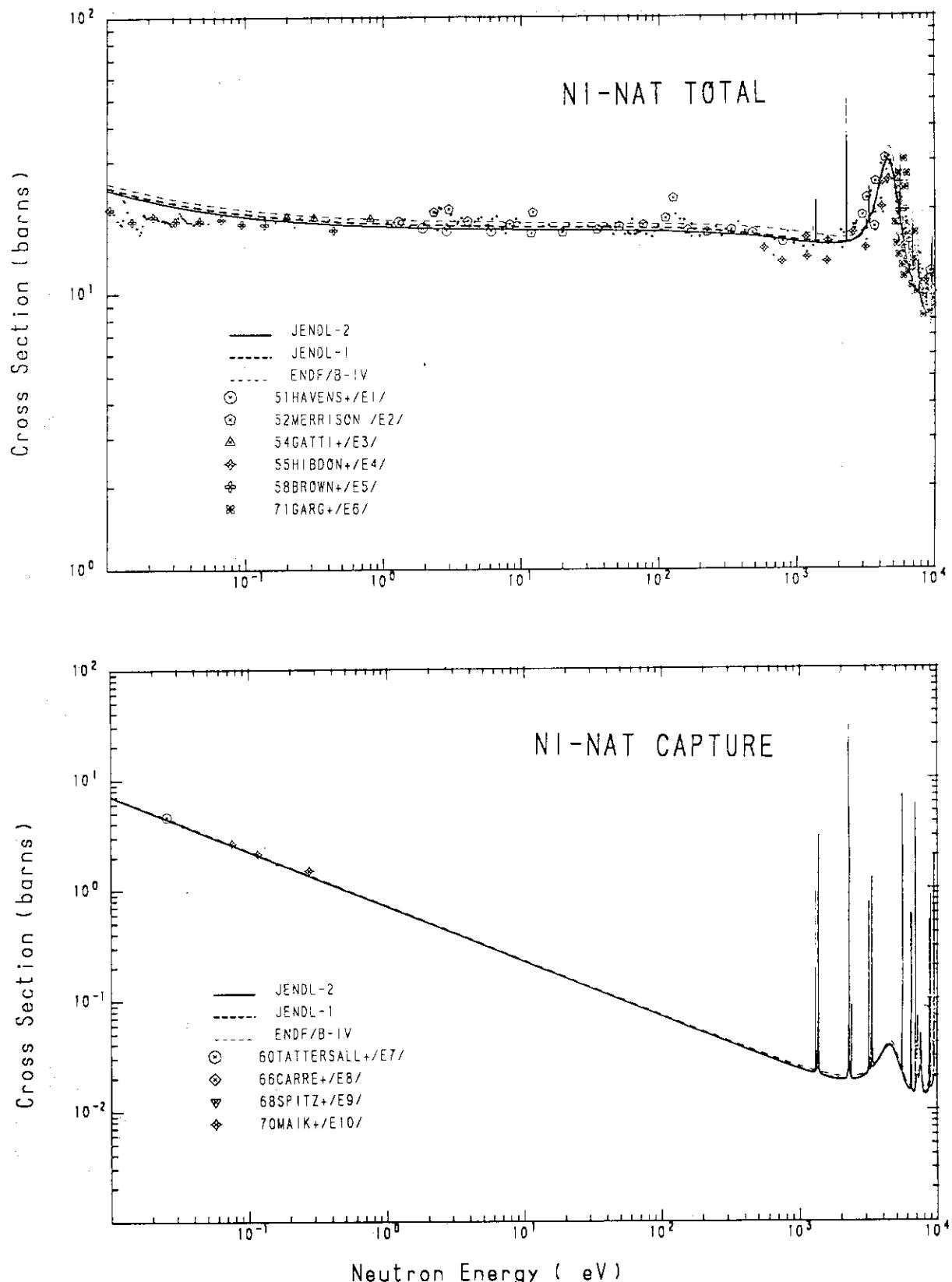


Fig. 1(a) Total and capture cross sections of natural nickel in the resonance region.

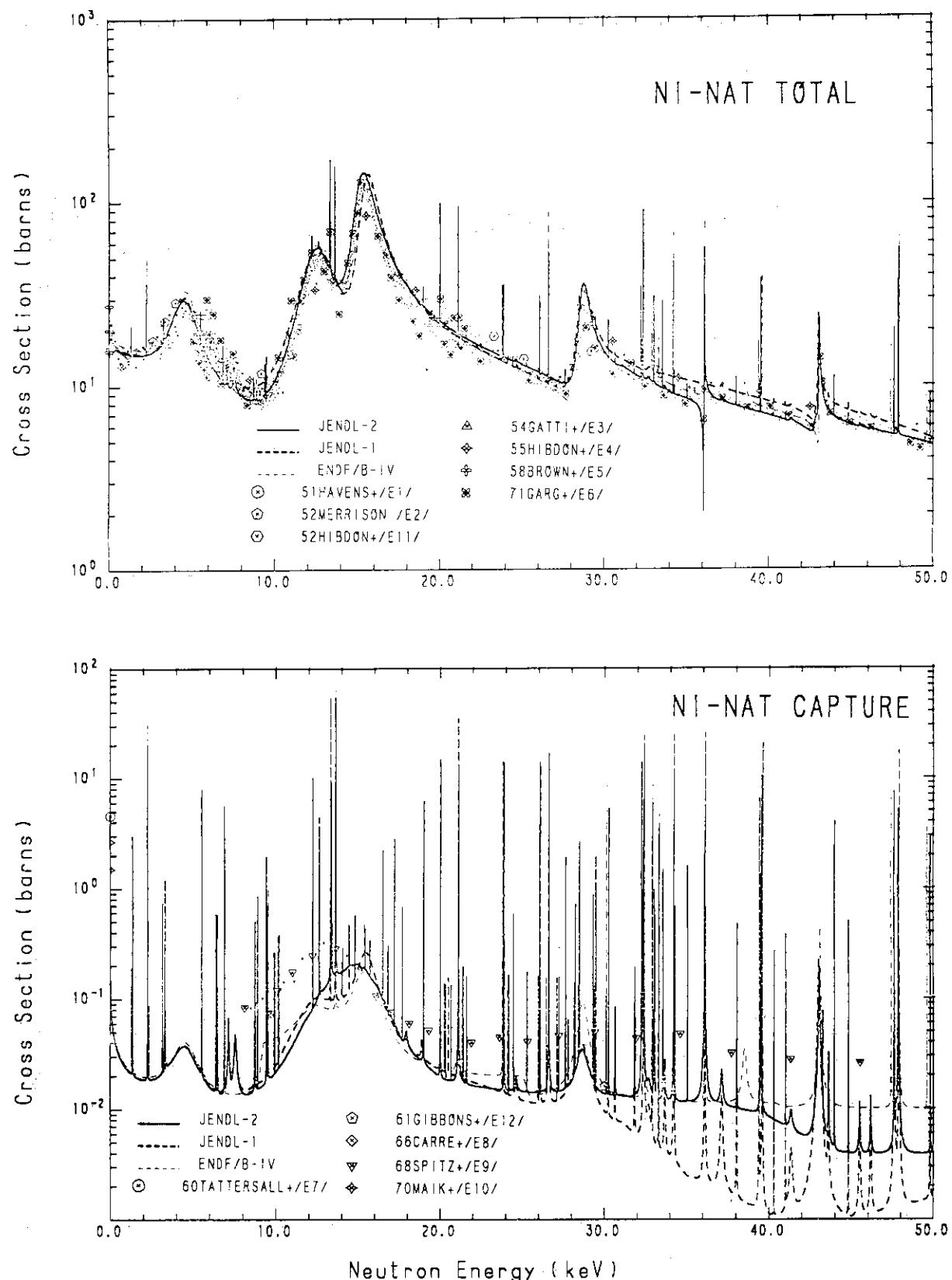


Fig. 1(b) Total and capture cross sections of natural nickel in the resonance region.

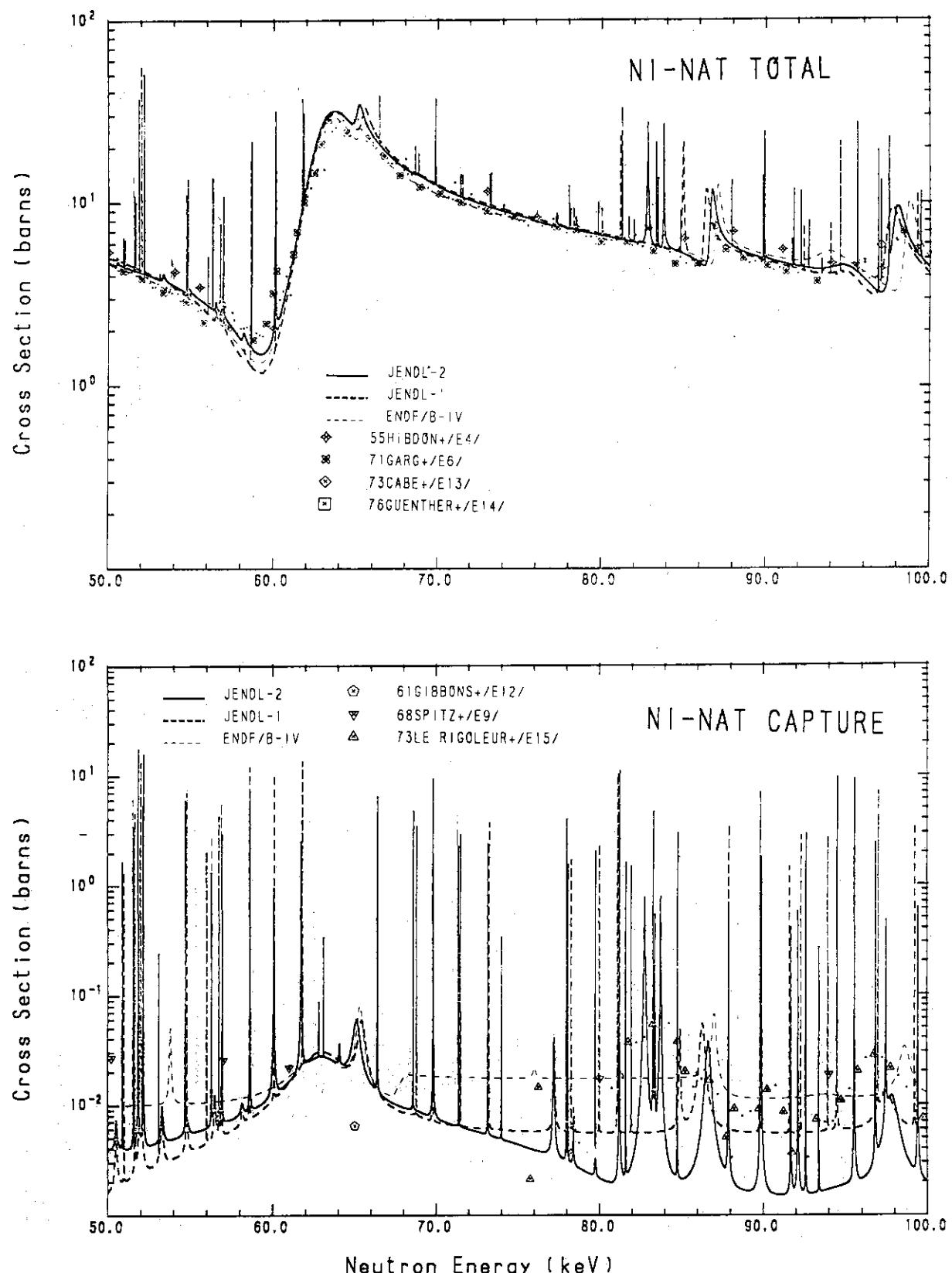


Fig. 1(c) Total and capture cross sections of natural nickel in the resonance region.

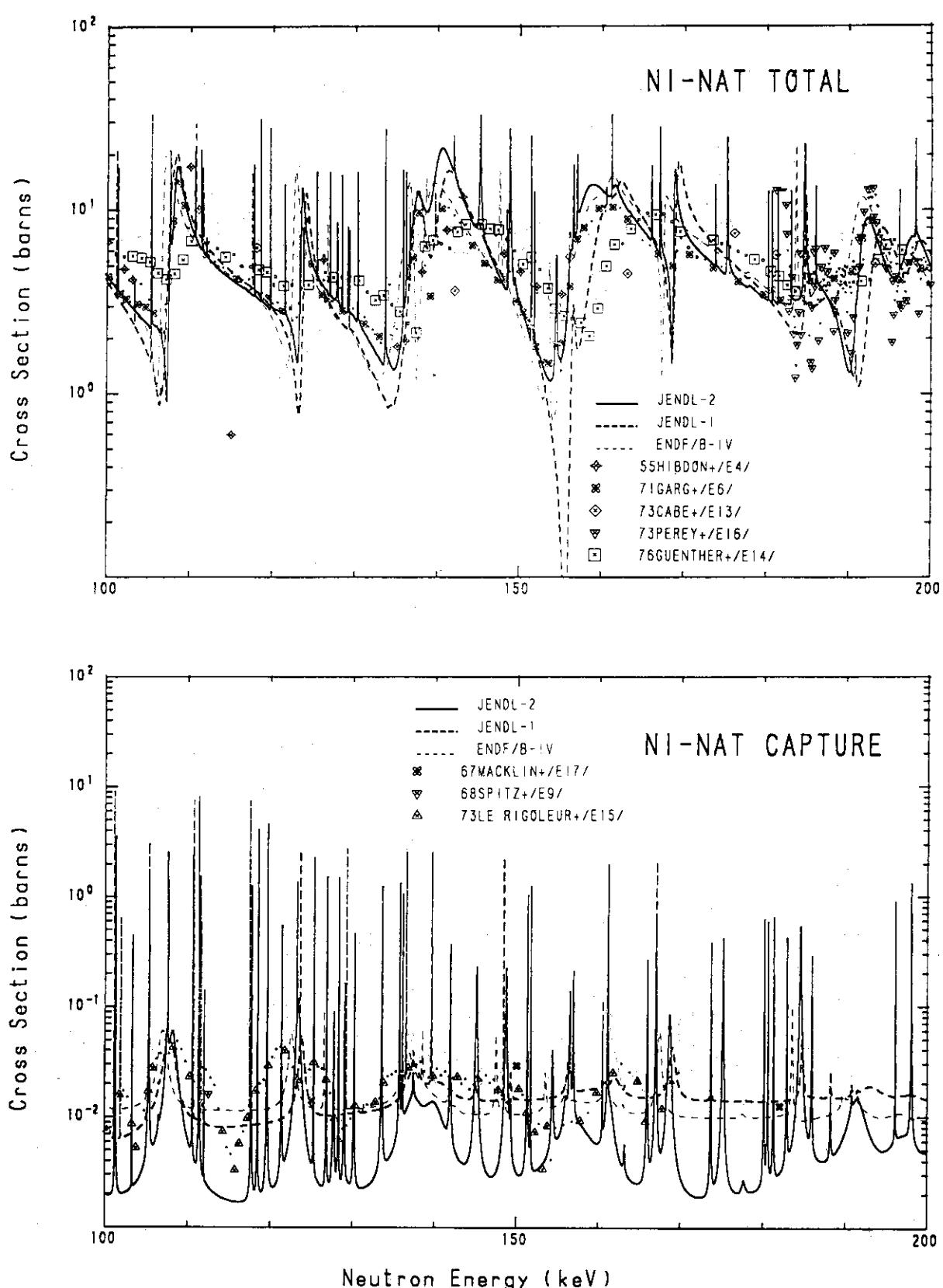


Fig. 1(d) Total and capture cross sections of natural nickel in the resonance region.

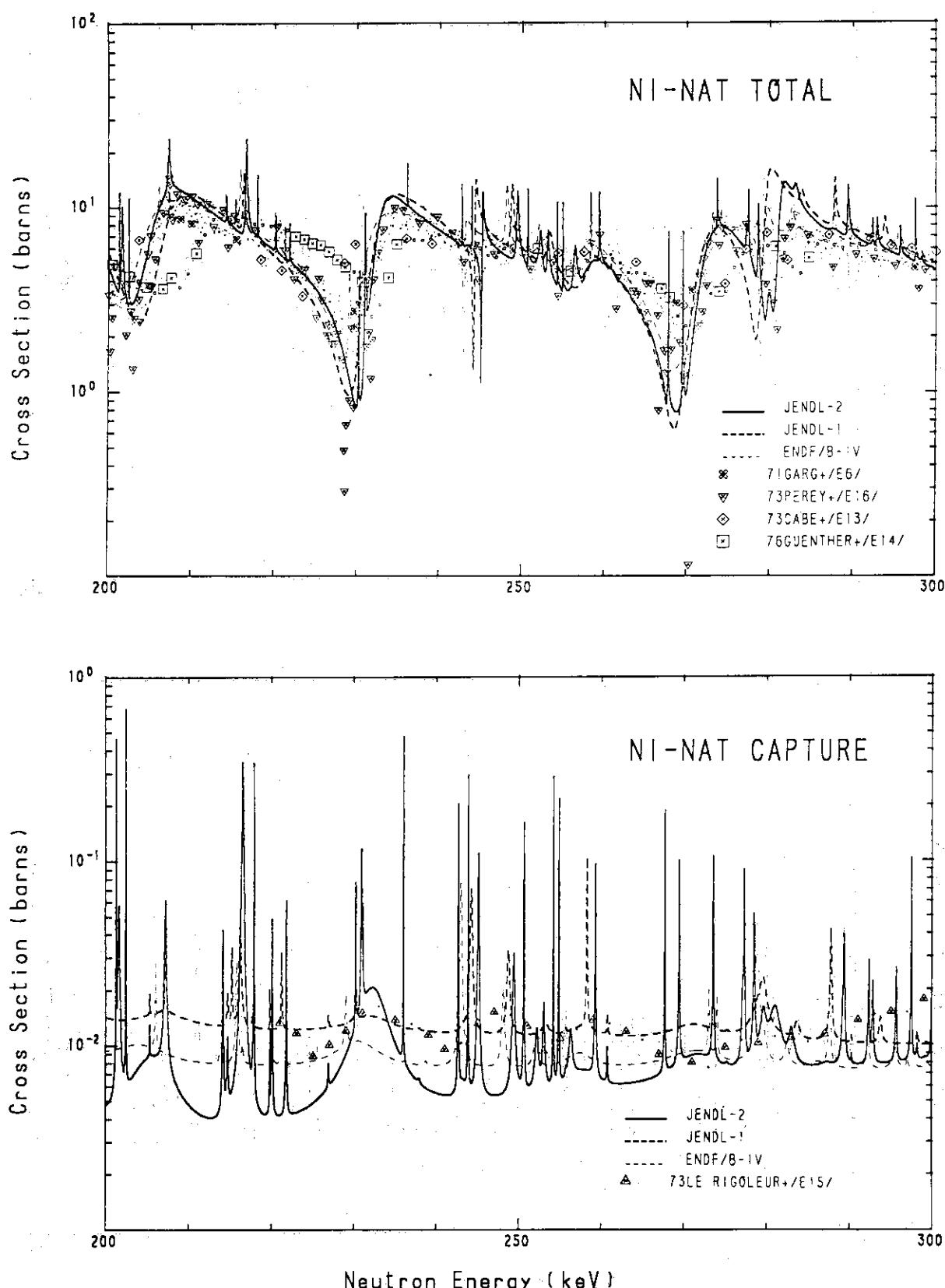


Fig. 1(e) Total and capture cross sections of natural nickel in the resonance region.

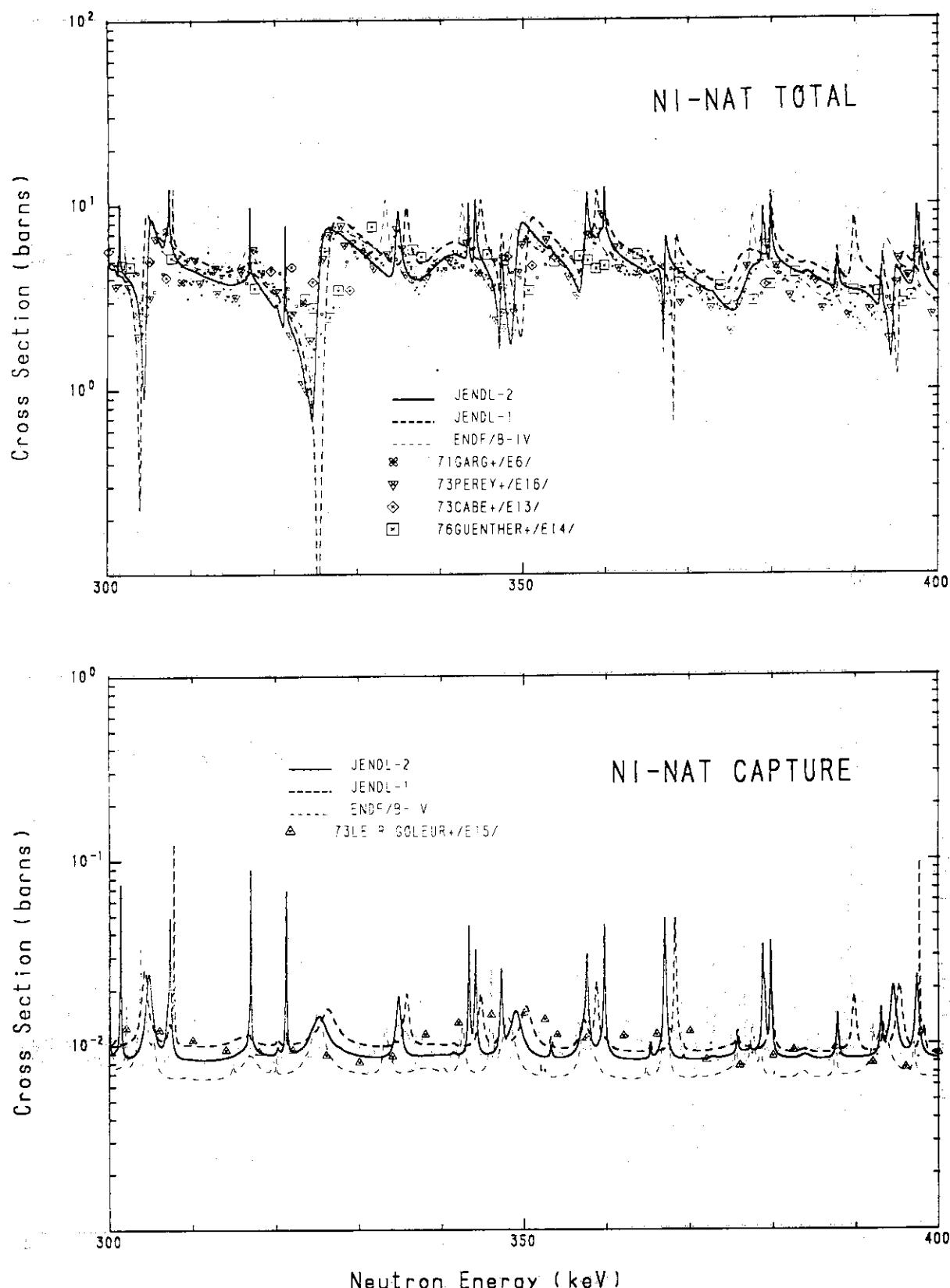


Fig. 1(f) Total and capture cross sections of natural nickel in the resonance region.

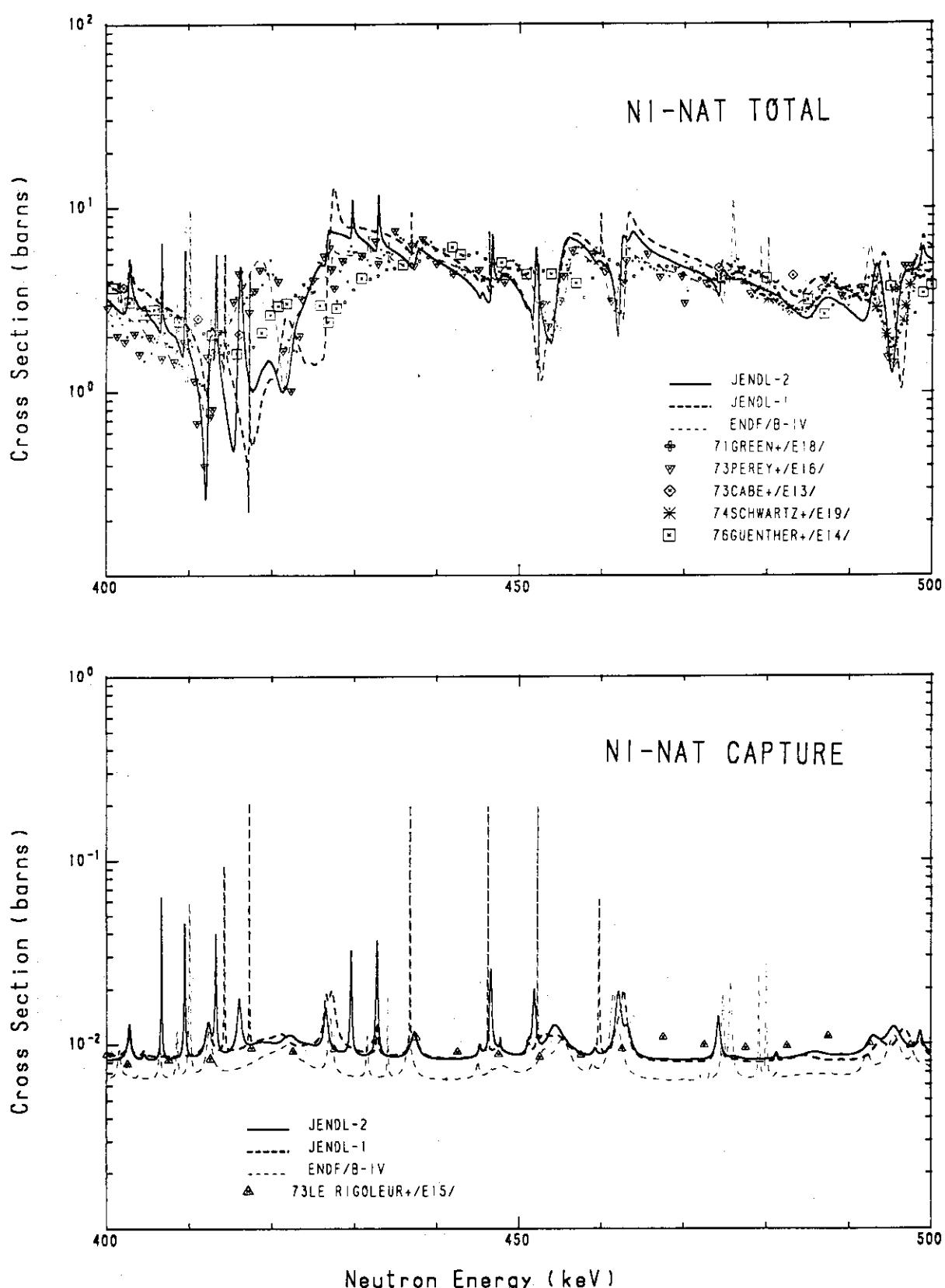


Fig. 1(g) Total and capture cross sections of natural nickel in the resonance region.

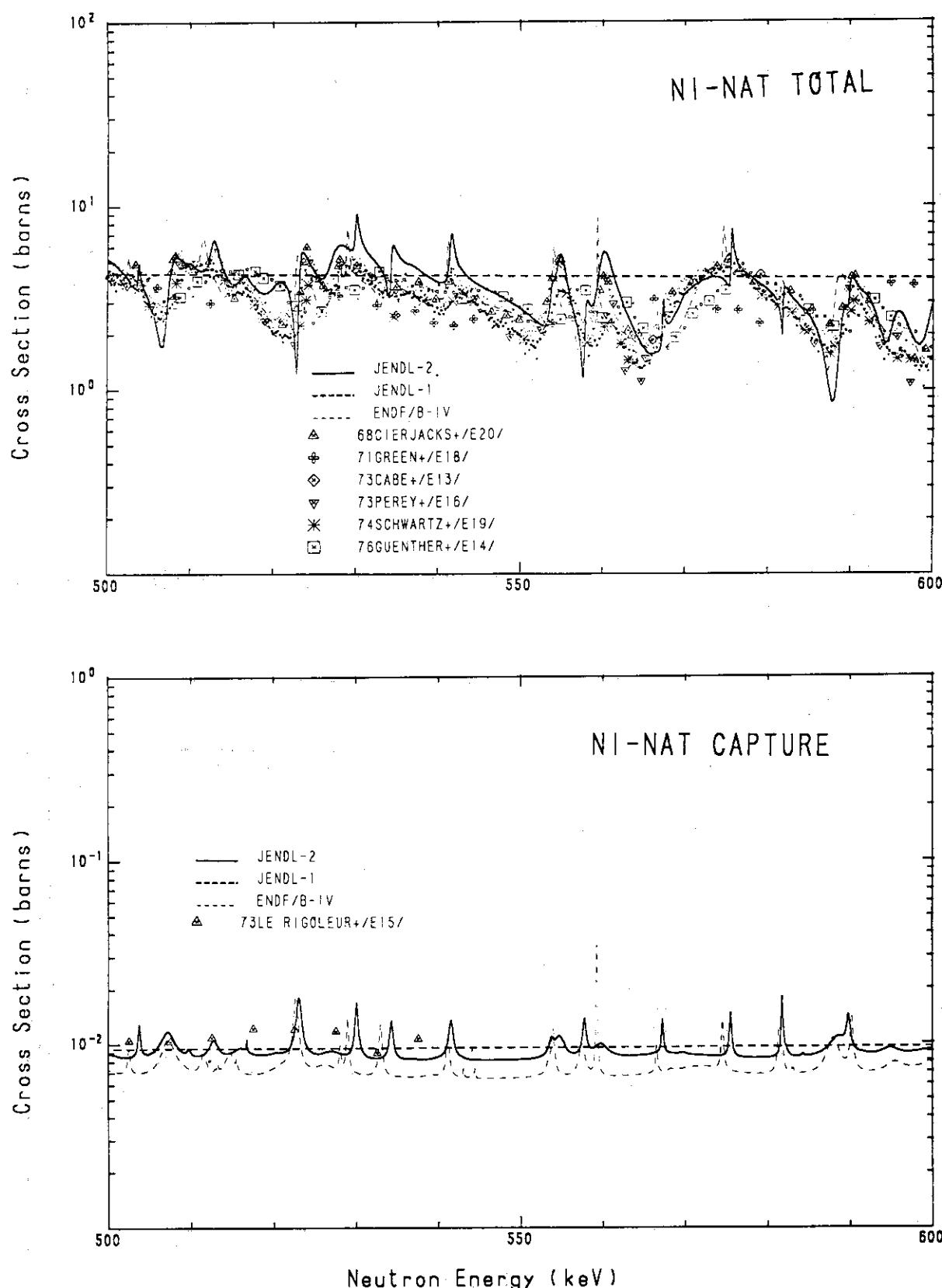


Fig. 1(h) Total and capture cross sections of natural nickel in the resonance region.

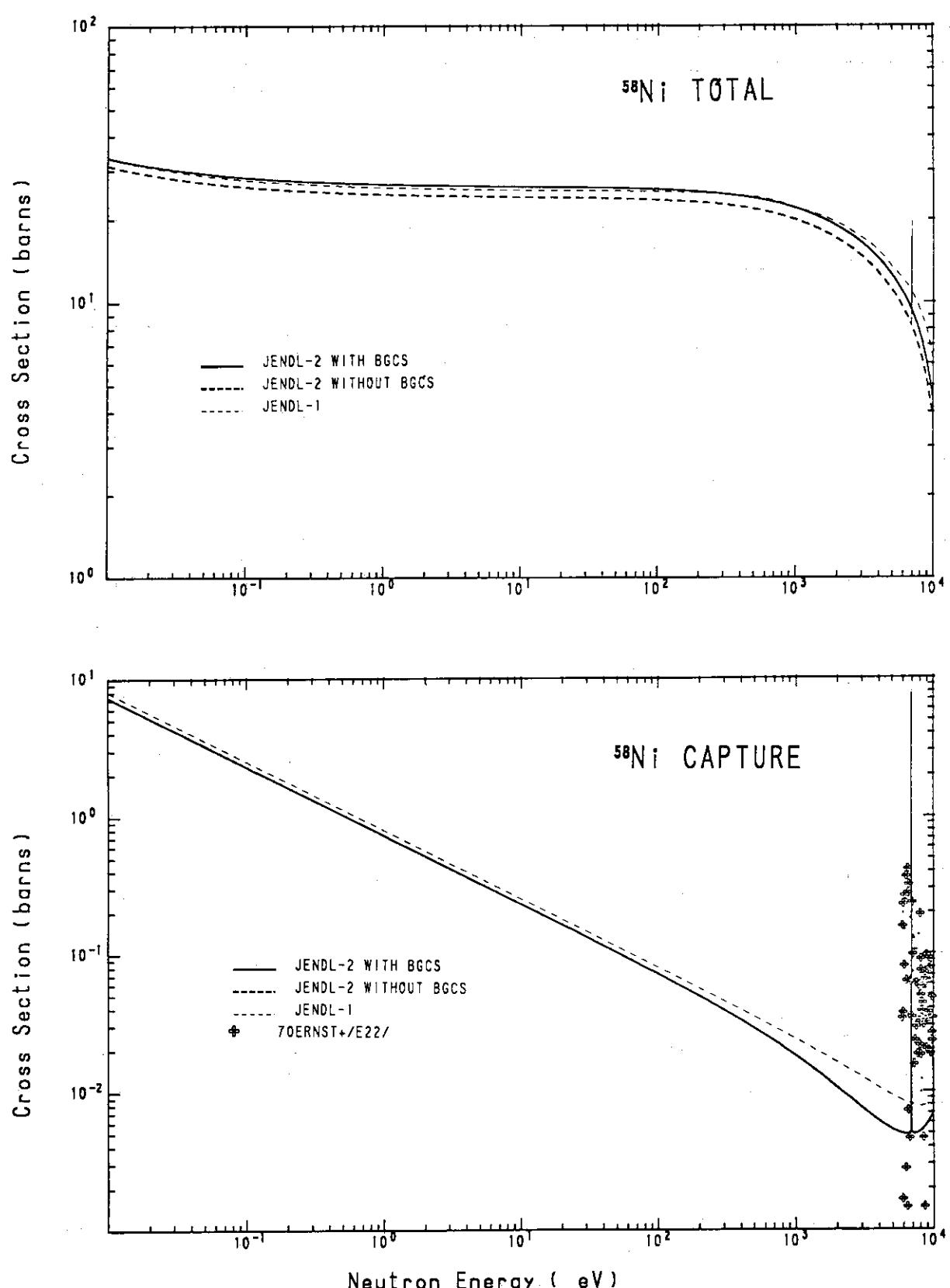


Fig. 2(a) Total and capture cross sections of ^{58}Ni in the resonance region.

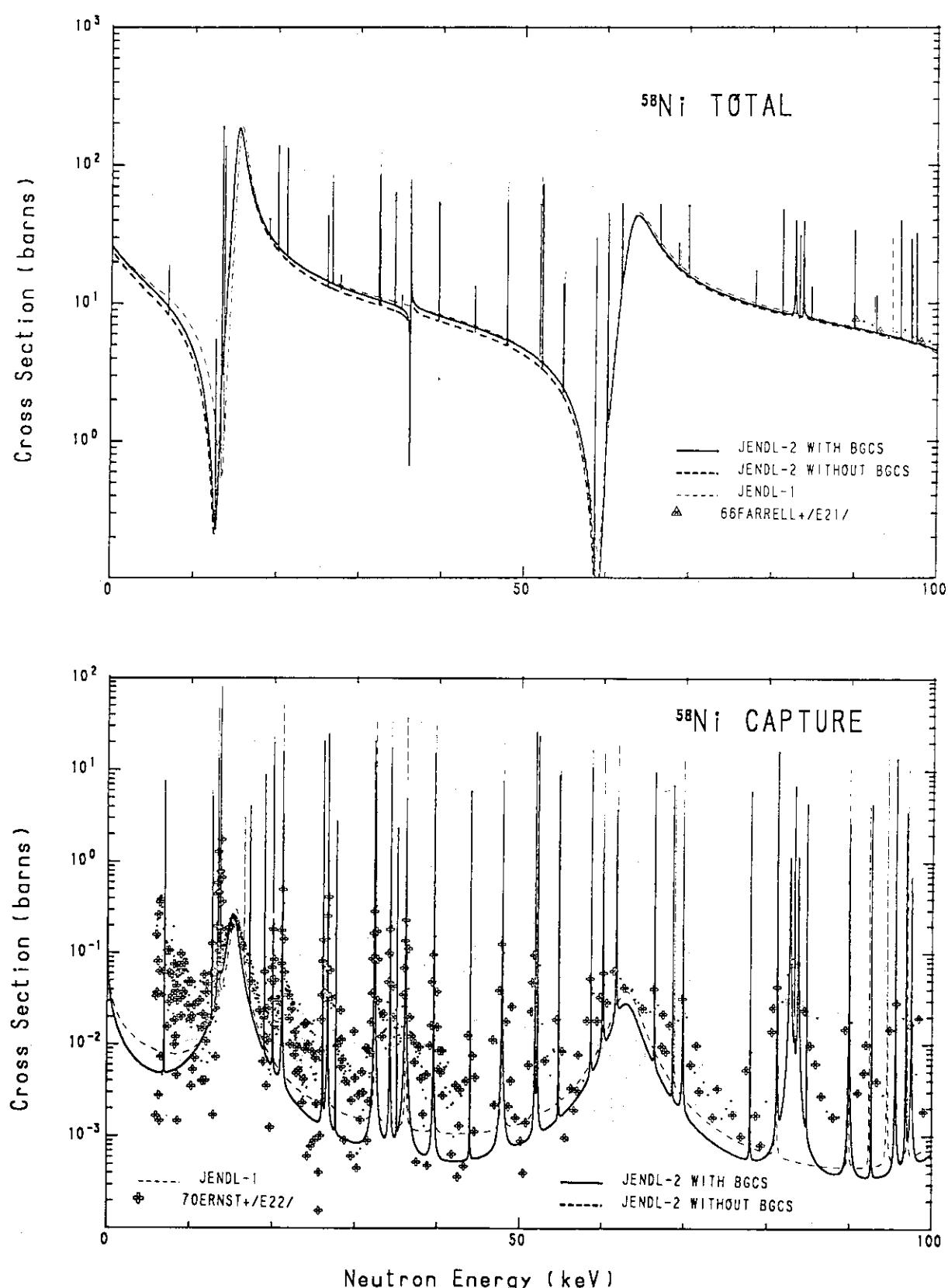


Fig. 2(b) Total and capture cross sections of ^{58}Ni in the resonance region.

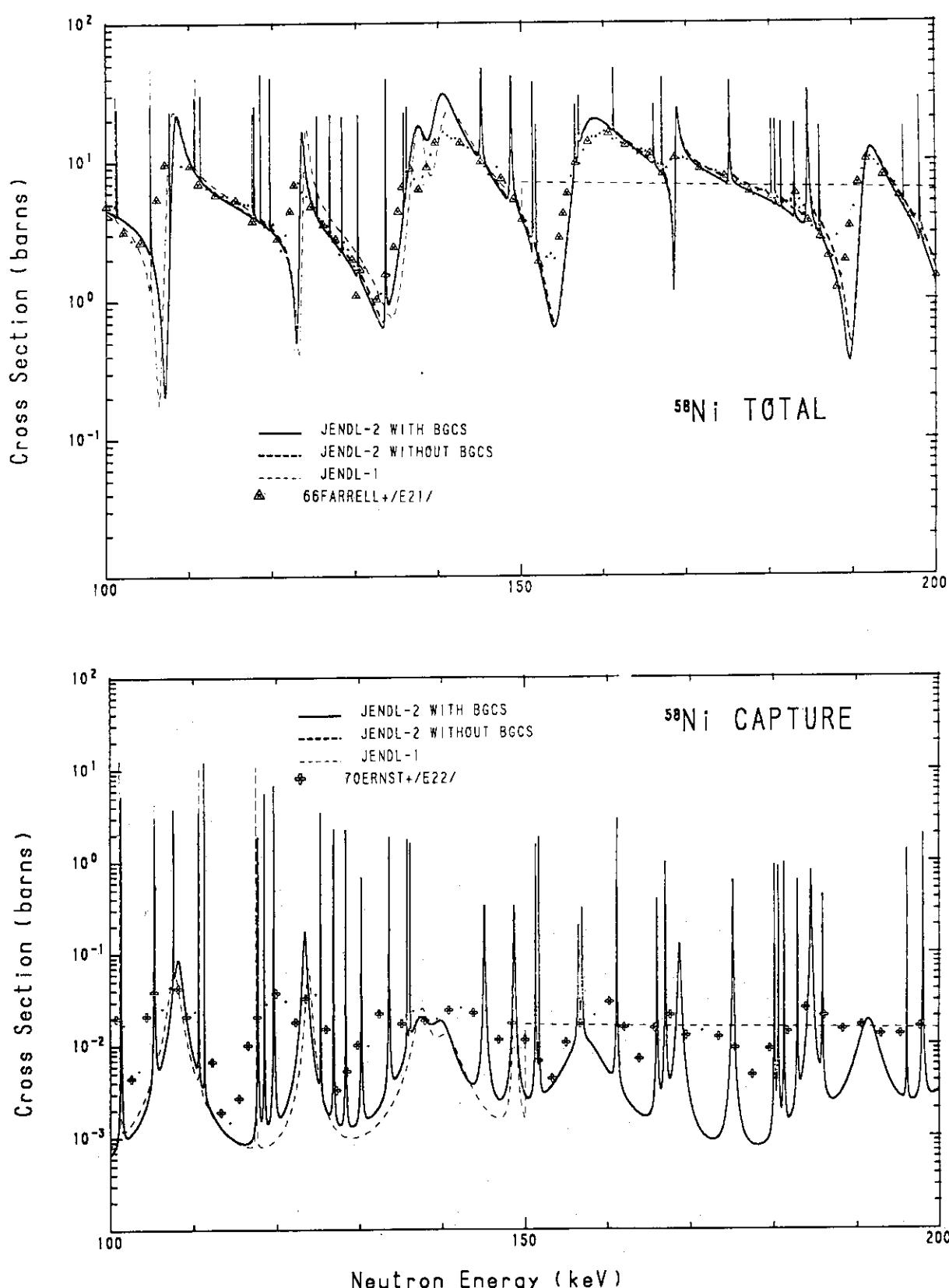


Fig. 2(c) Total and capture cross sections of ^{58}Ni in the resonance region.

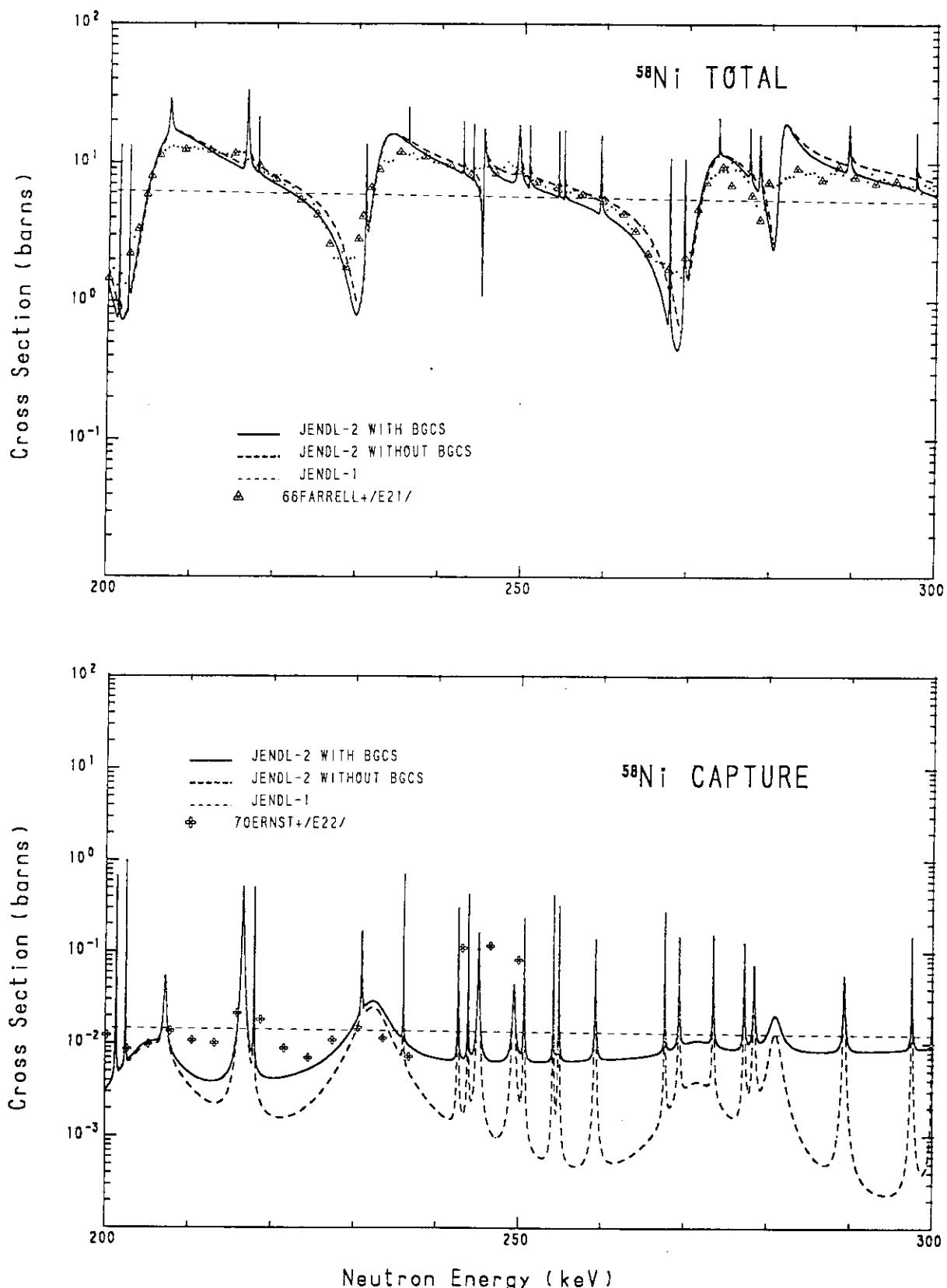


Fig. 2(d) Total and capture cross sections of ^{58}Ni in the resonance region.

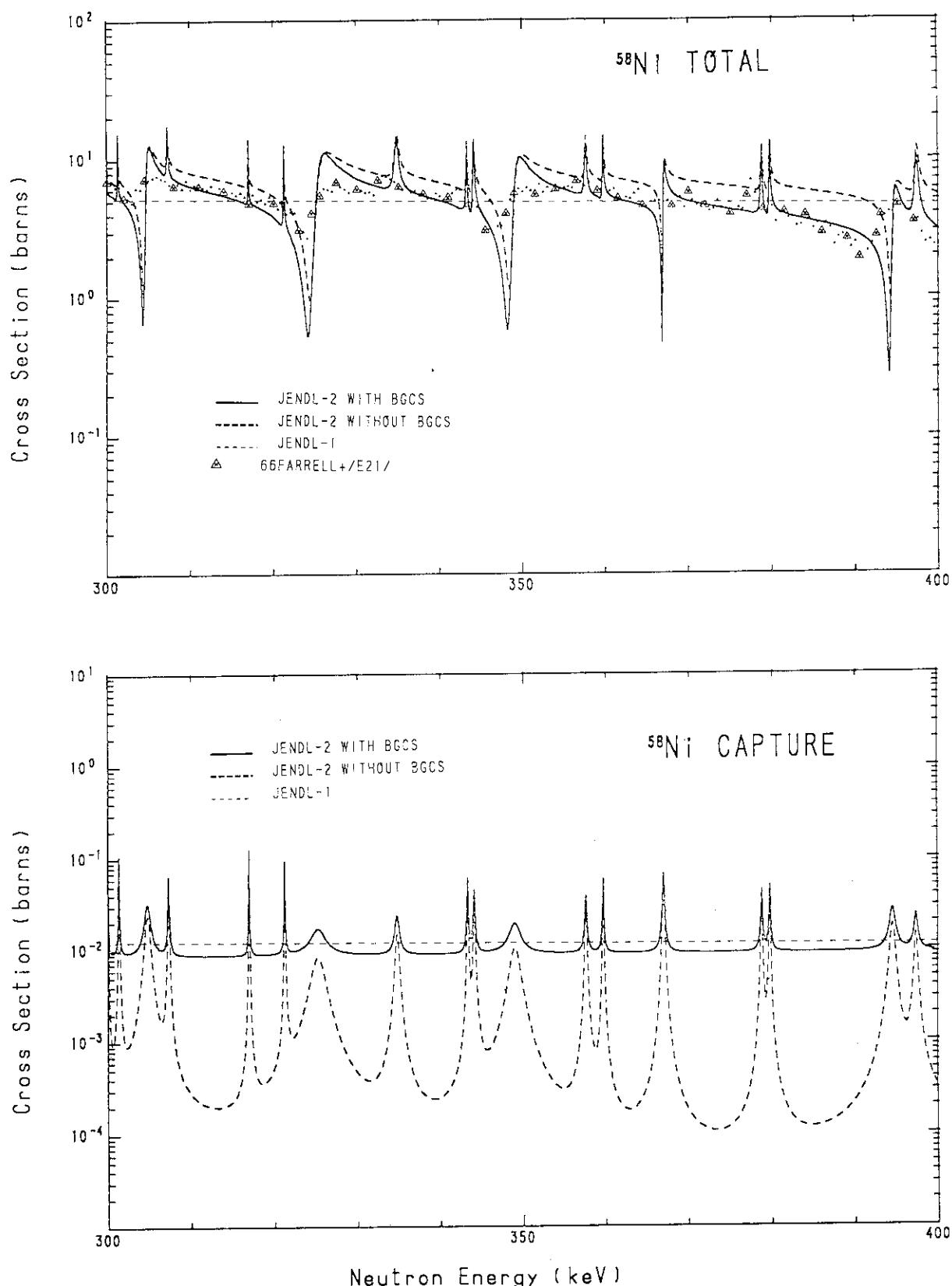


Fig. 2(e) Total and capture cross sections of ^{58}Ni in the resonance region.

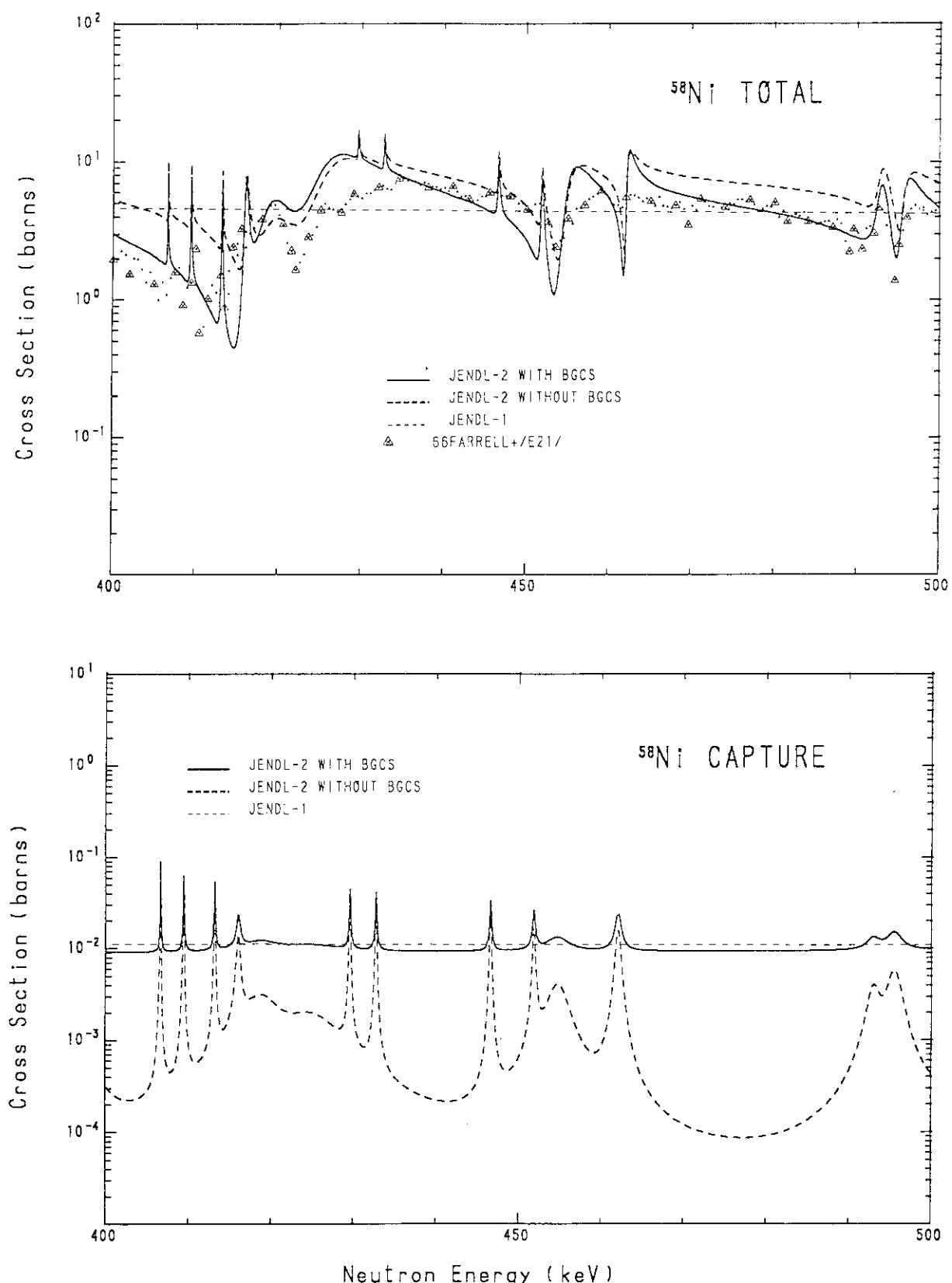


Fig. 2(f) Total and capture cross sections of ^{58}Ni in the resonance region.

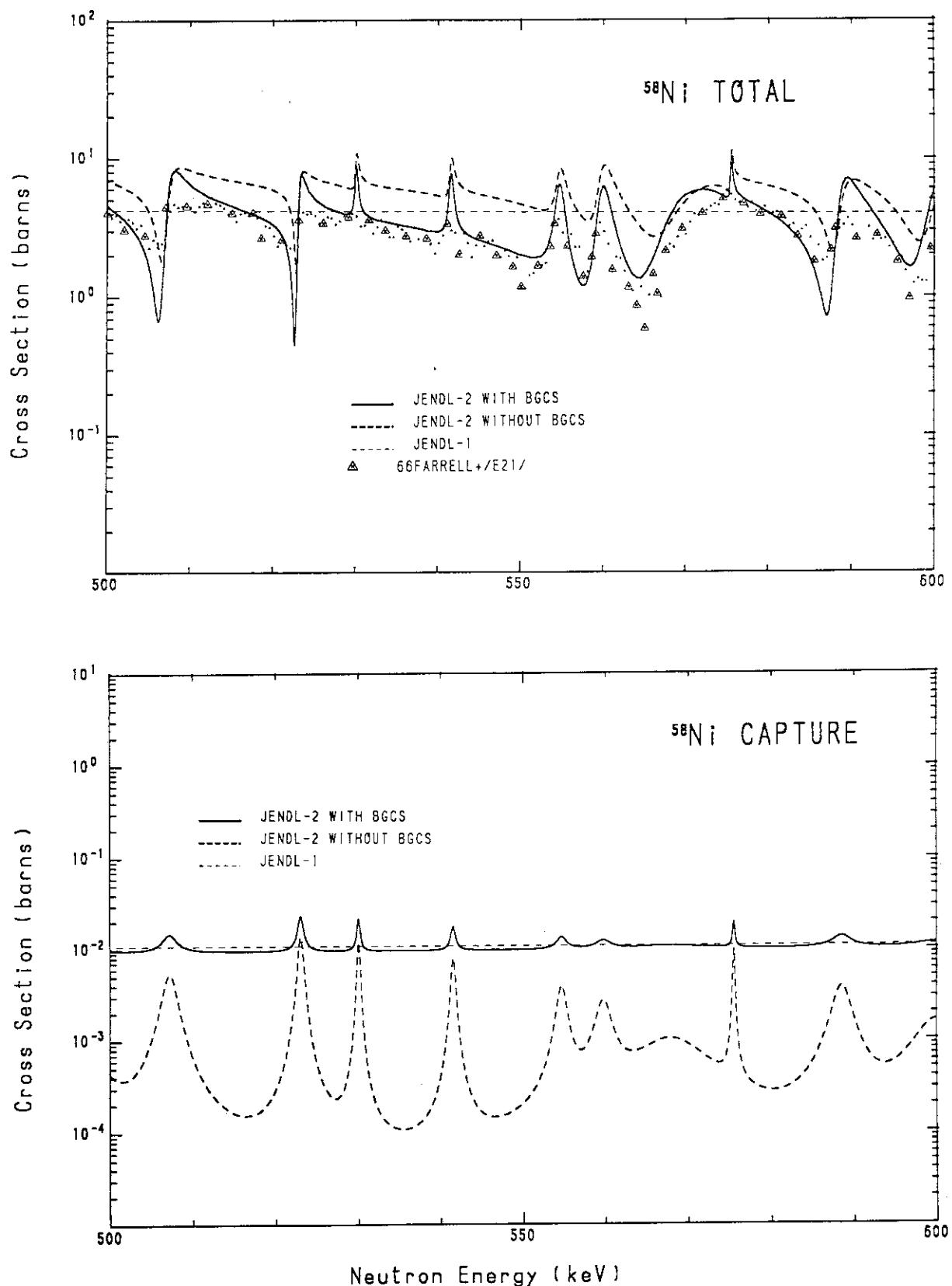


Fig. 2(g) Total and capture cross sections of ^{58}Ni in the resonance region.

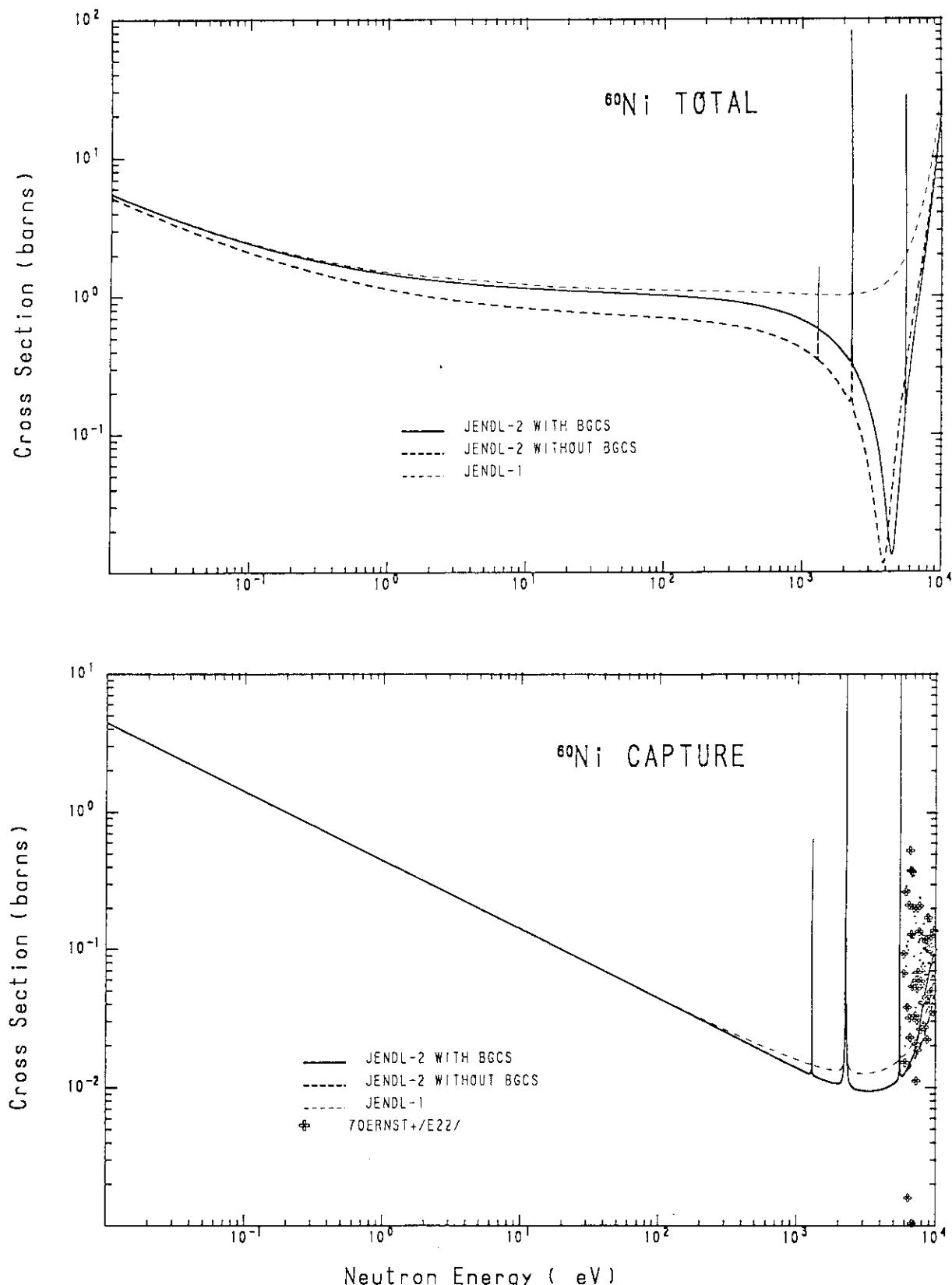


Fig. 3(a) Total and capture cross sections of ^{60}Ni in the resonance region.

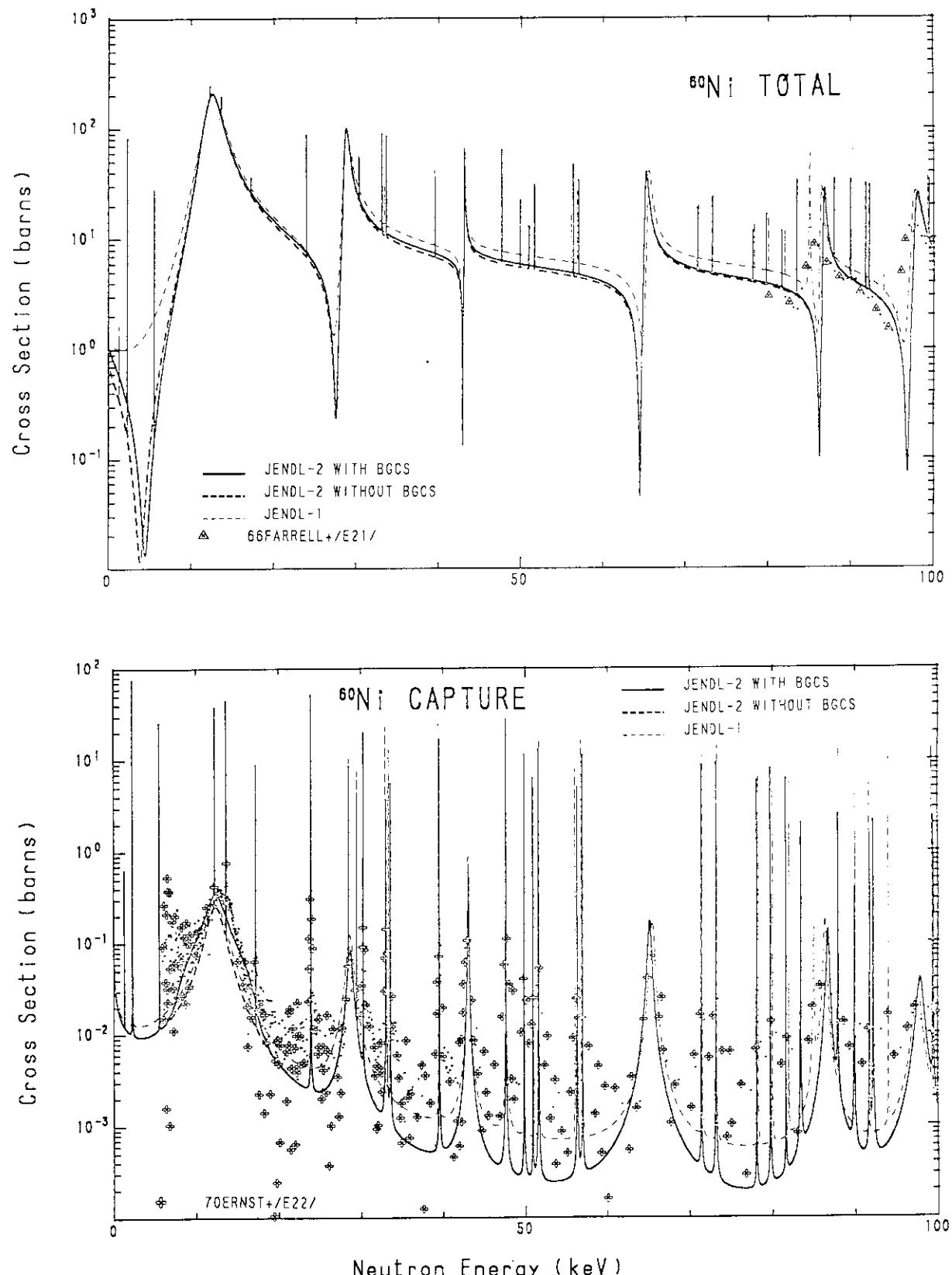


Fig. 3(b) Total and capture cross sections of ^{60}Ni in the resonance region.

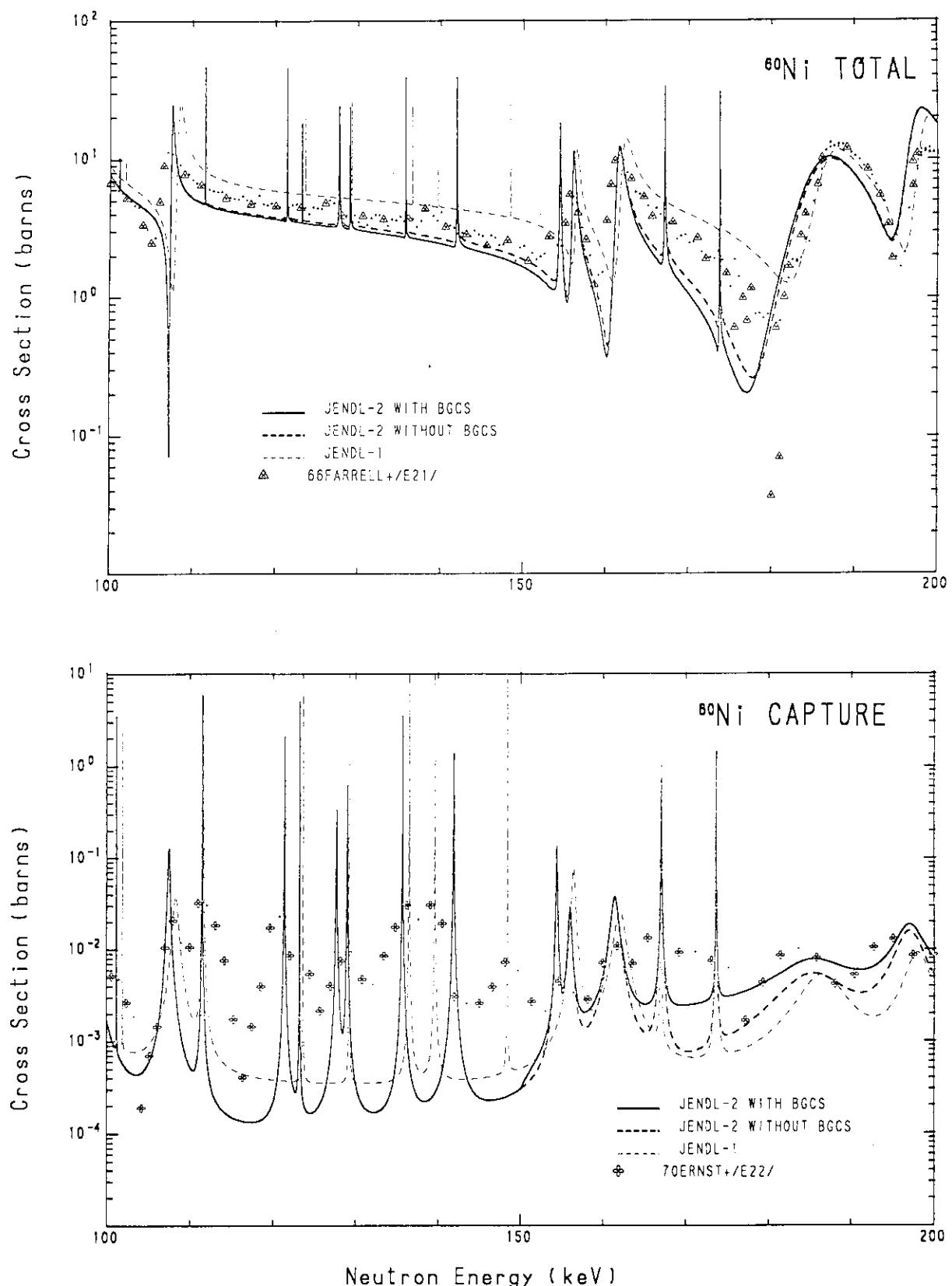


Fig. 3(c) Total and capture cross sections of ^{60}Ni in the resonance region.

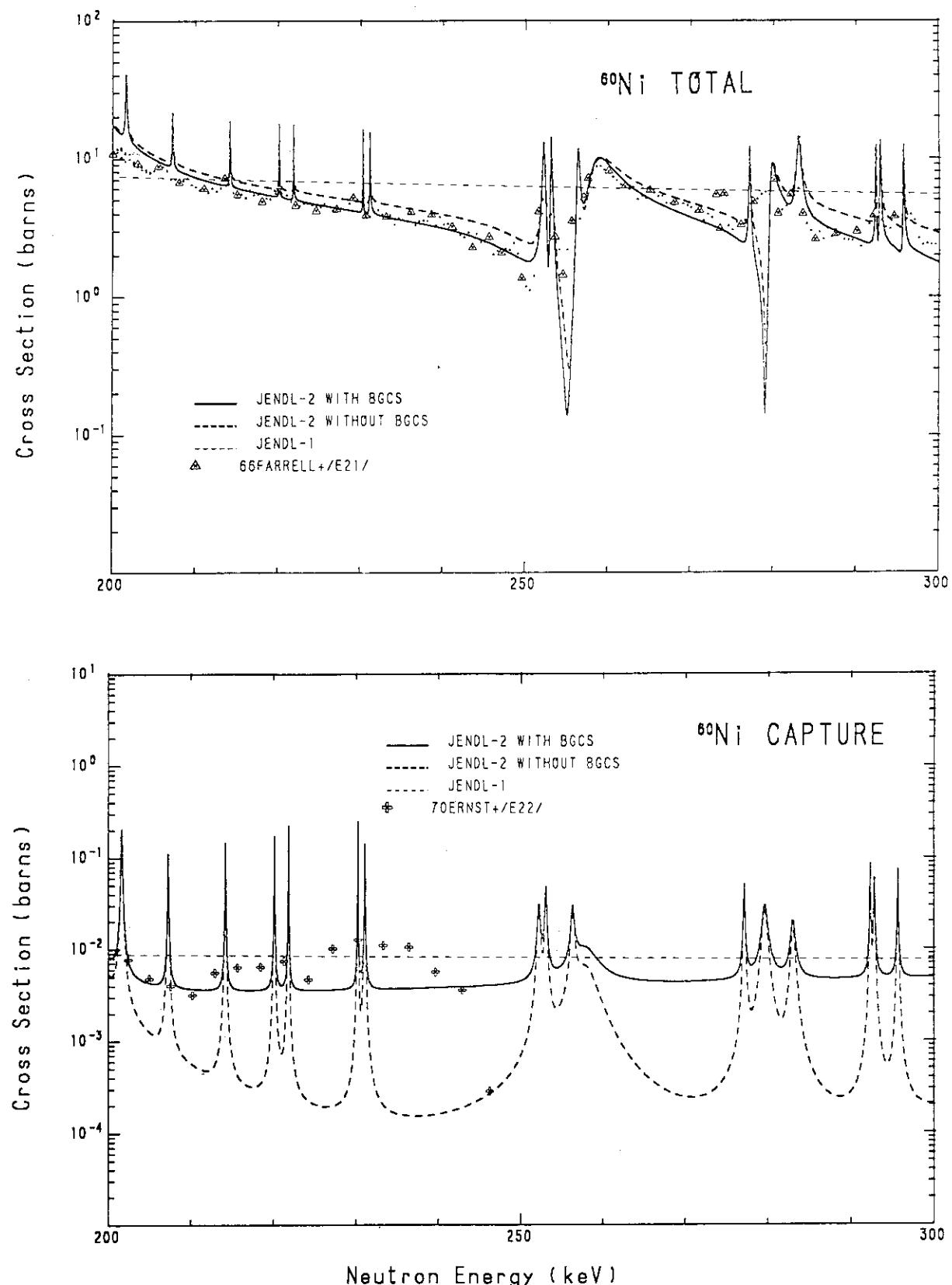


Fig. 3(d) Total and capture cross sections of ^{60}Ni in the resonance region.

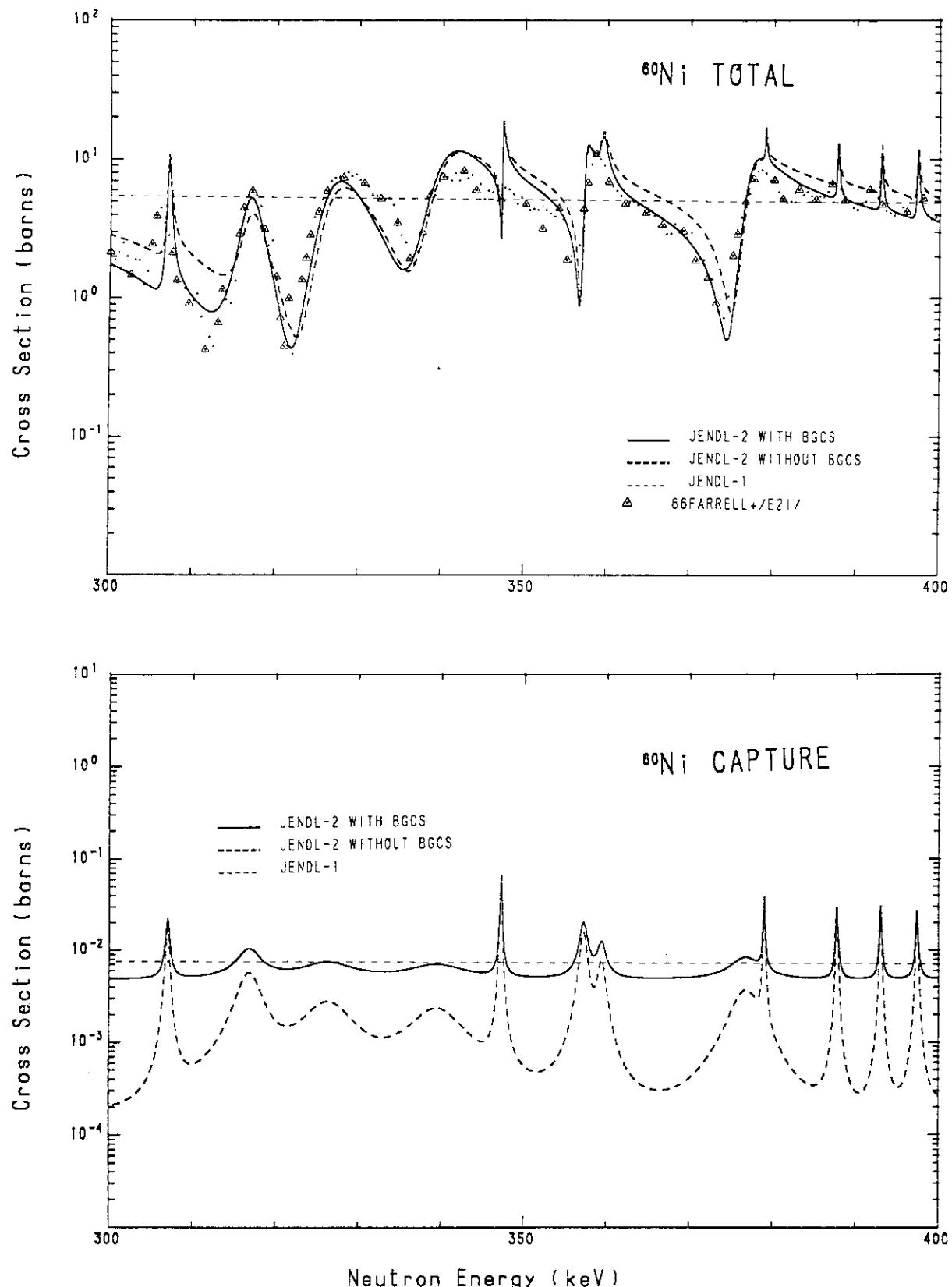


Fig. 3(e) Total and capture cross sections of ^{60}Ni in the resonance region.

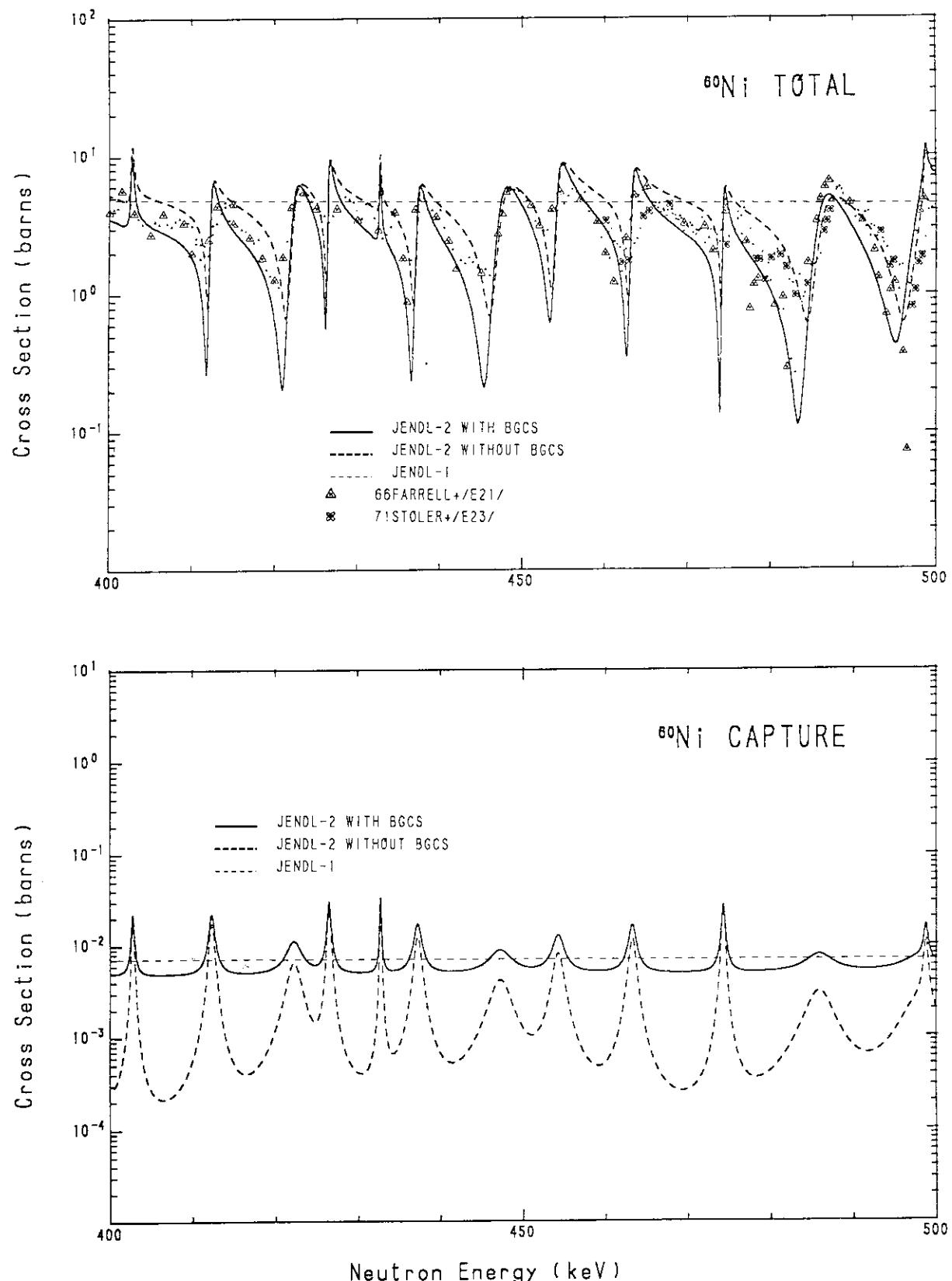


Fig. 3(f) Total and capture cross sections of ^{60}Ni in the resonance region.

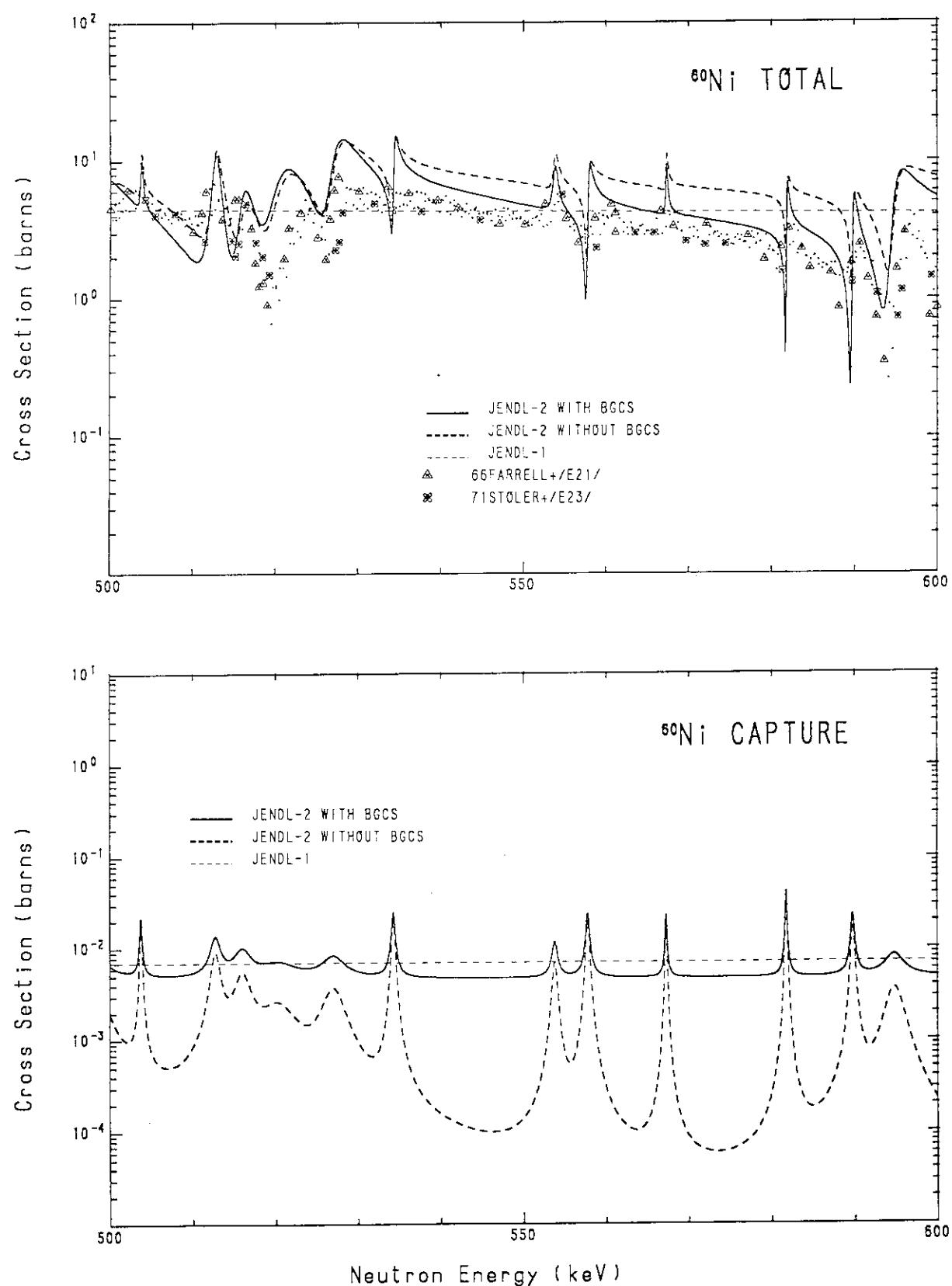


Fig. 3(g) Total and capture cross sections of ^{60}Ni in the resonance region.

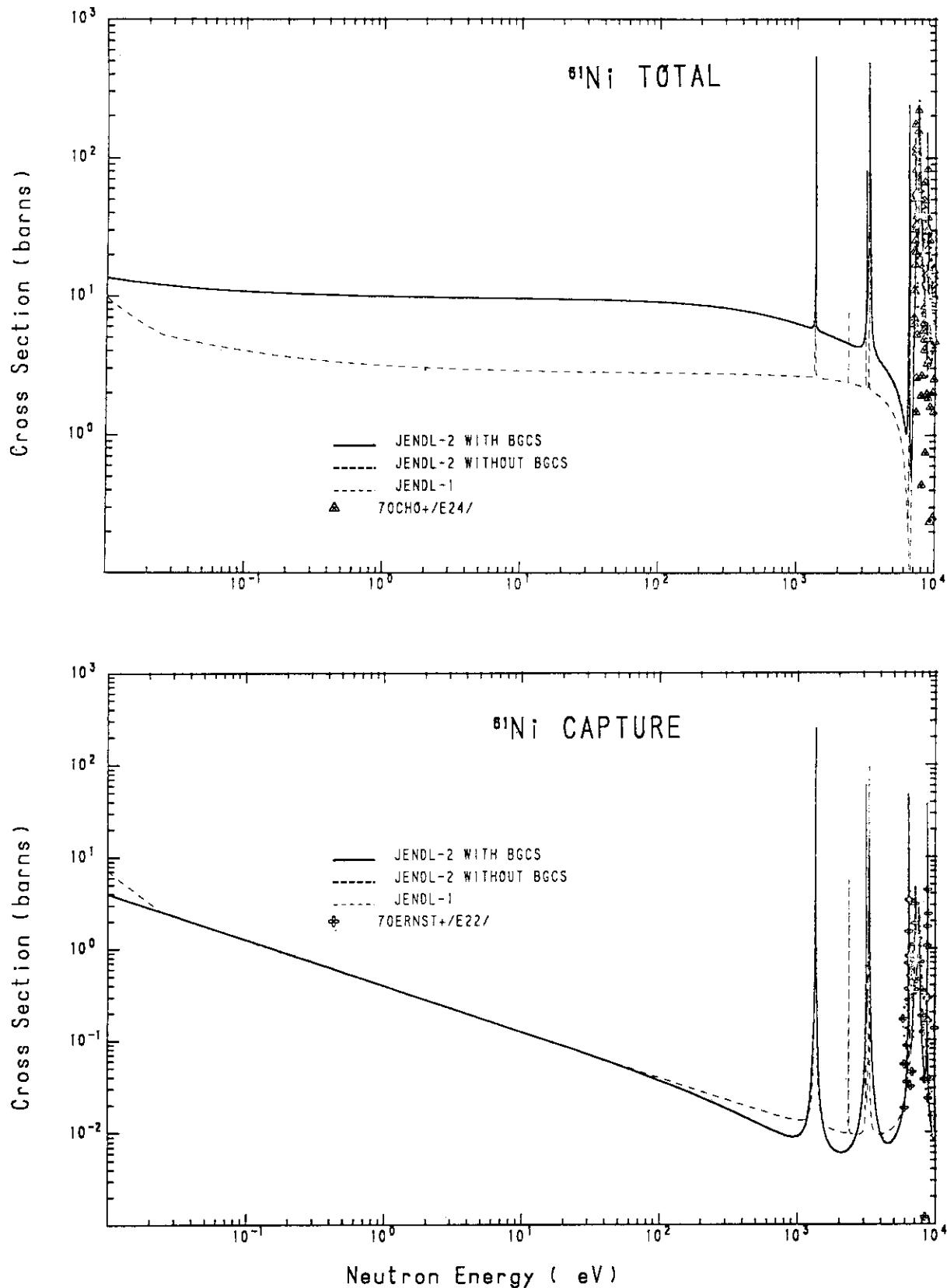


Fig. 4(a) Total and capture cross sections of ^{61}Ni in the resonance region.

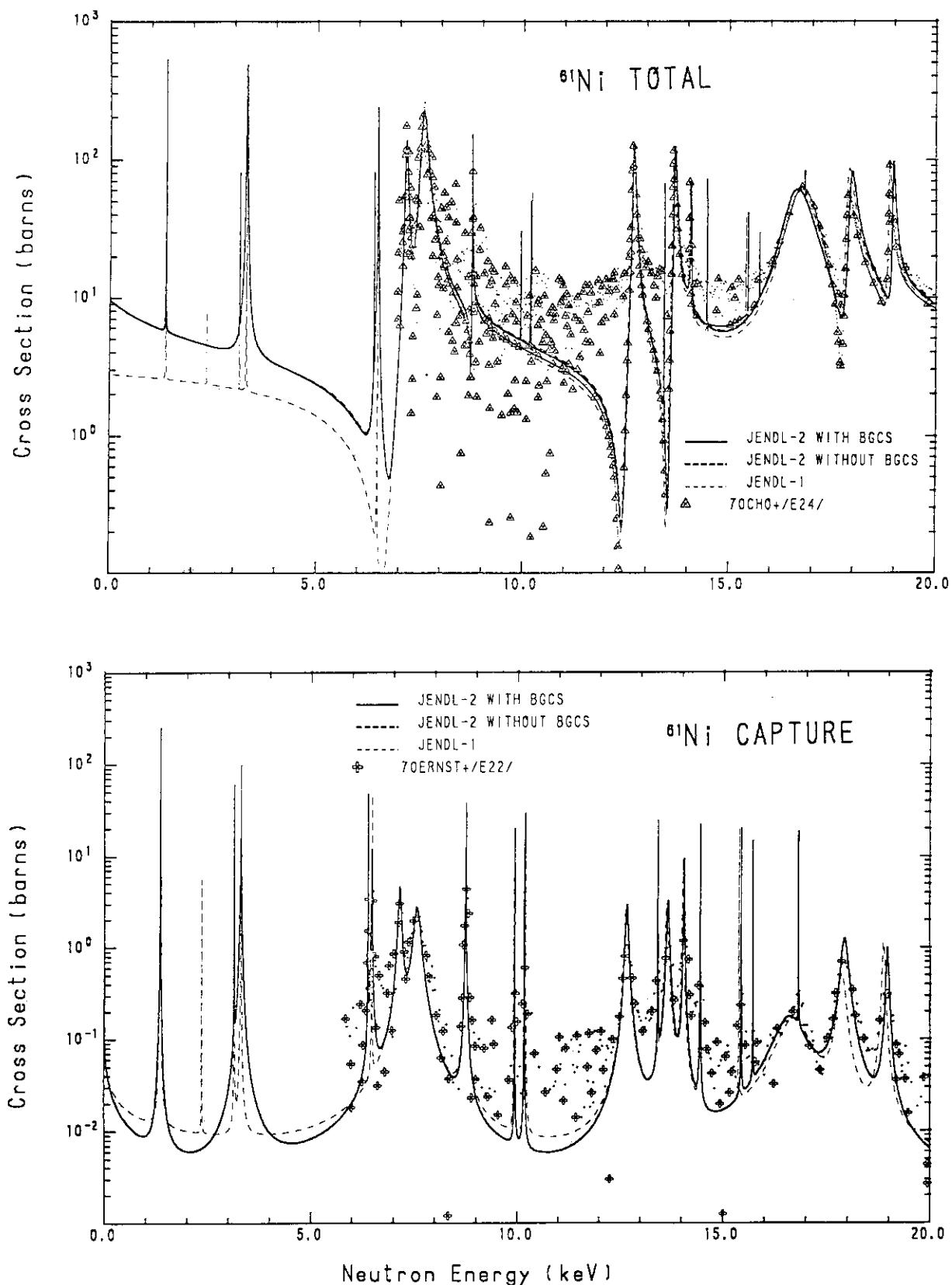


Fig. 4(b) Total and capture cross sections of ^{61}Ni in the resonance region.

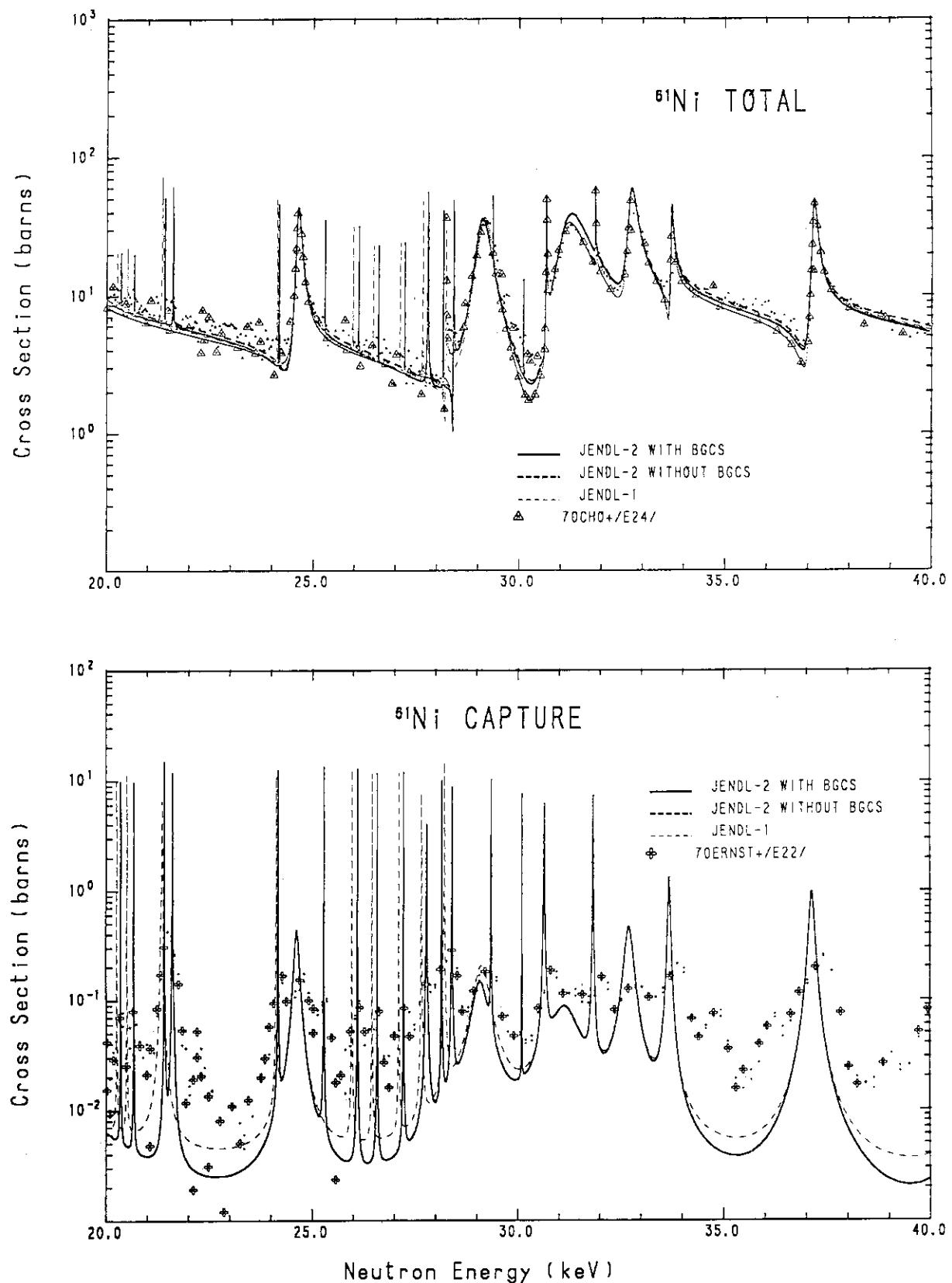


Fig. 4(c) Total and capture cross sections of ^{61}Ni in the resonance region.

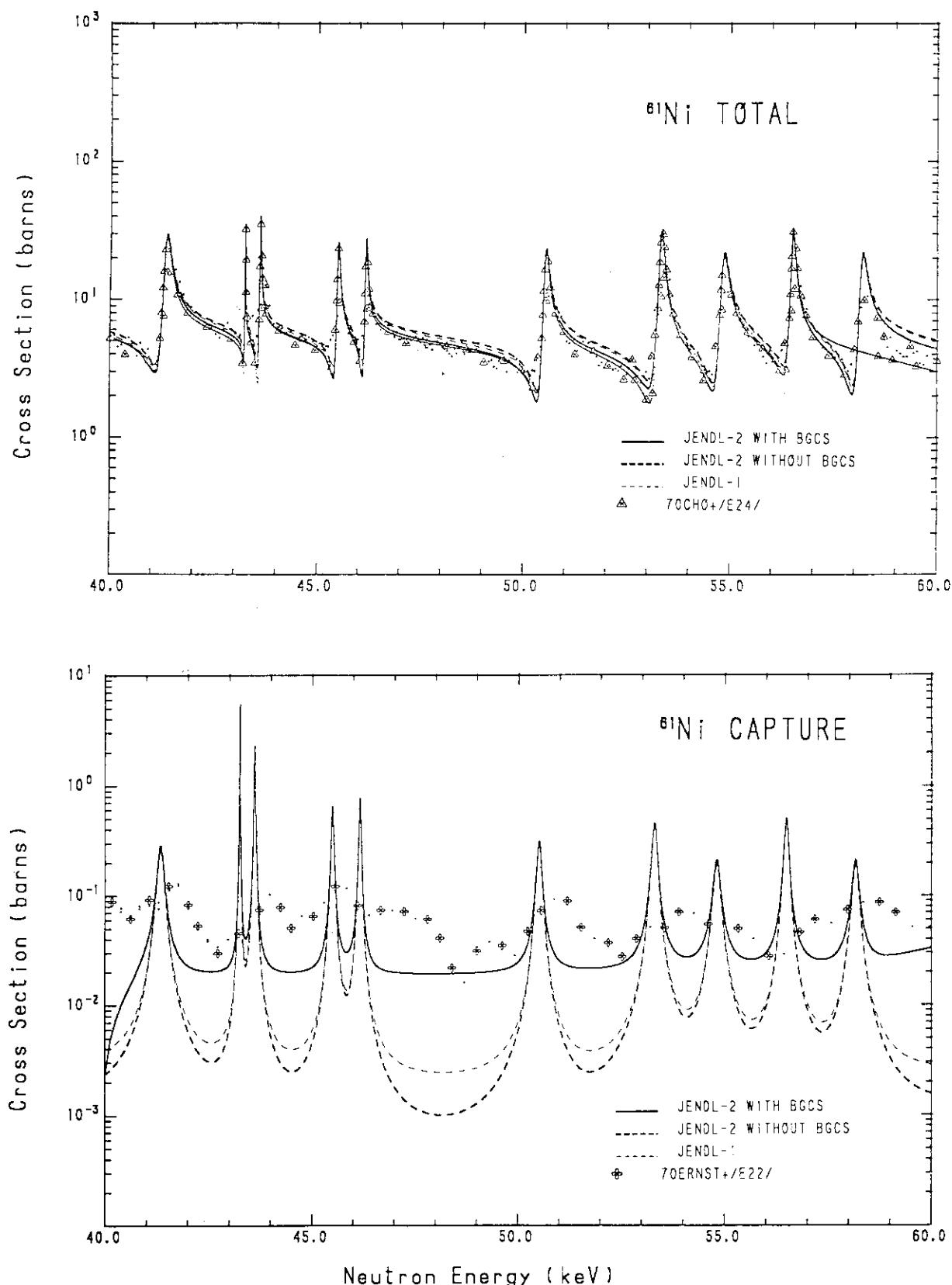


Fig. 4(d) Total and capture cross sections of ^{61}Ni in the resonance region.

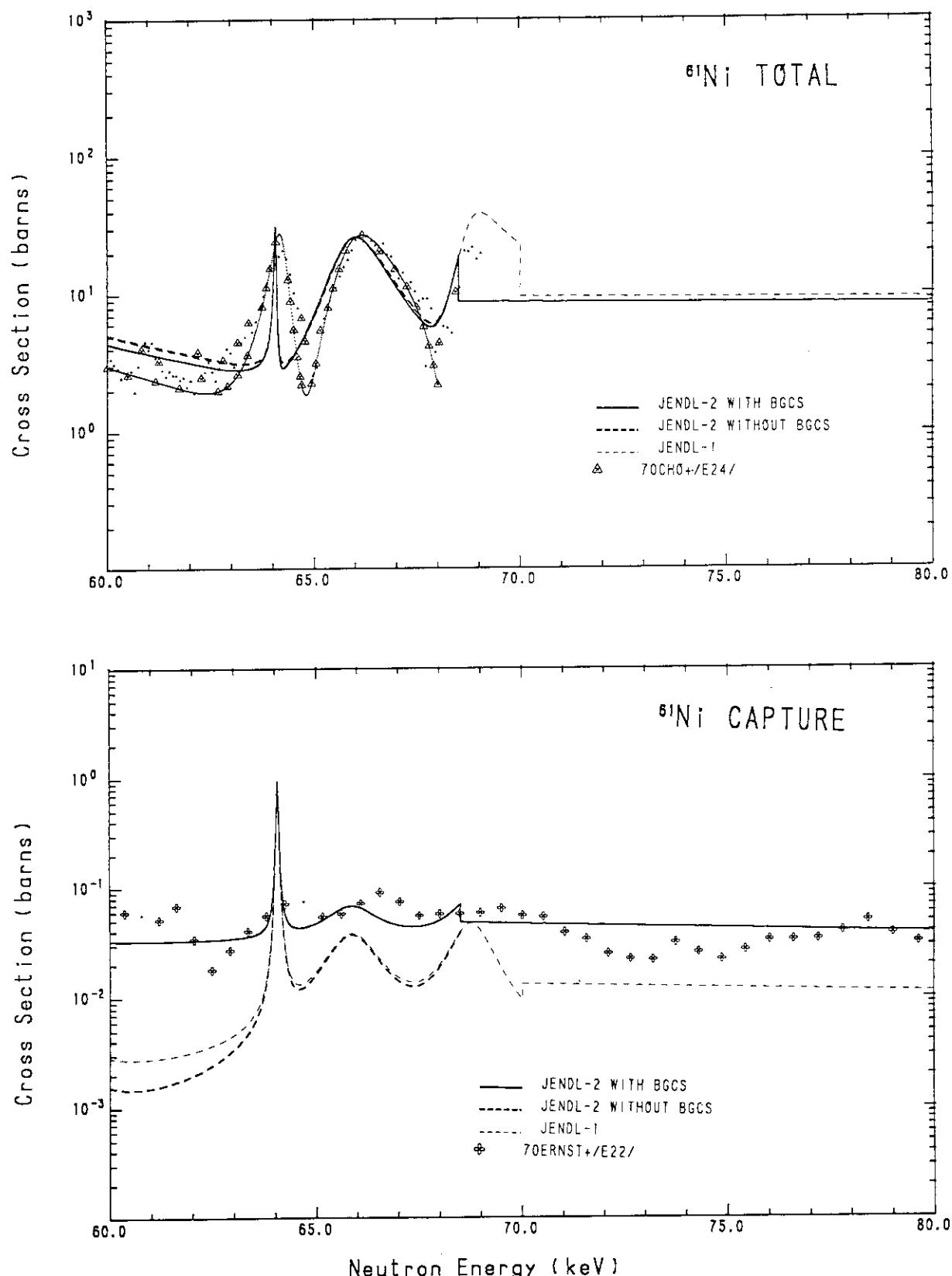


Fig. 4(e) Total and capture cross sections of ^{61}Ni in the resonance region.

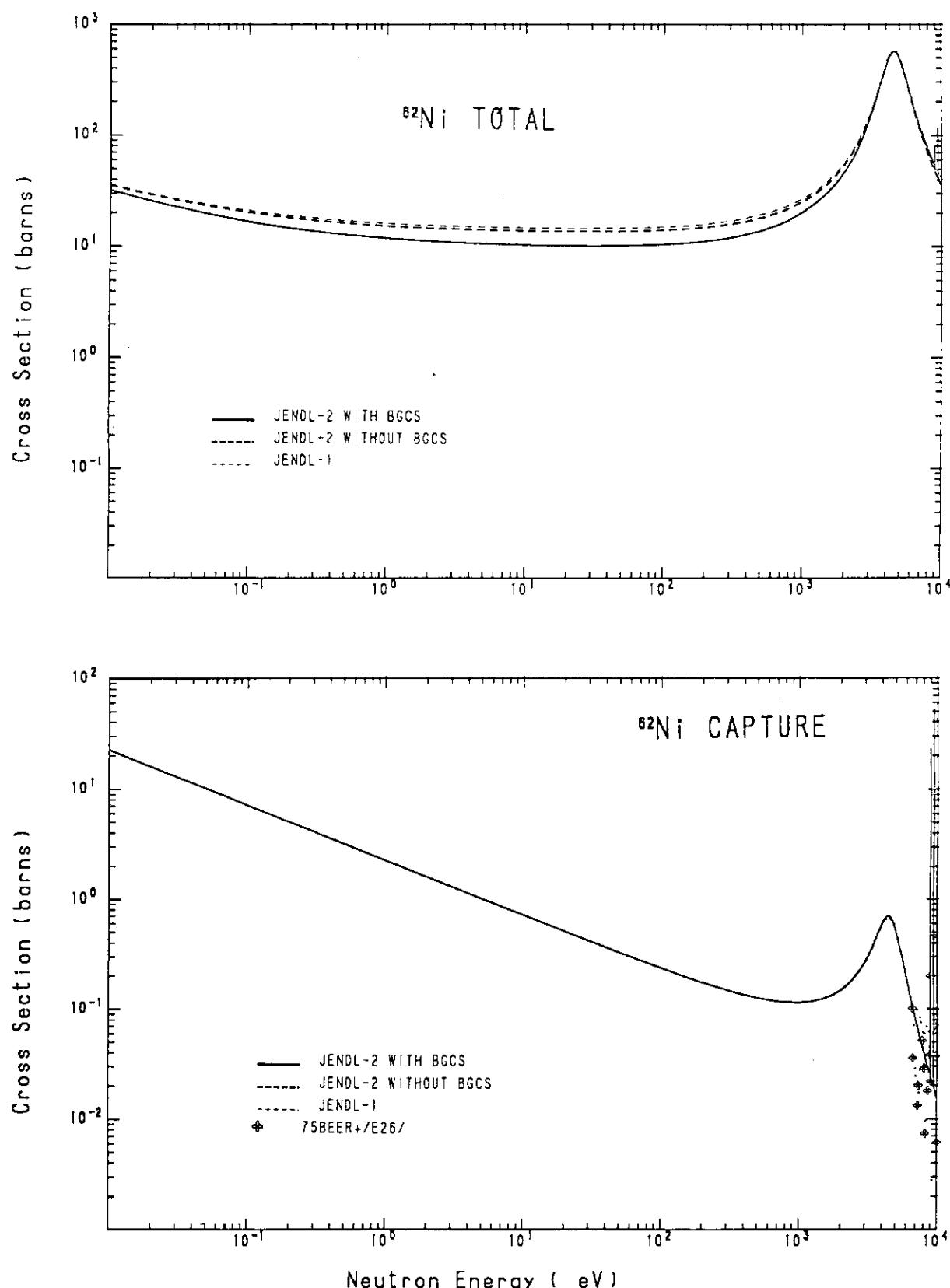


Fig. 5(a) Total and capture cross sections of ^{62}Ni in the resonance region.

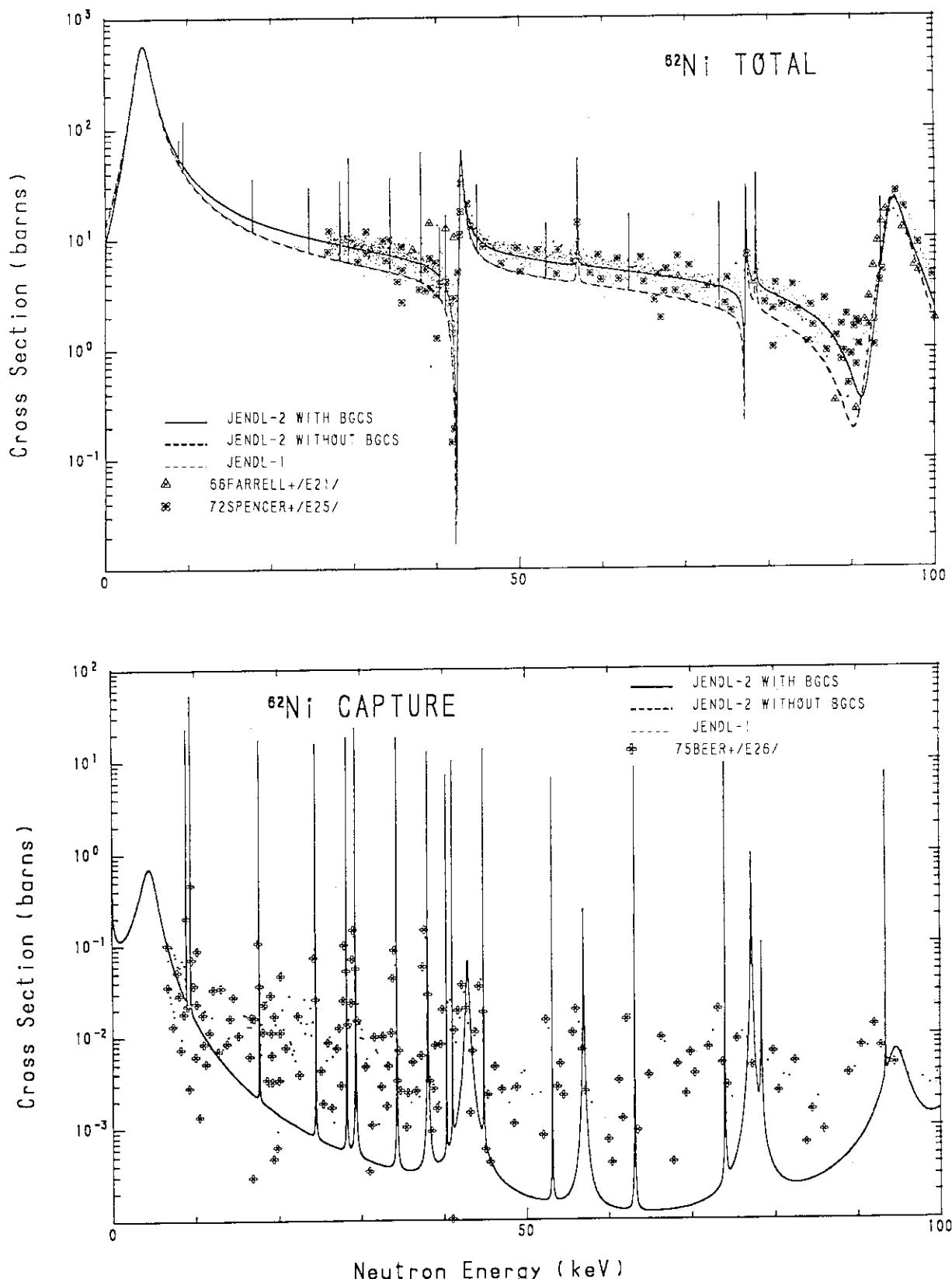


Fig. 5(b) Total and capture cross sections of ^{62}Ni in the resonance region.

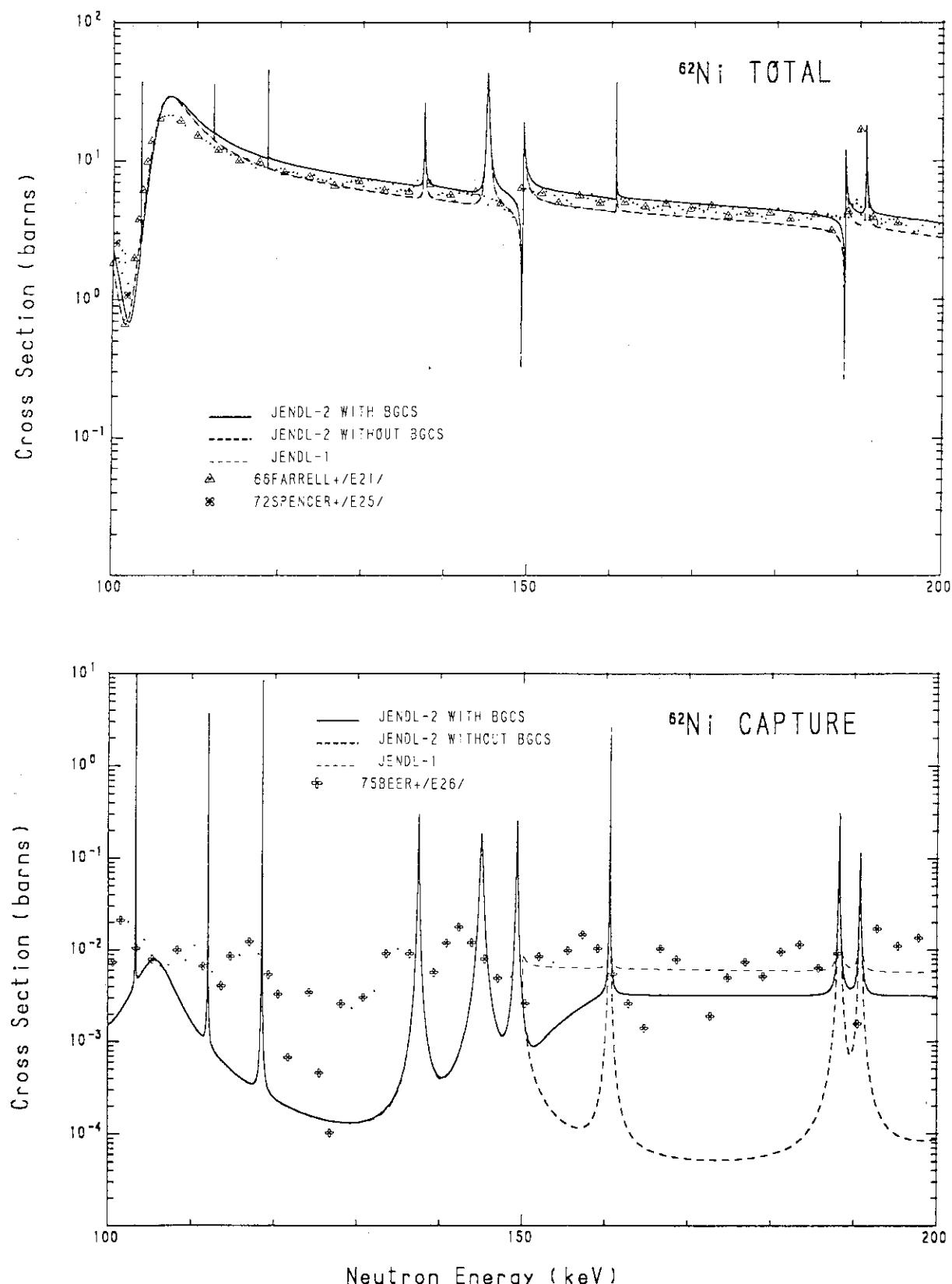


Fig. 5(c) Total and capture cross sections of ^{62}Ni in the resonance region.

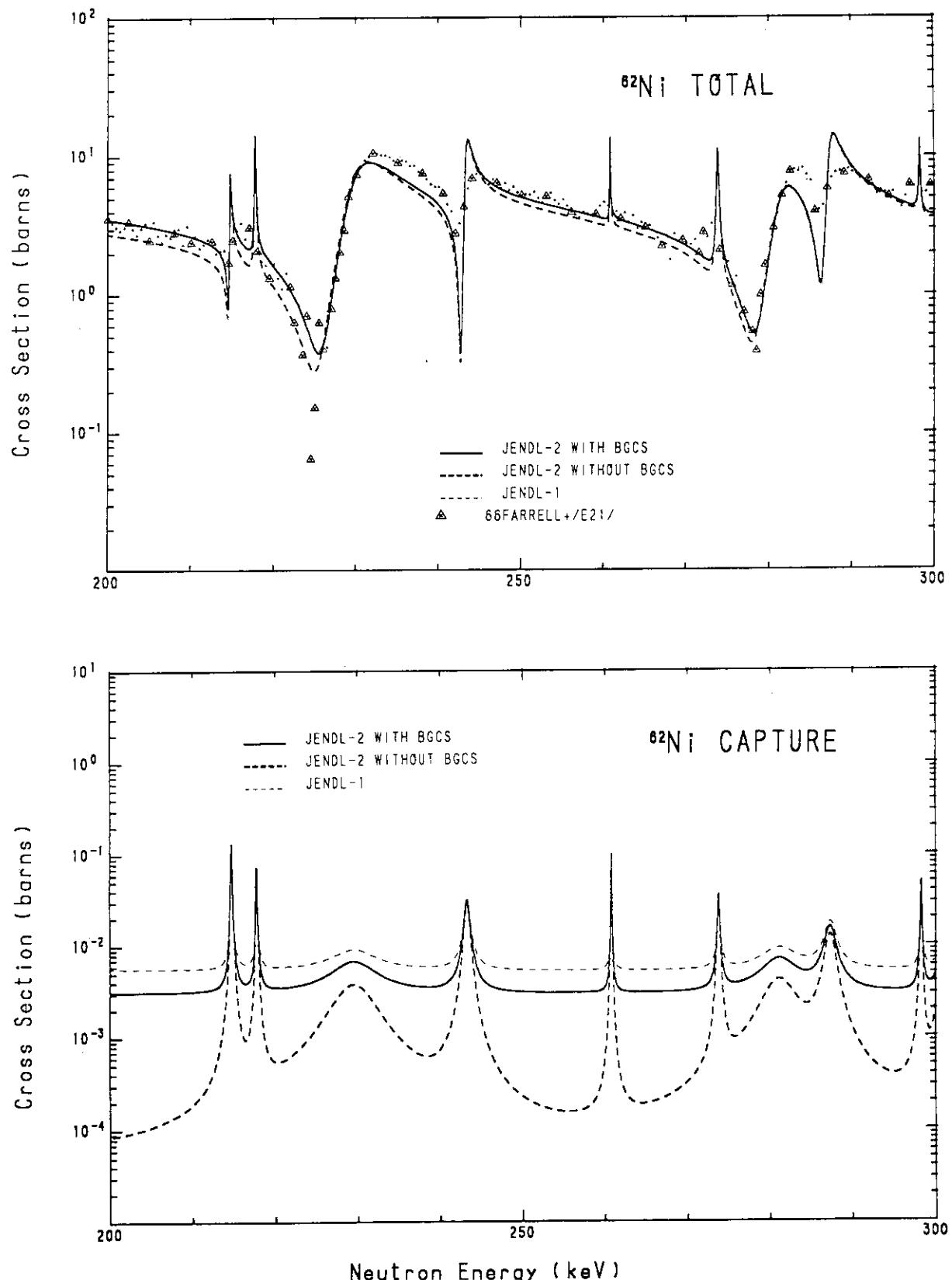


Fig. 5(d) Total and capture cross sections of ^{62}Ni in the resonance region.

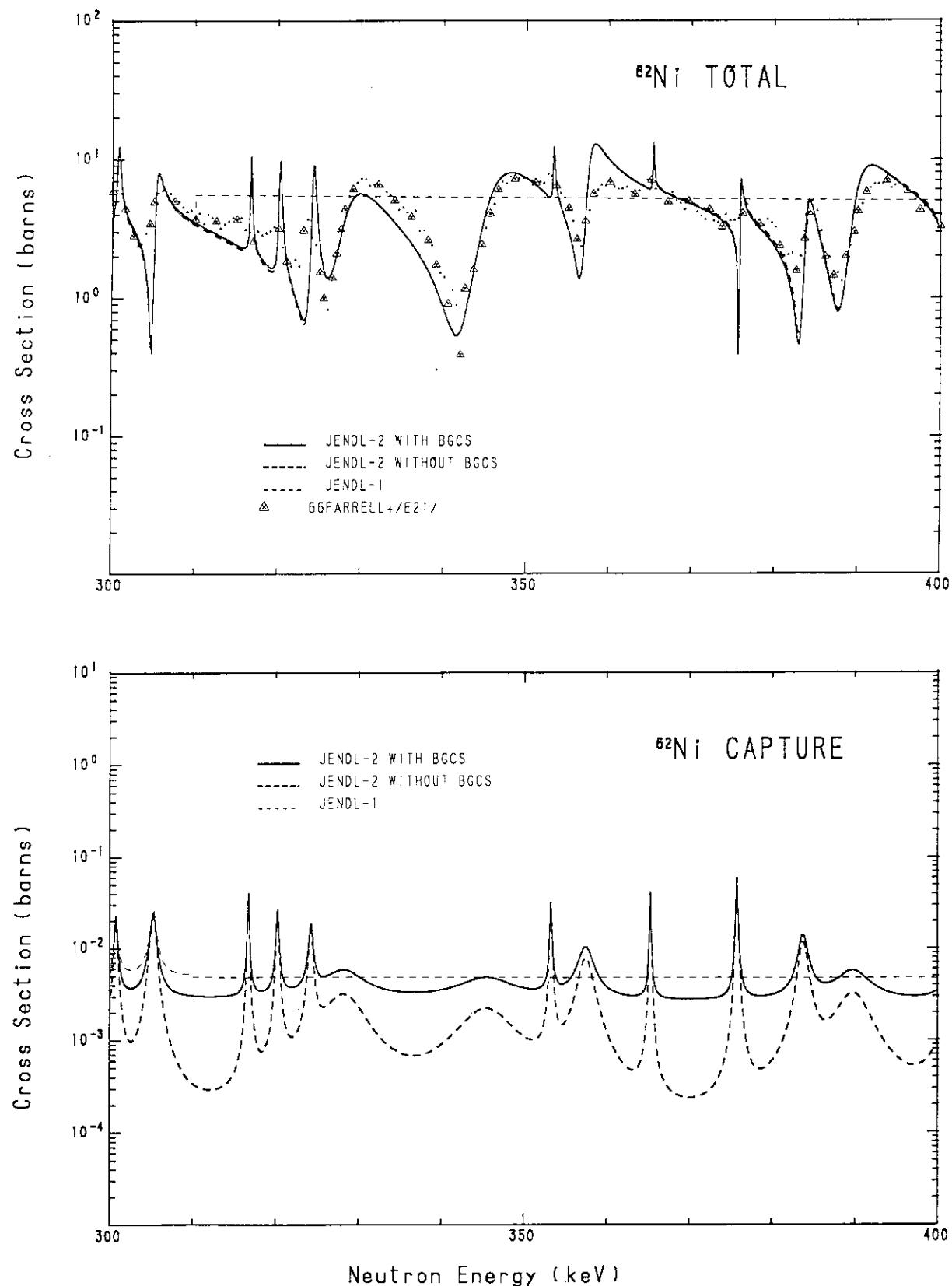


Fig. 5(e) Total and capture cross sections of ^{62}Ni in the resonance region.

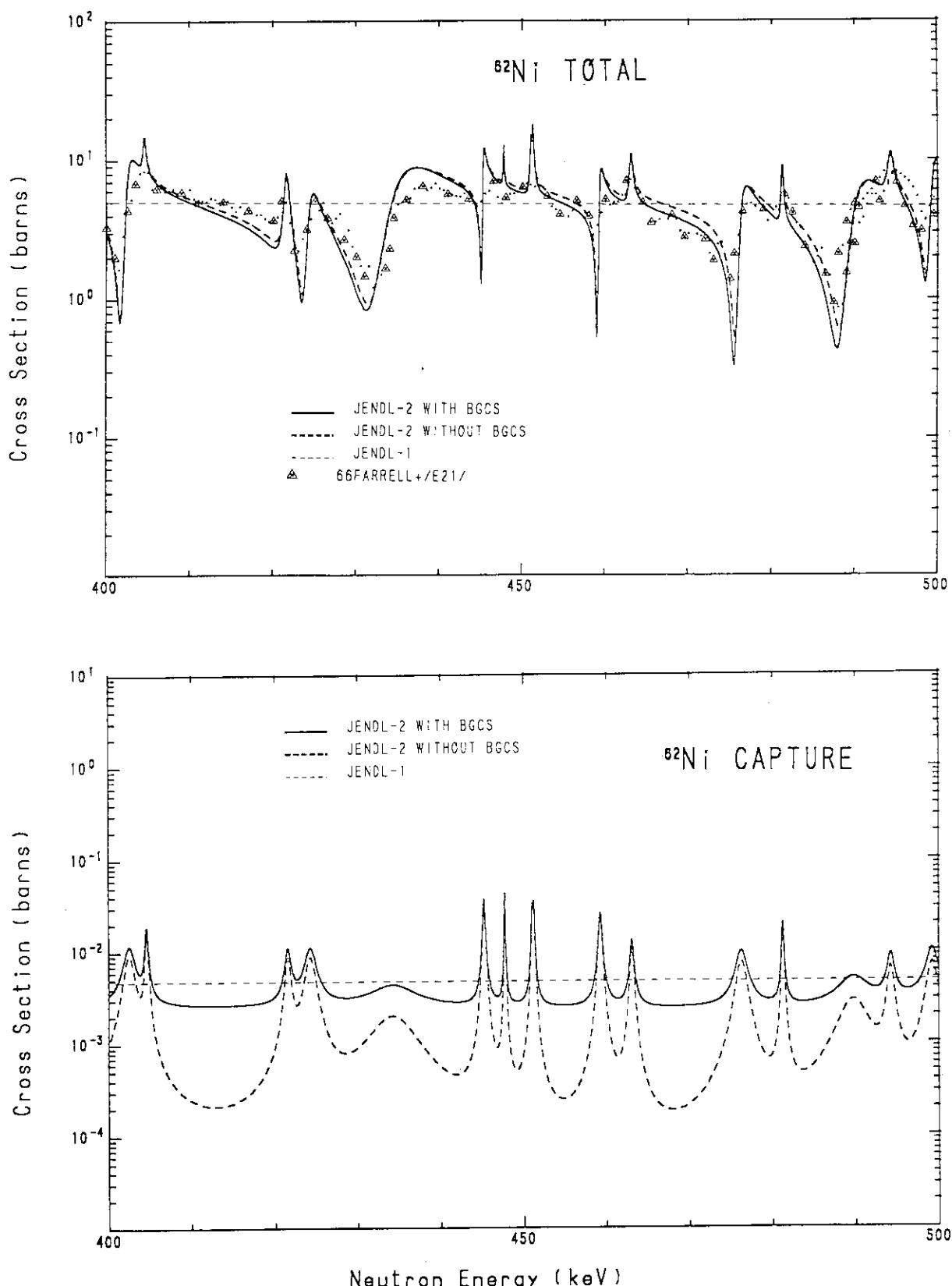


Fig. 5(f) Total and capture cross sections of ^{62}Ni in the resonance region.

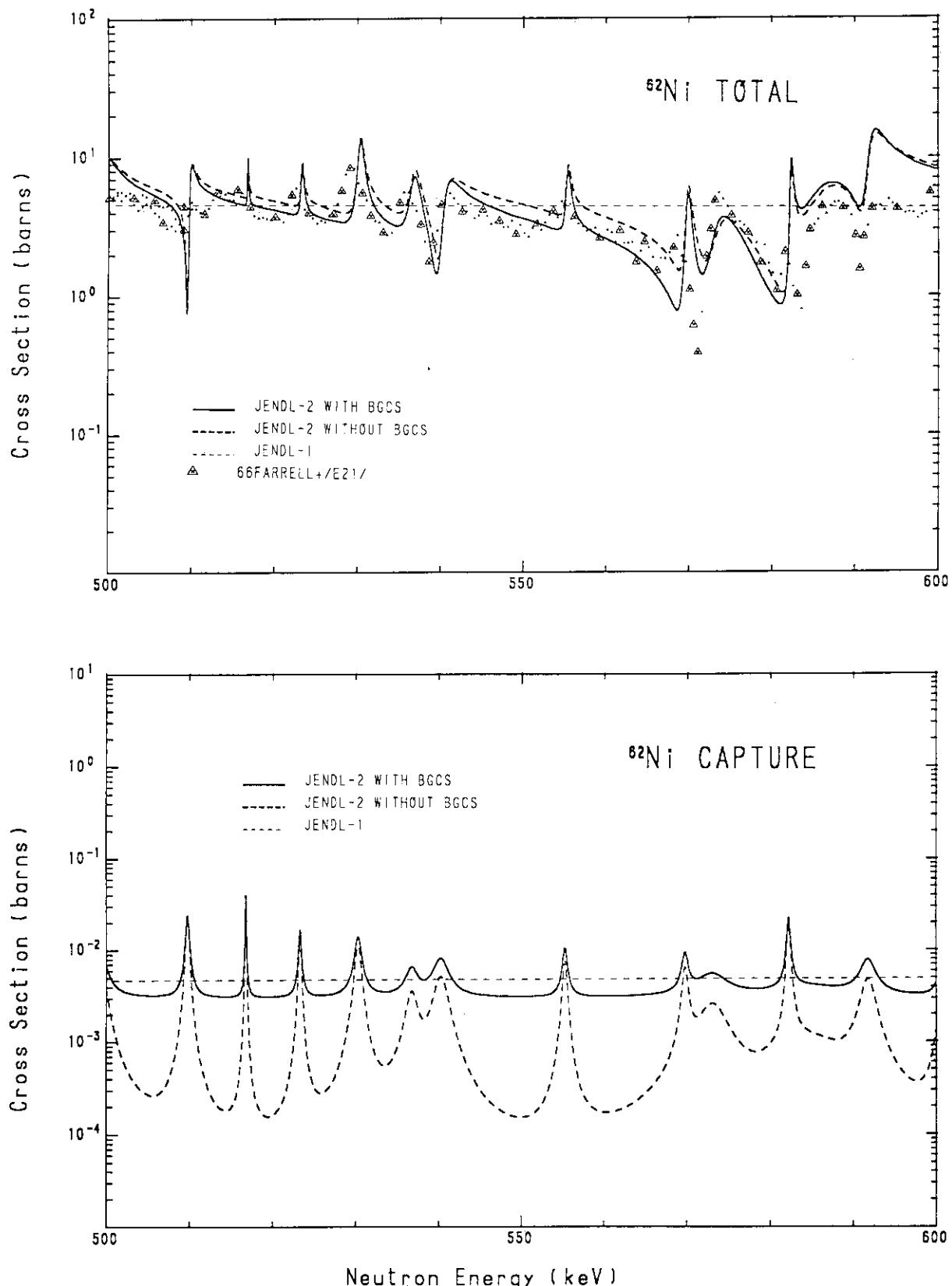


Fig. 5(g) Total and capture cross sections of ^{62}Ni in the resonance region.

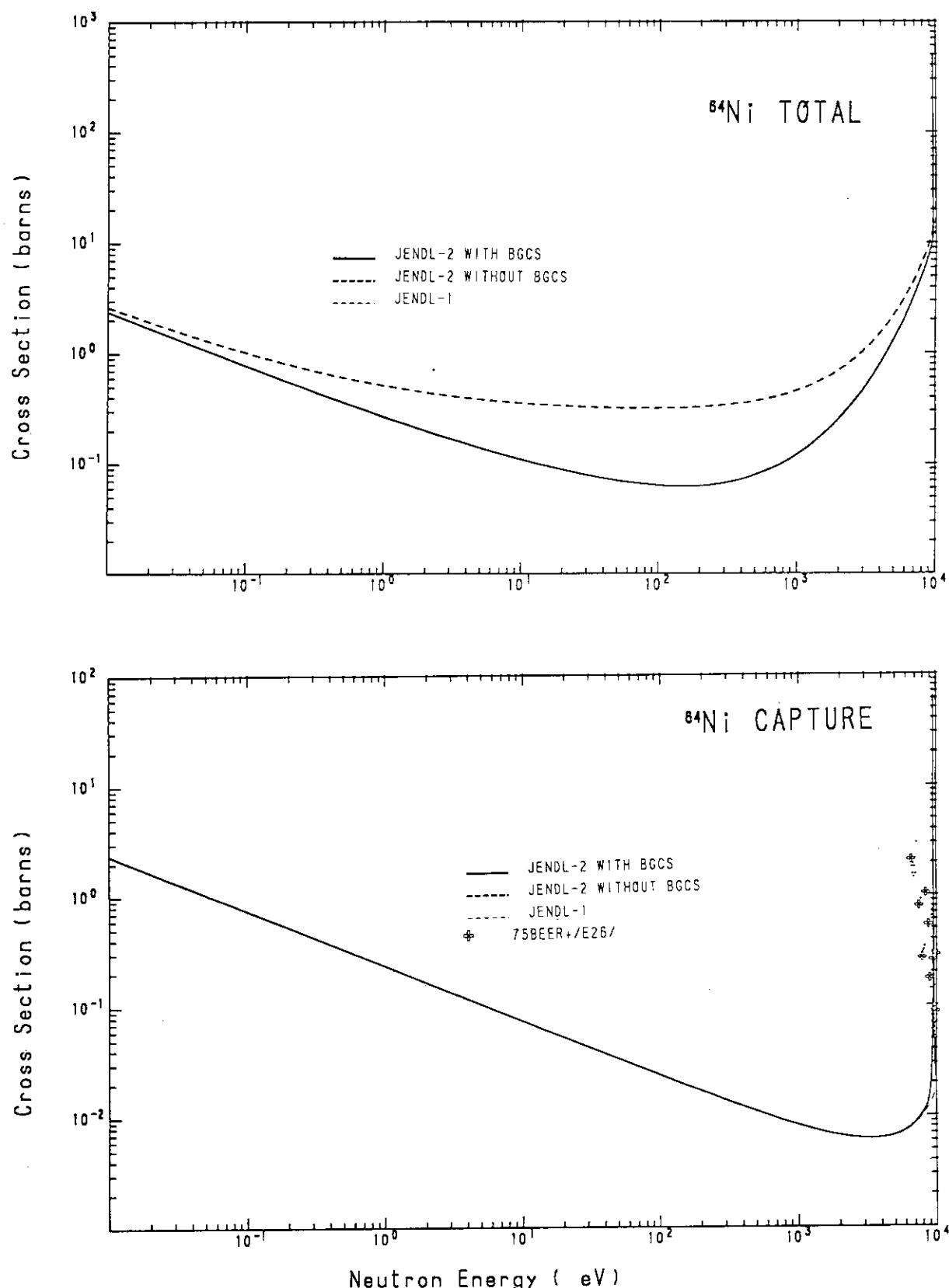


Fig. 6(a) Total and capture cross sections of ^{64}Ni in the resonance region.

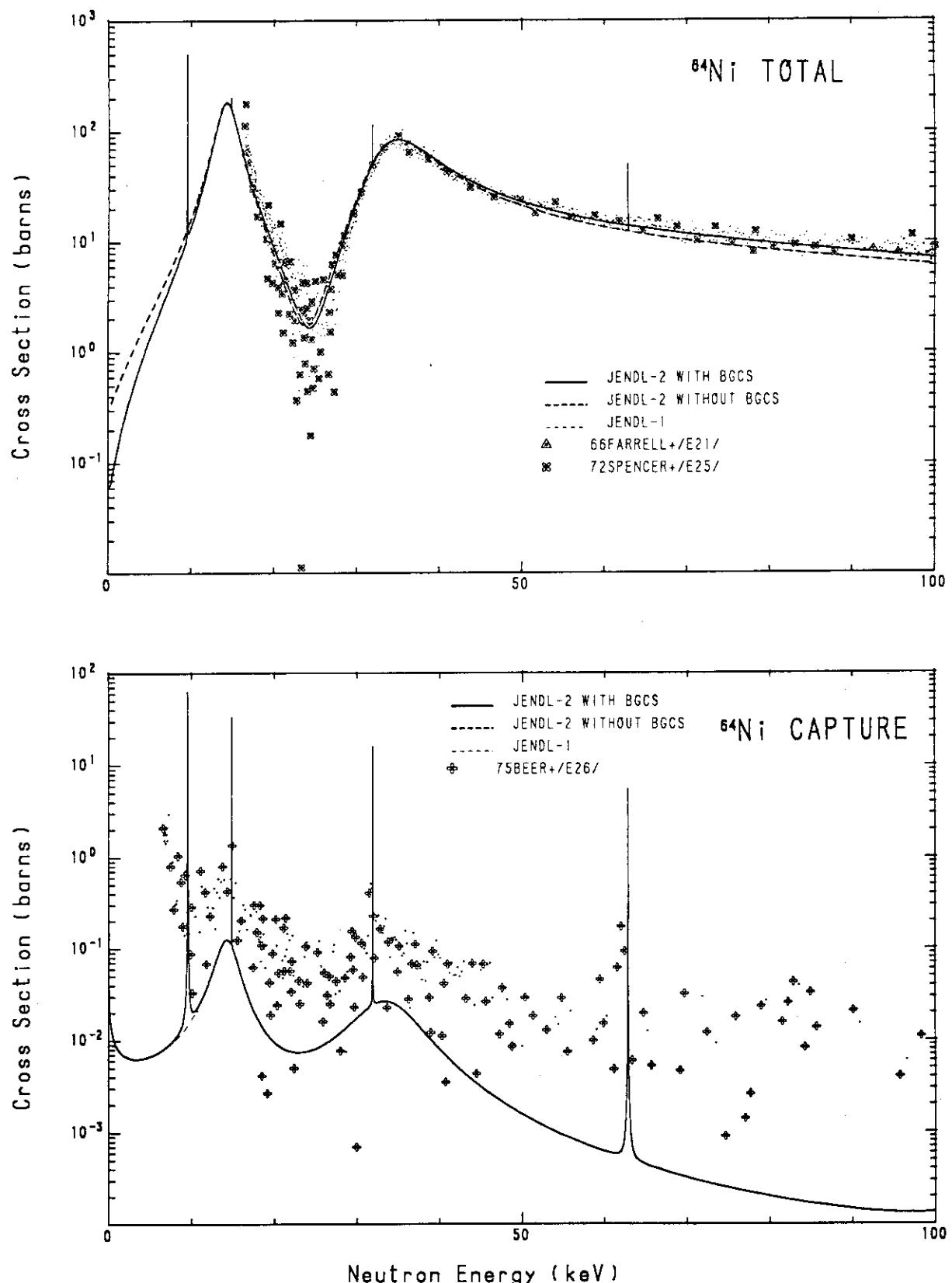


Fig. 6(b) Total and capture cross sections of ^{64}Ni in the resonance region.

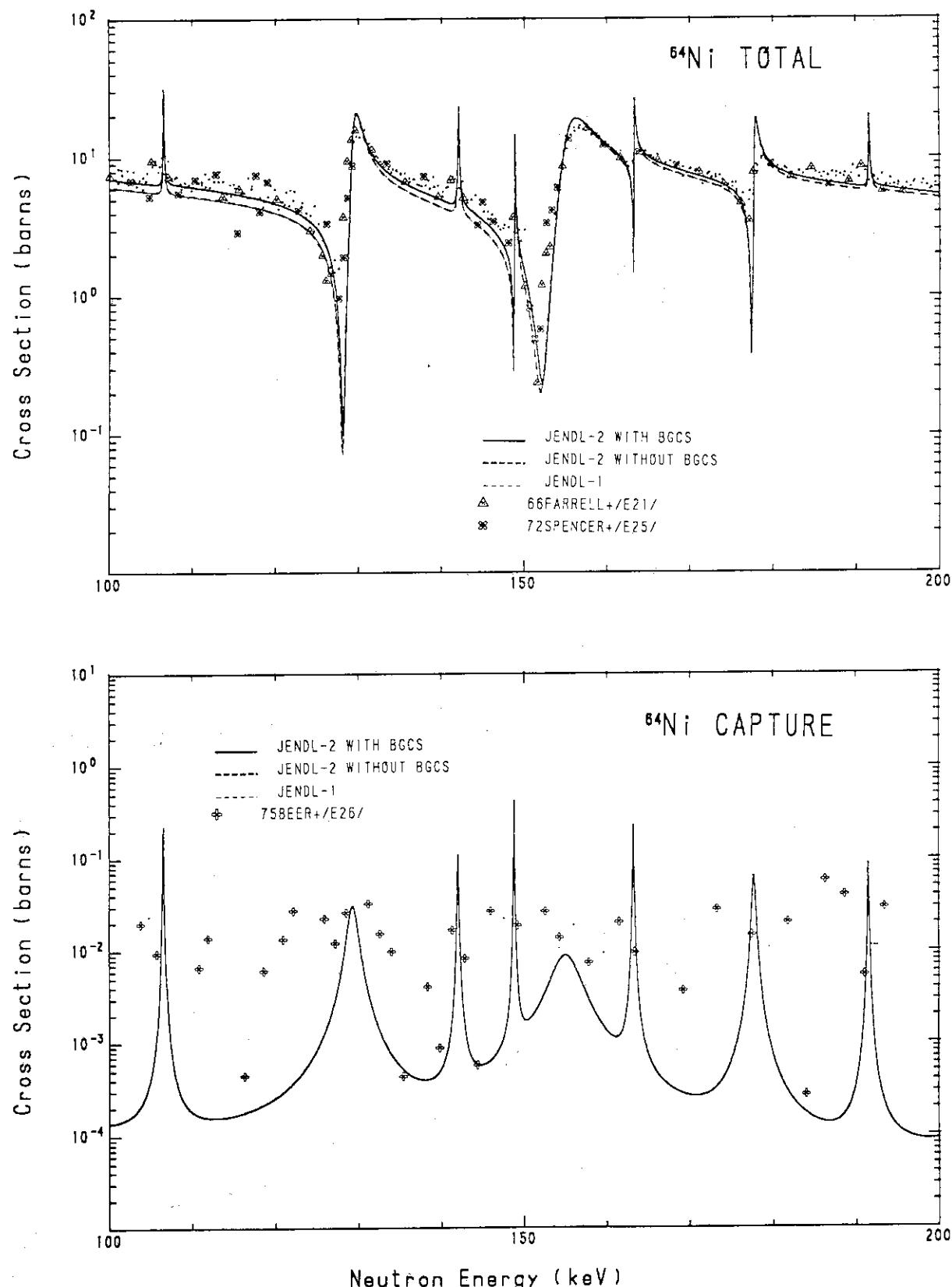


Fig. 6(c) Total and capture cross sections of ^{64}Ni in the resonance region.

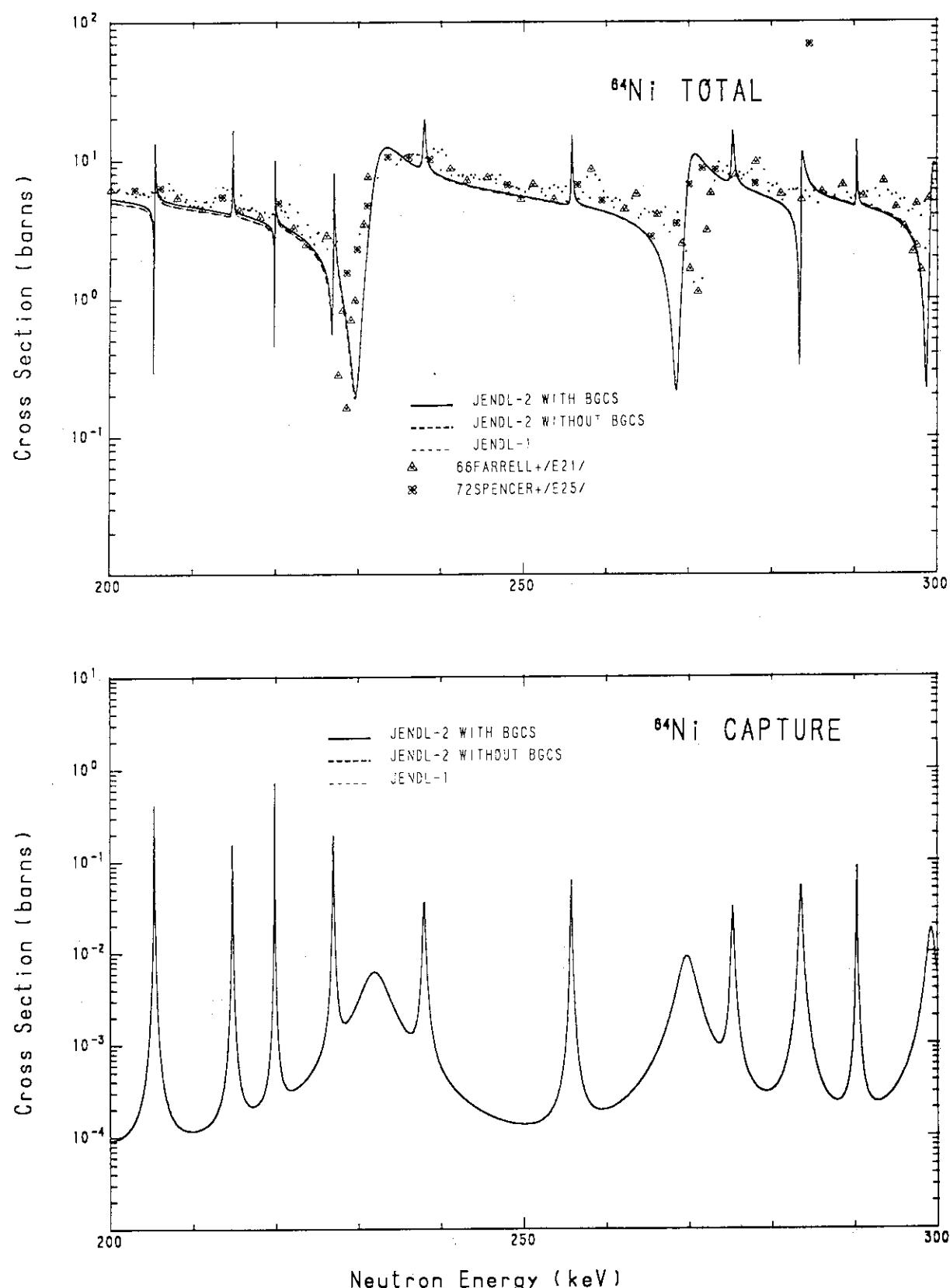


Fig. 6(d) Total and capture cross sections of ^{64}Ni in the resonance region.

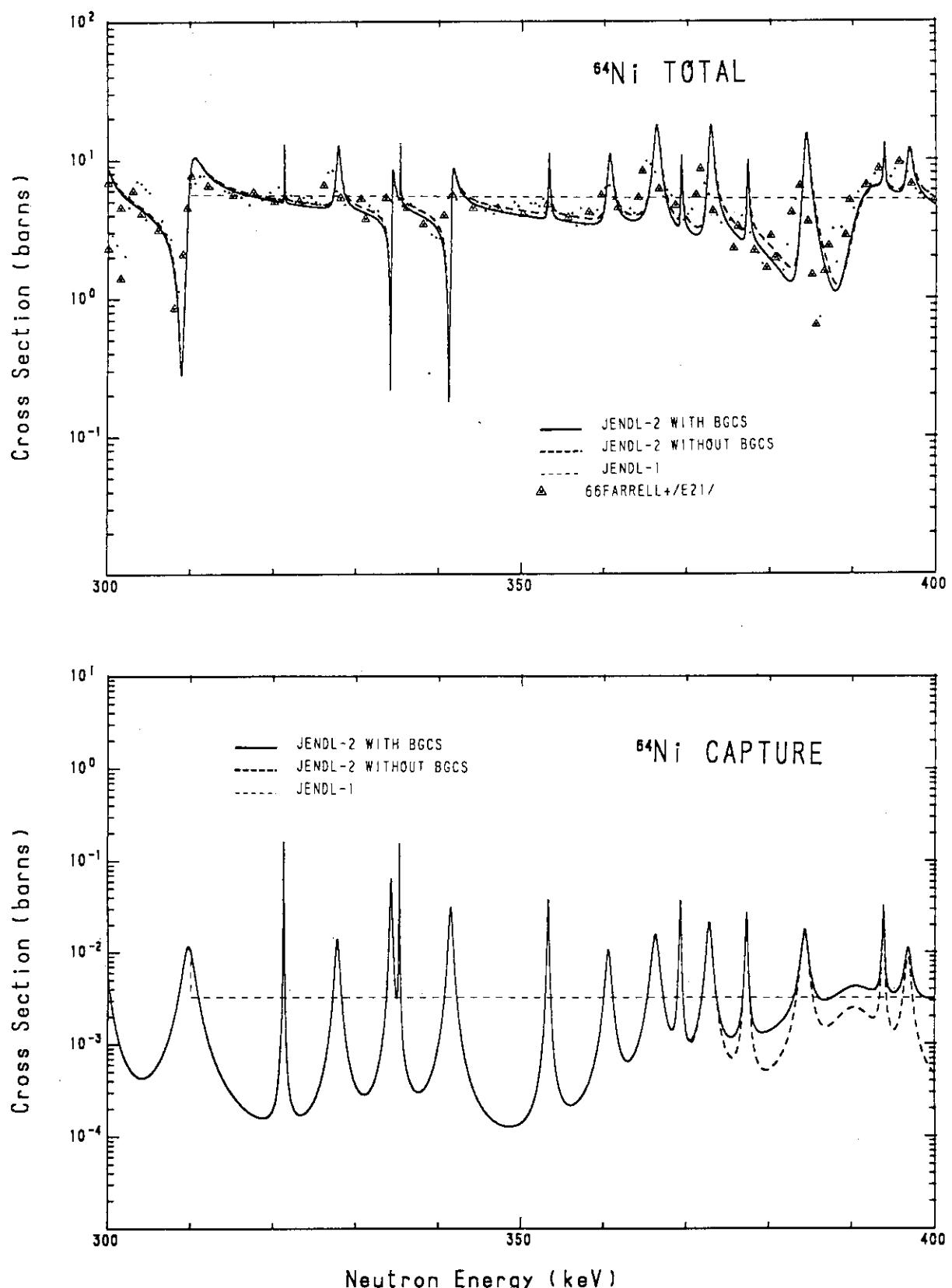


Fig. 6(e) Total and capture cross sections of ^{64}Ni in the resonance region.

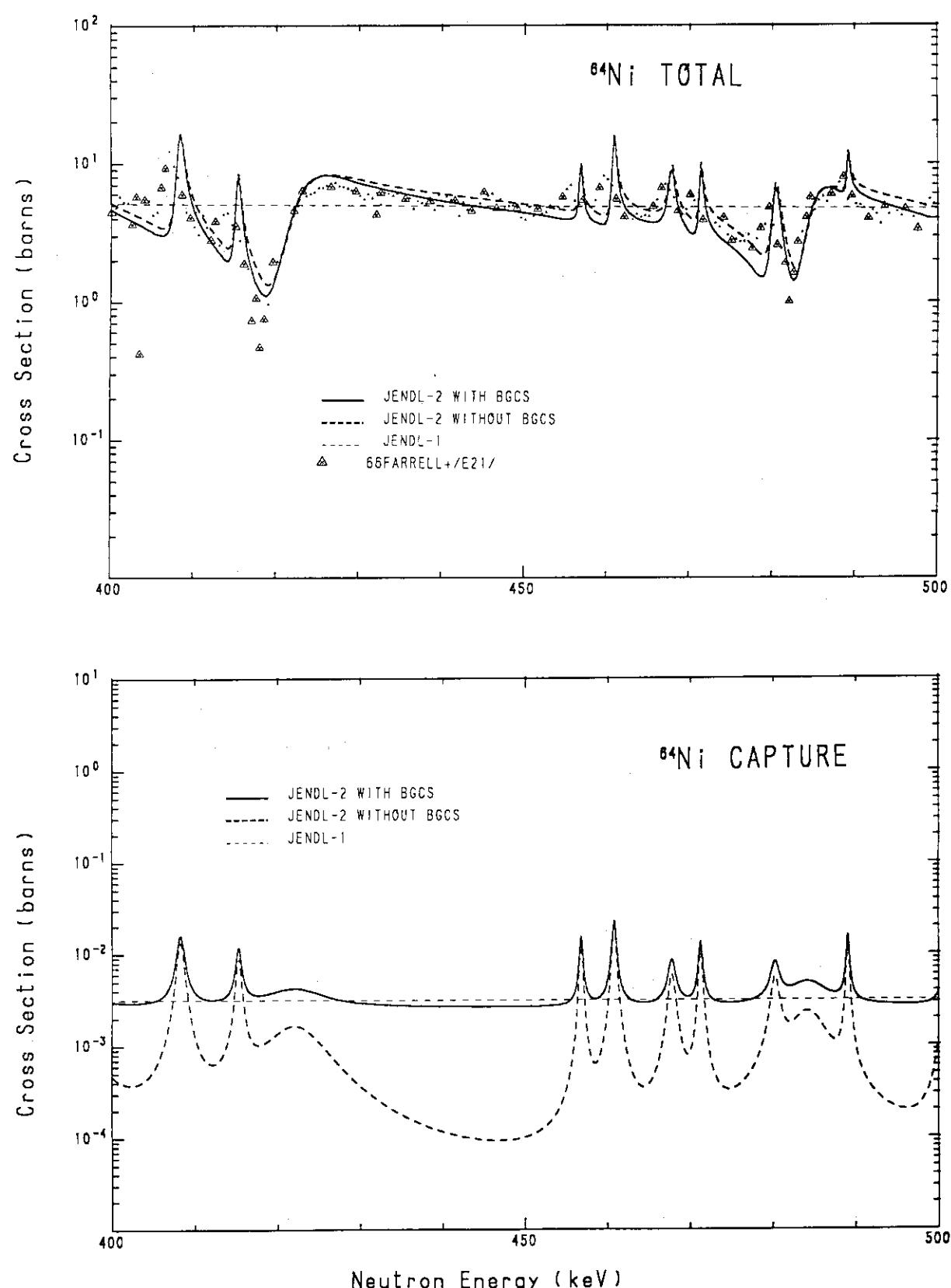


Fig. 6(f) Total and capture cross sections of ^{64}Ni in the resonance region.

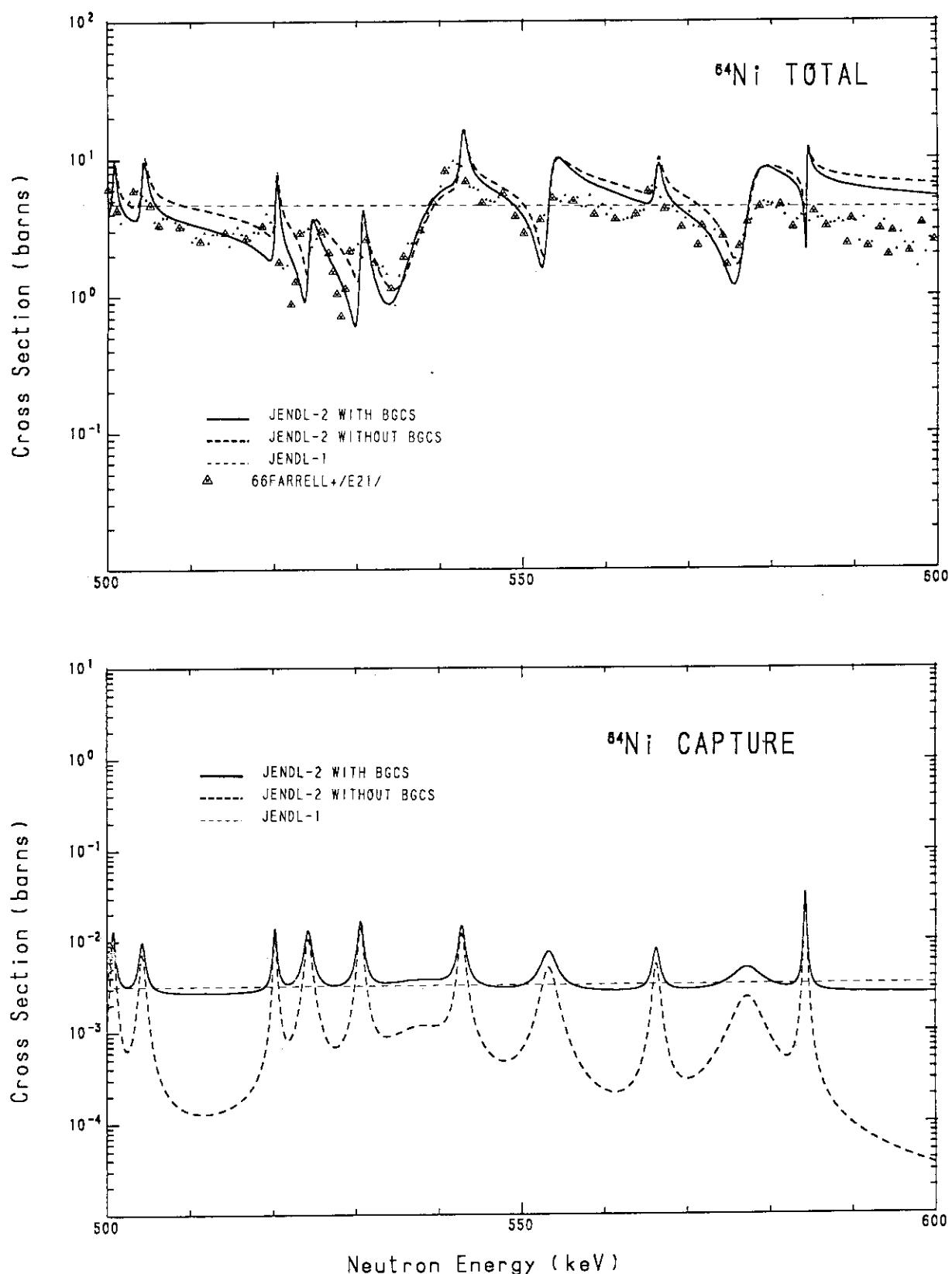


Fig. 6(g) Total and capture cross sections of ^{64}Ni in the resonance region.

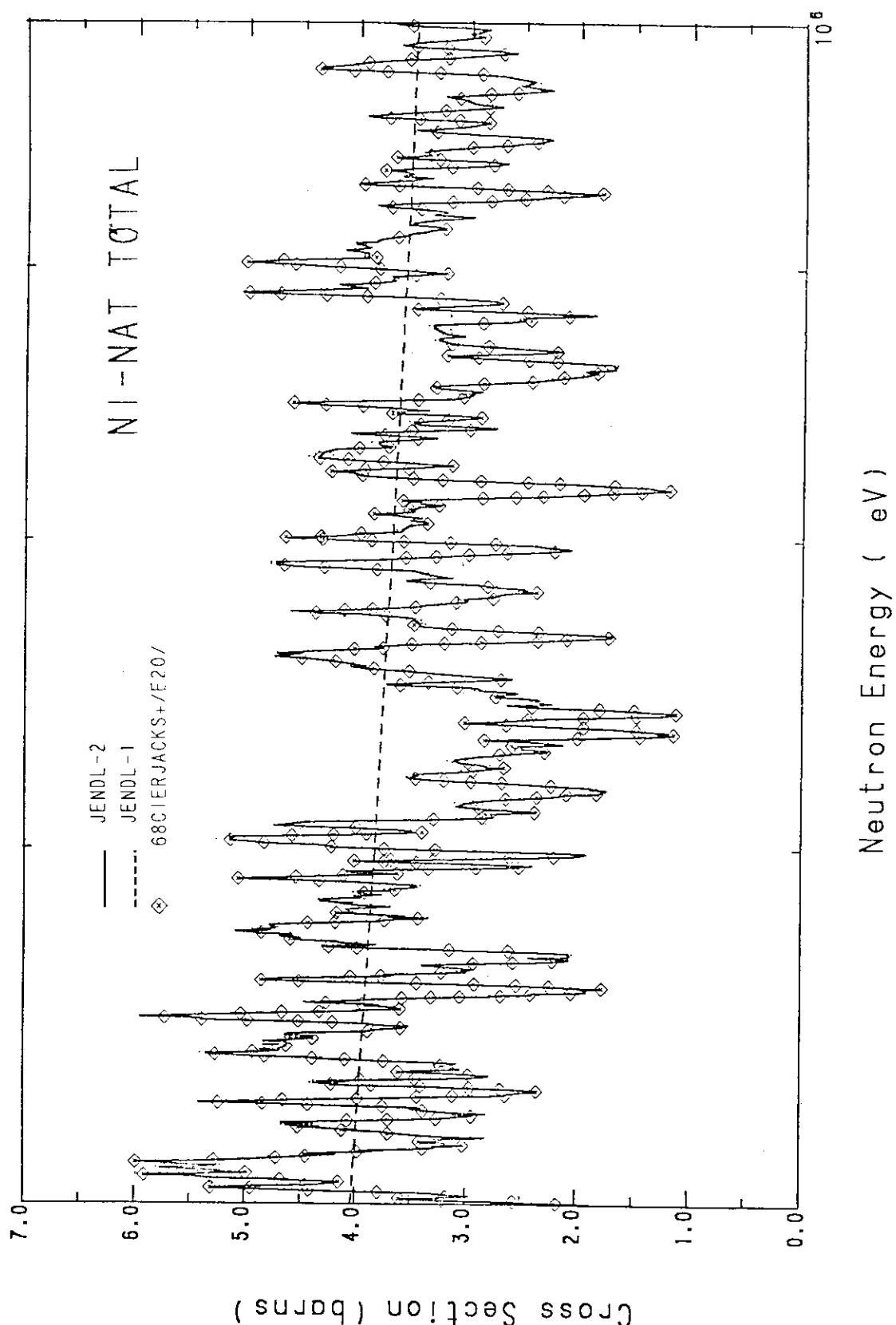


Fig. 7(a) Total cross sections of natural nickel.

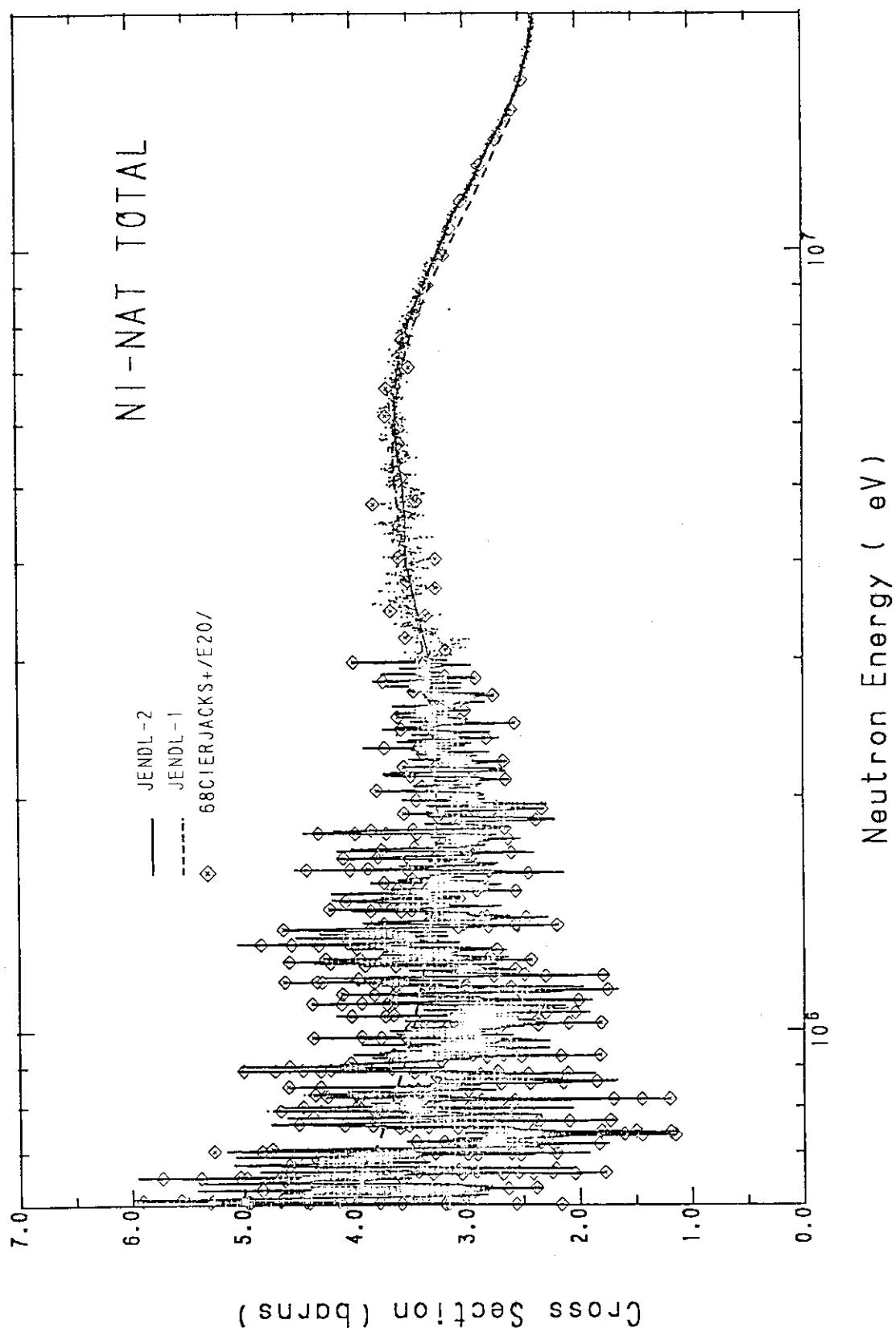
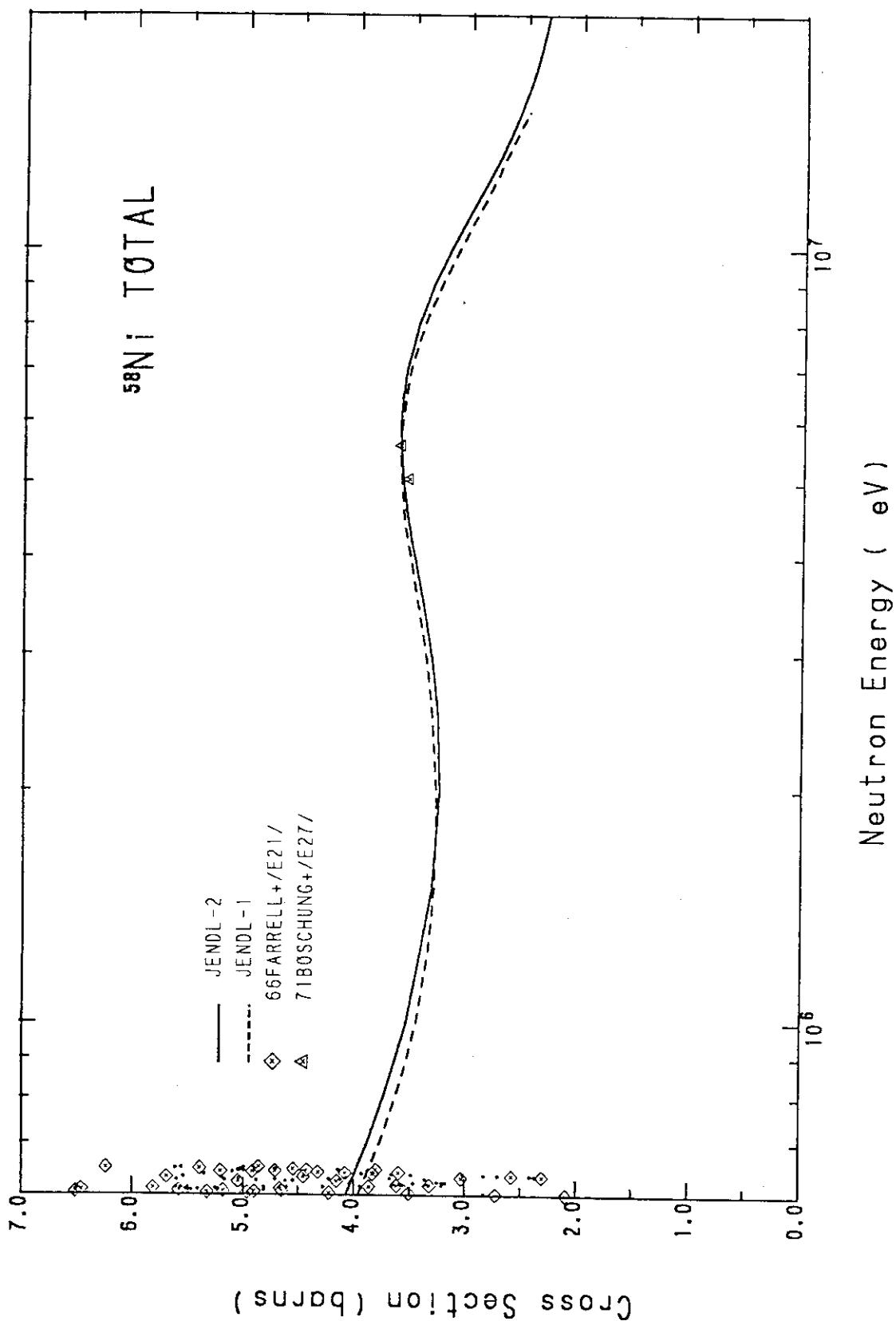


Fig. 7(b) Total cross sections of natural nickel.

Fig. 8 Total cross section of ^{58}Ni .

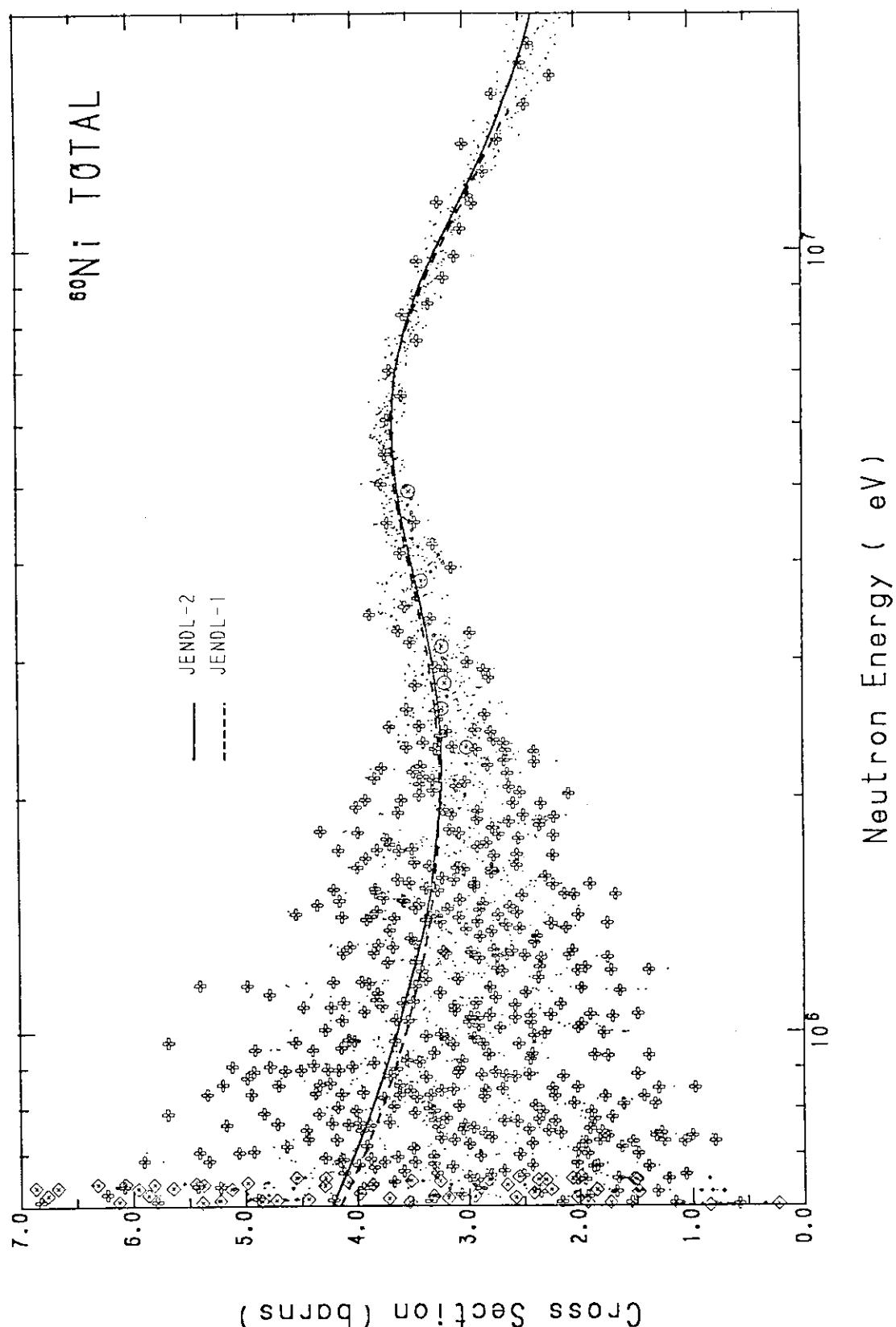


Fig. 9 Total cross section of ^{60}Ni .

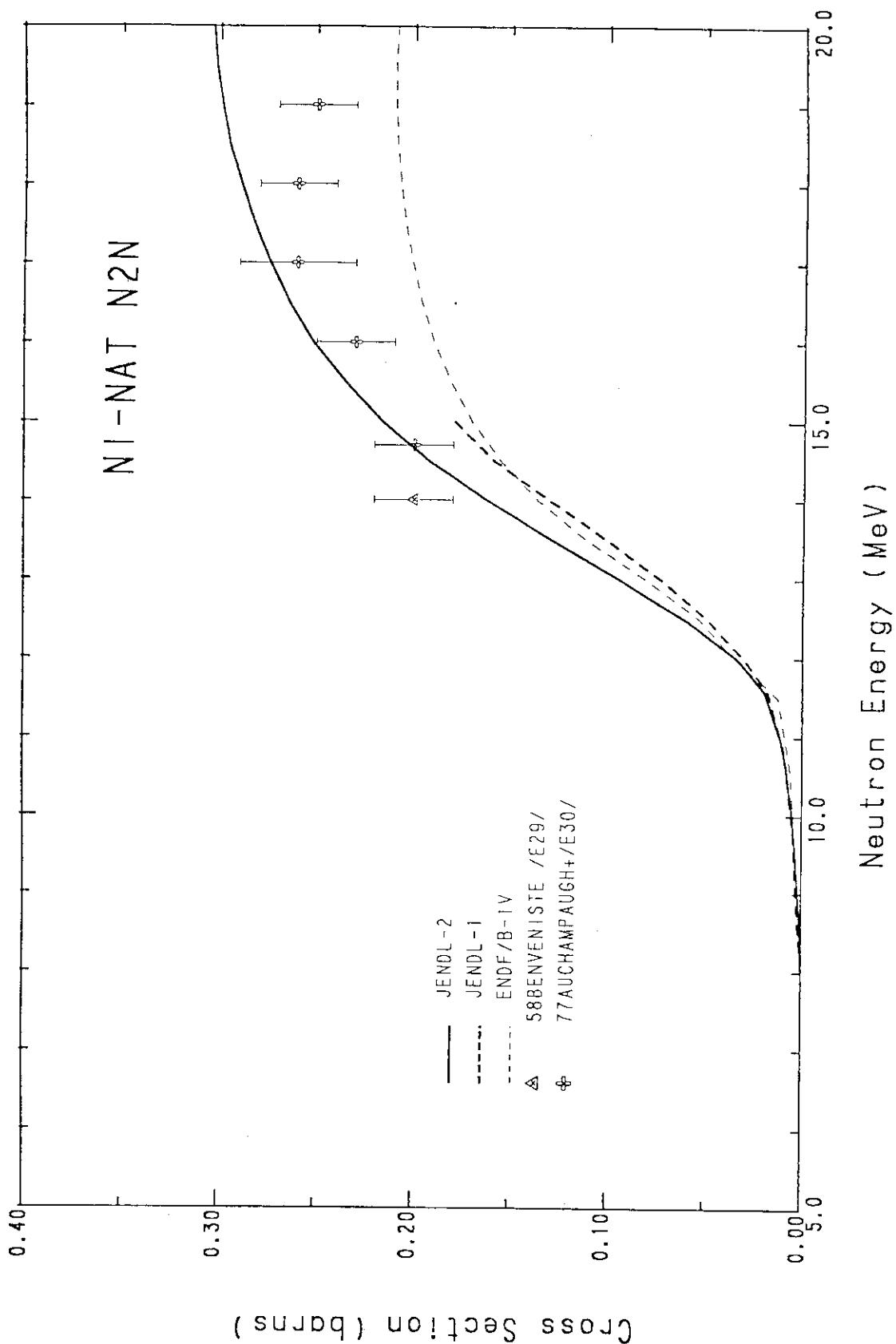


Fig. 10 (n,2n) reaction cross section of natural nickel.

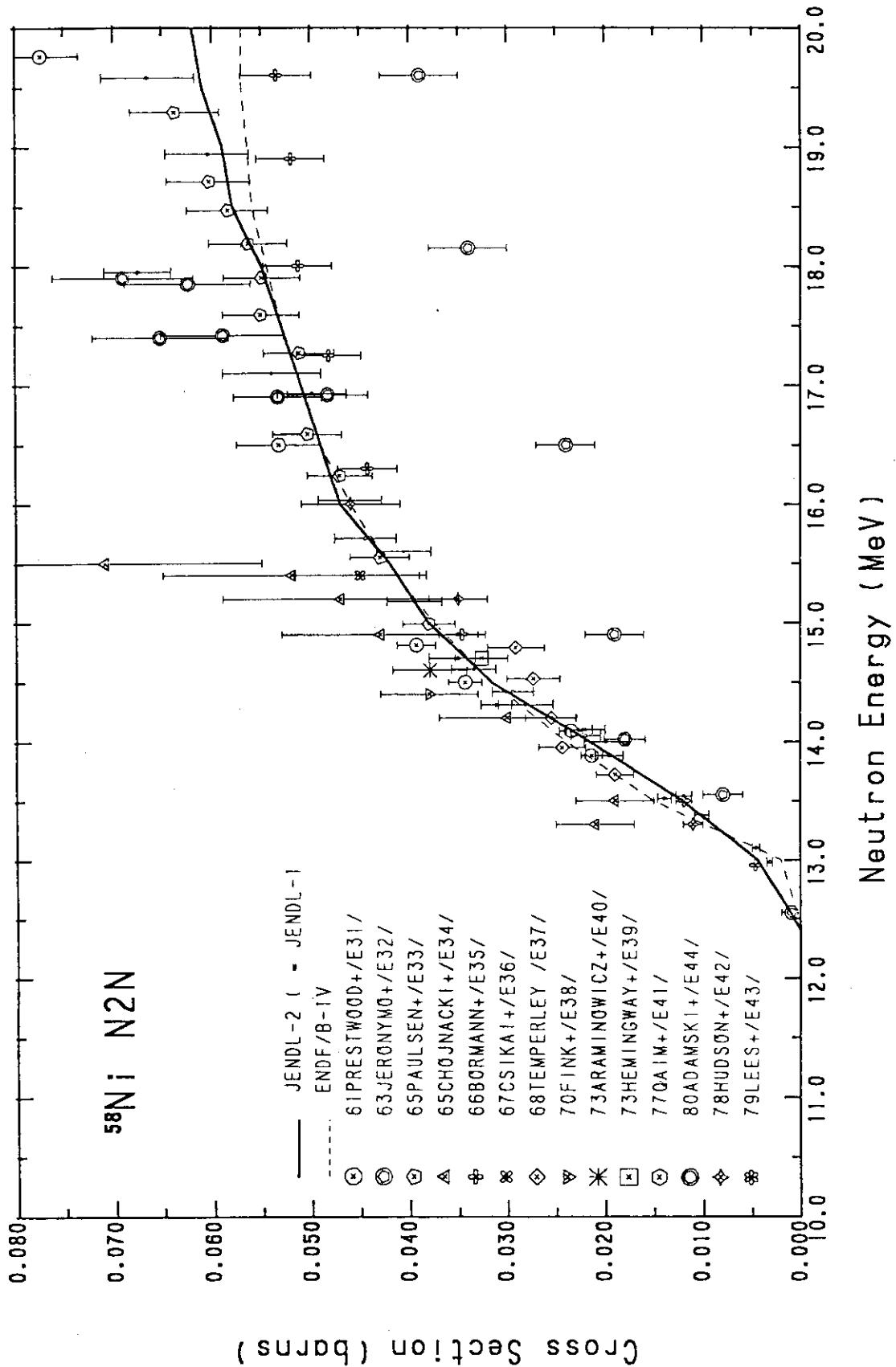
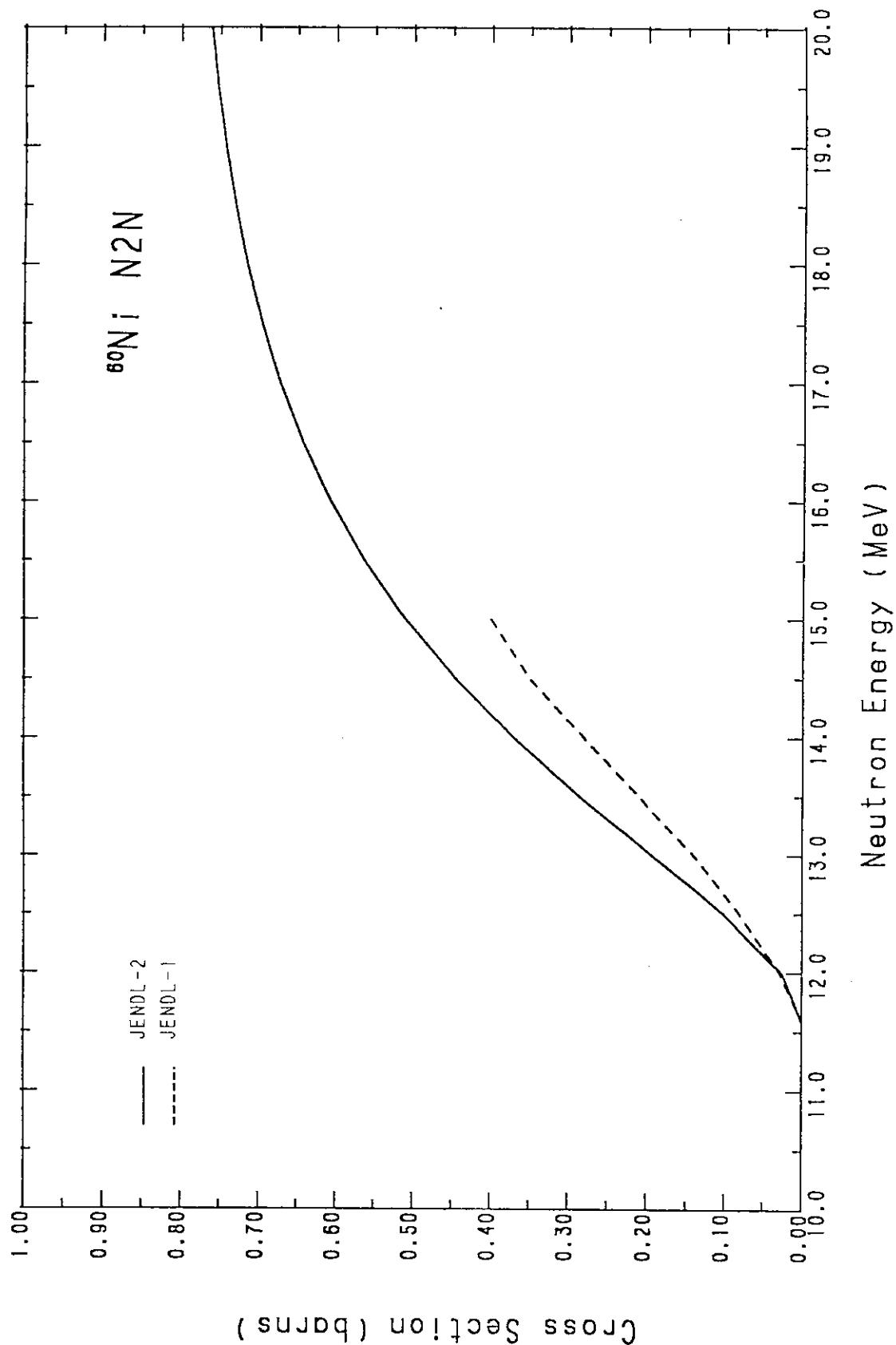


Fig. 11 (n,2n) reaction cross section of ^{58}Ni .

Fig. 12 ($n,2n$) reaction cross section of ^{60}Ni .

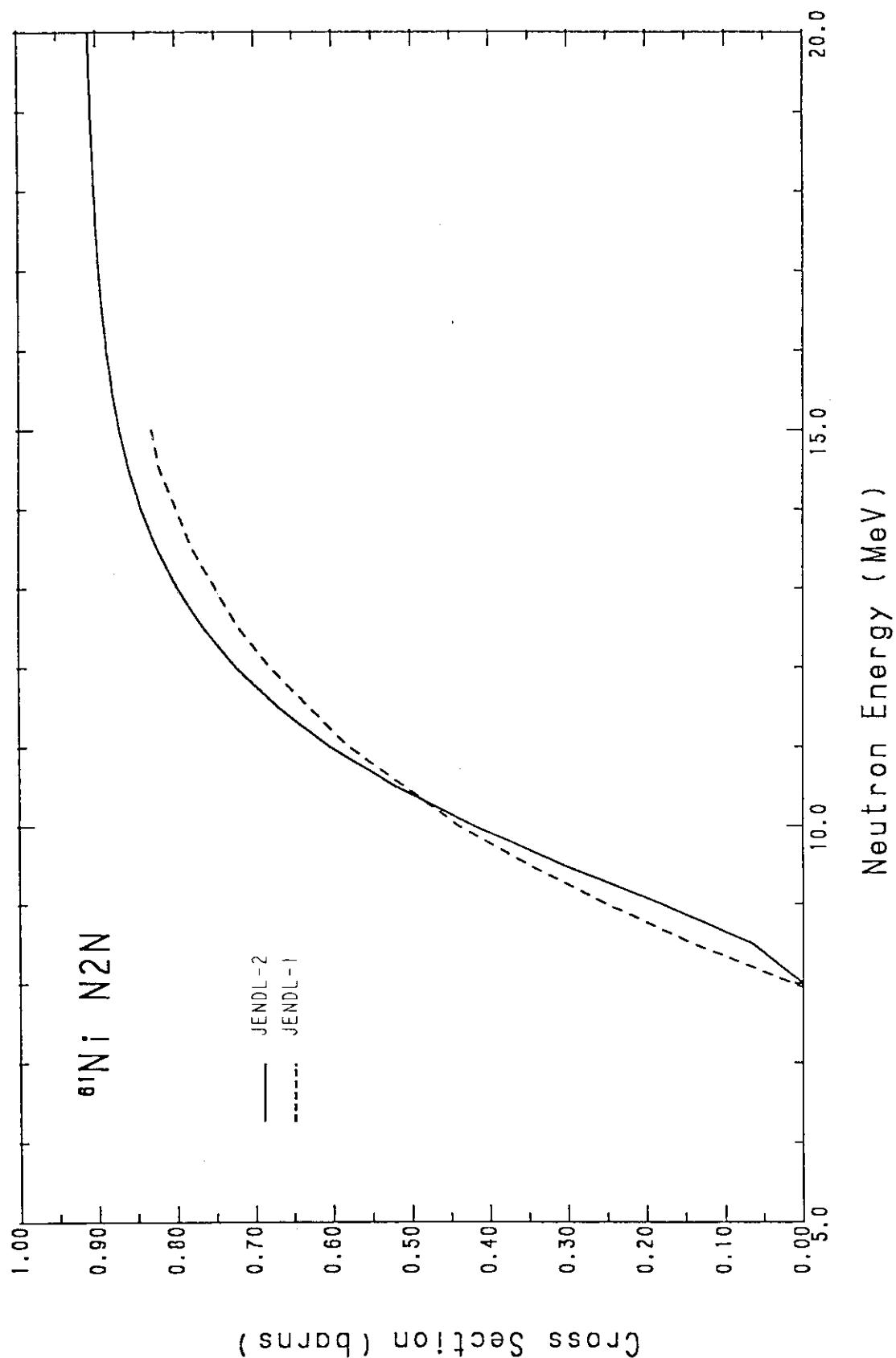


Fig. 13 $(n,2n)$ reaction cross section of ^{61}Ni .

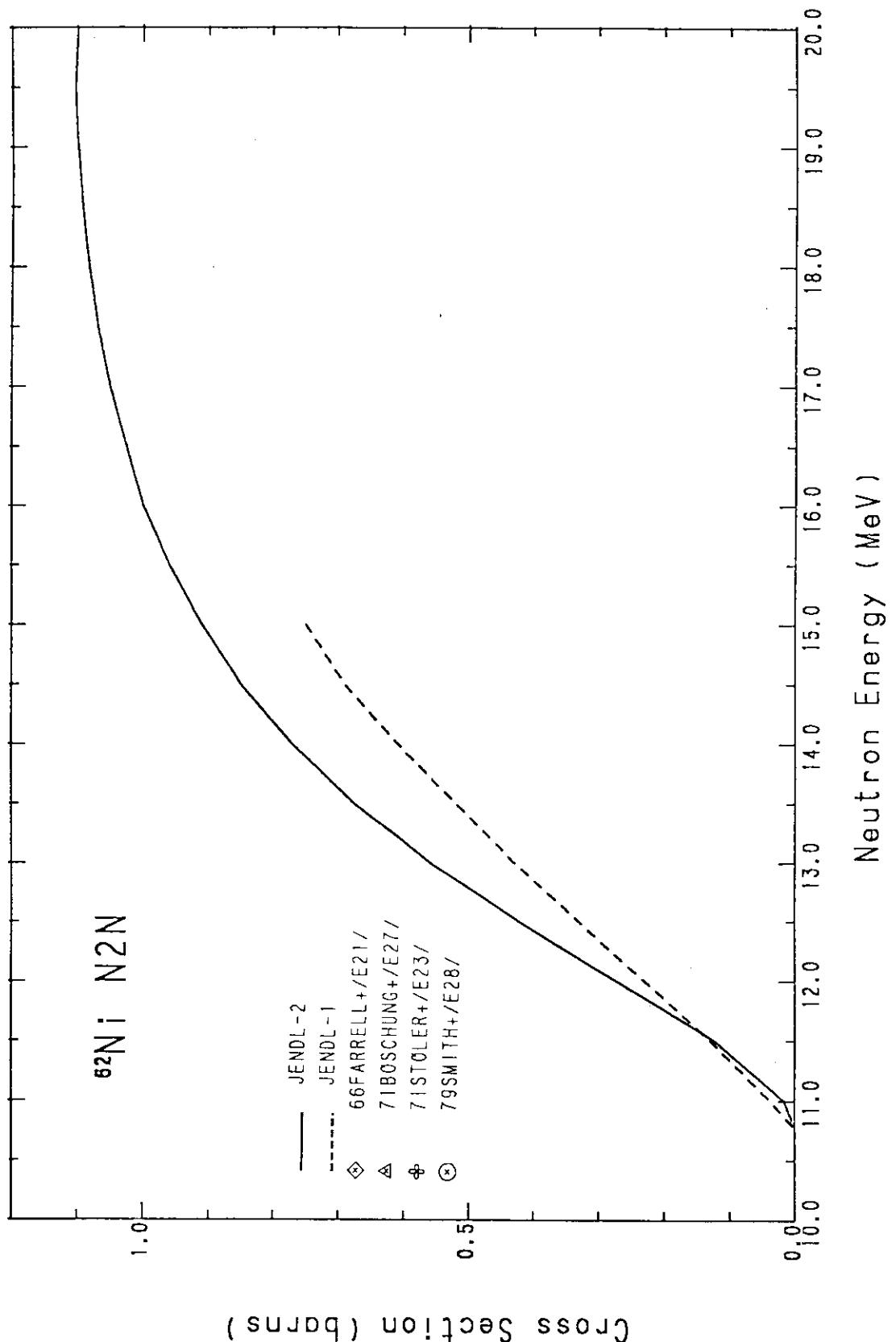


Fig. 14 (n,2n) reaction cross section of ^{62}Ni .

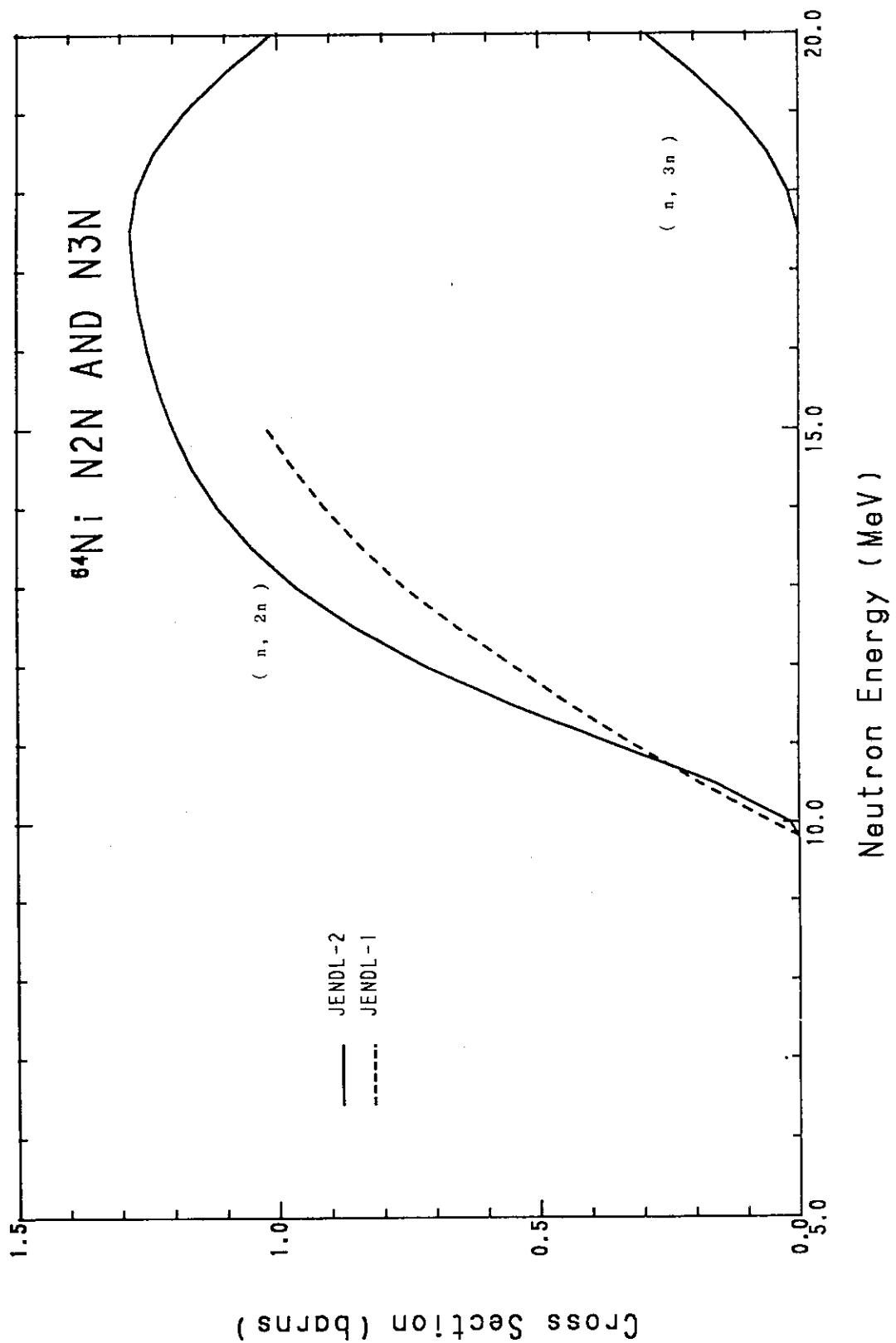


Fig. 15 $(n, 2n)$ and $(n, 3n)$ reaction cross sections of ^{64}Ni .

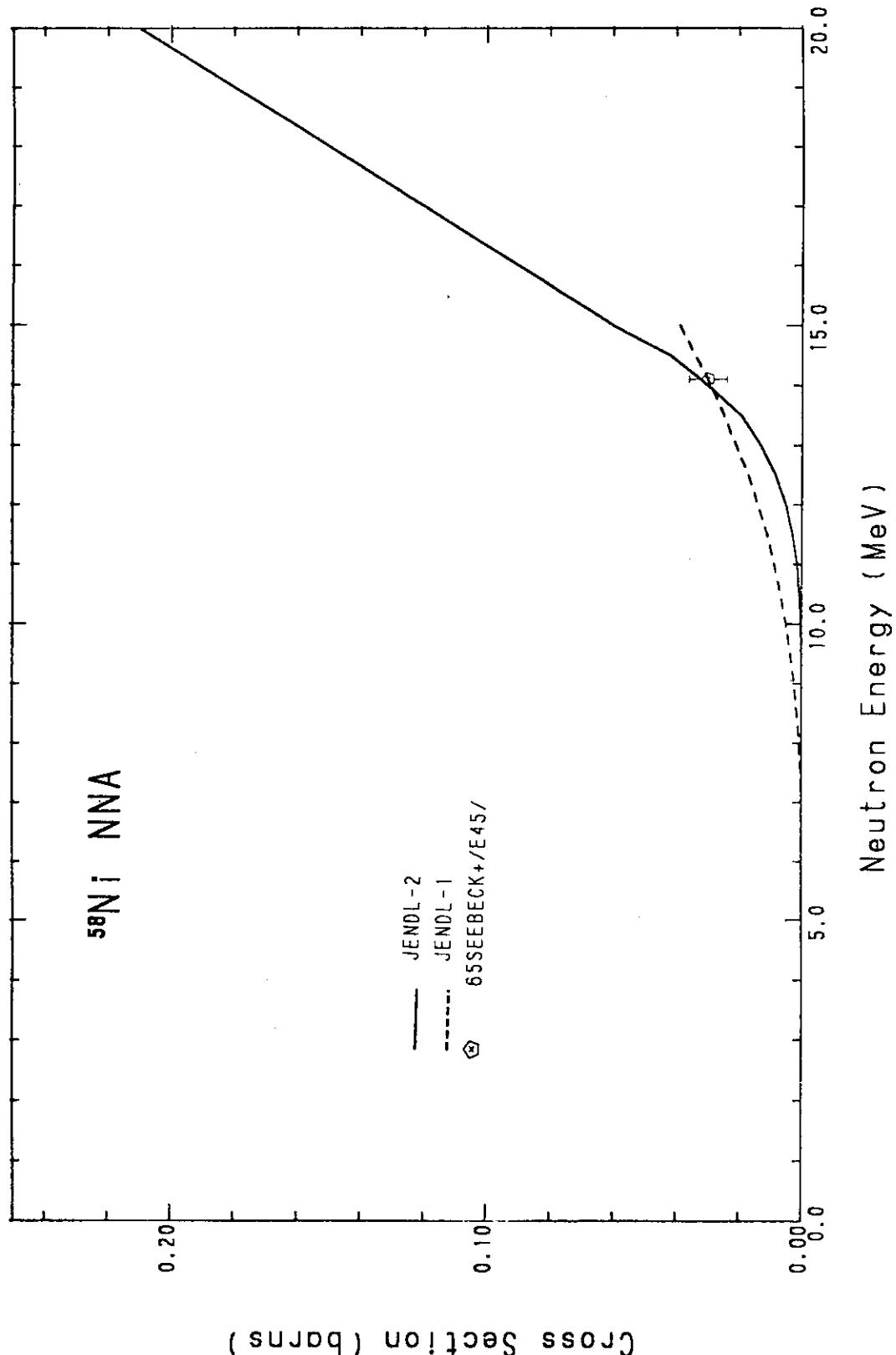


Fig. 16 ($n, n'\alpha$) reaction cross section of ^{58}Ni .

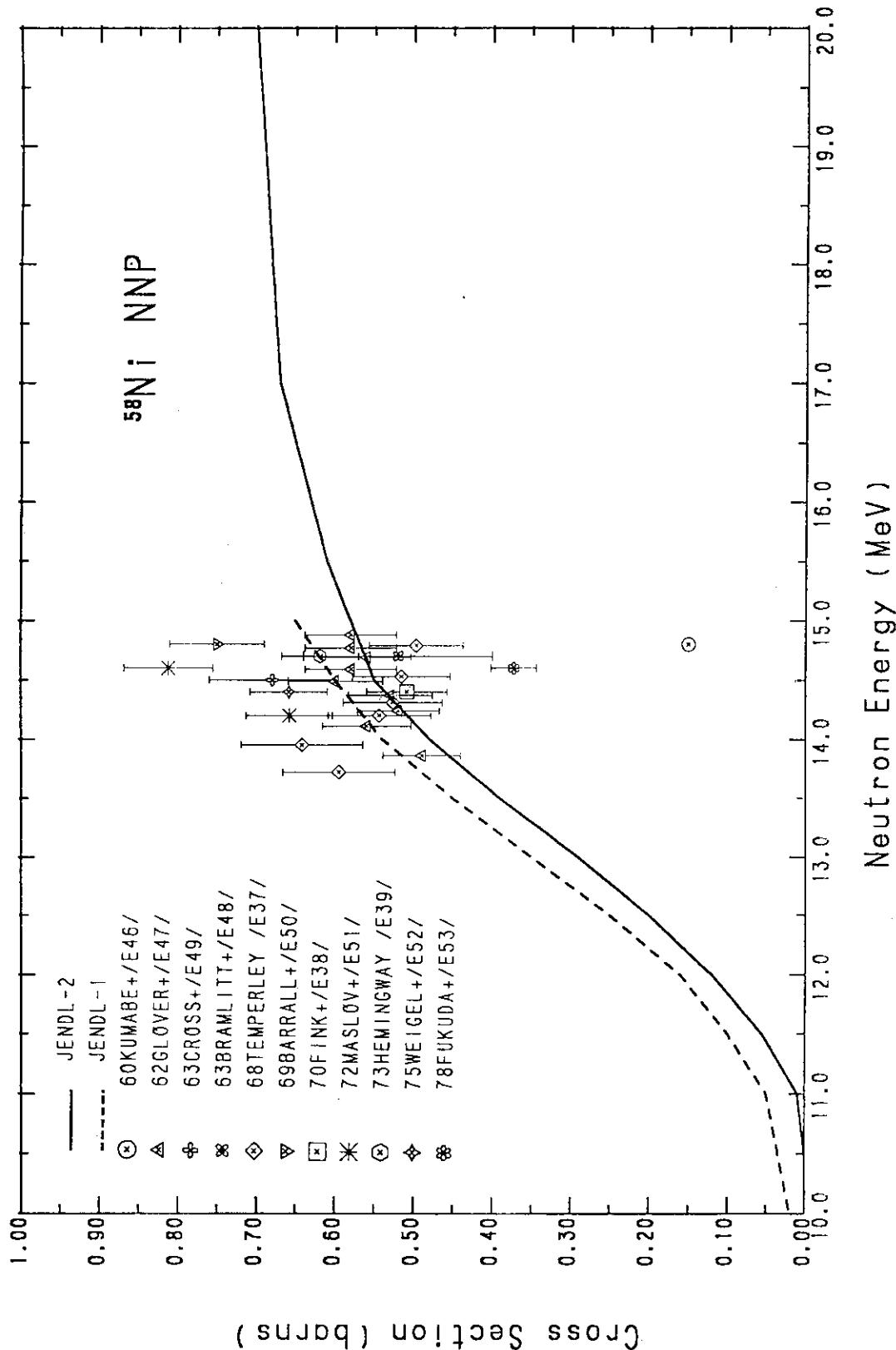


Fig. 17 $(n, n'p)$ reaction cross section of ^{58}Ni .

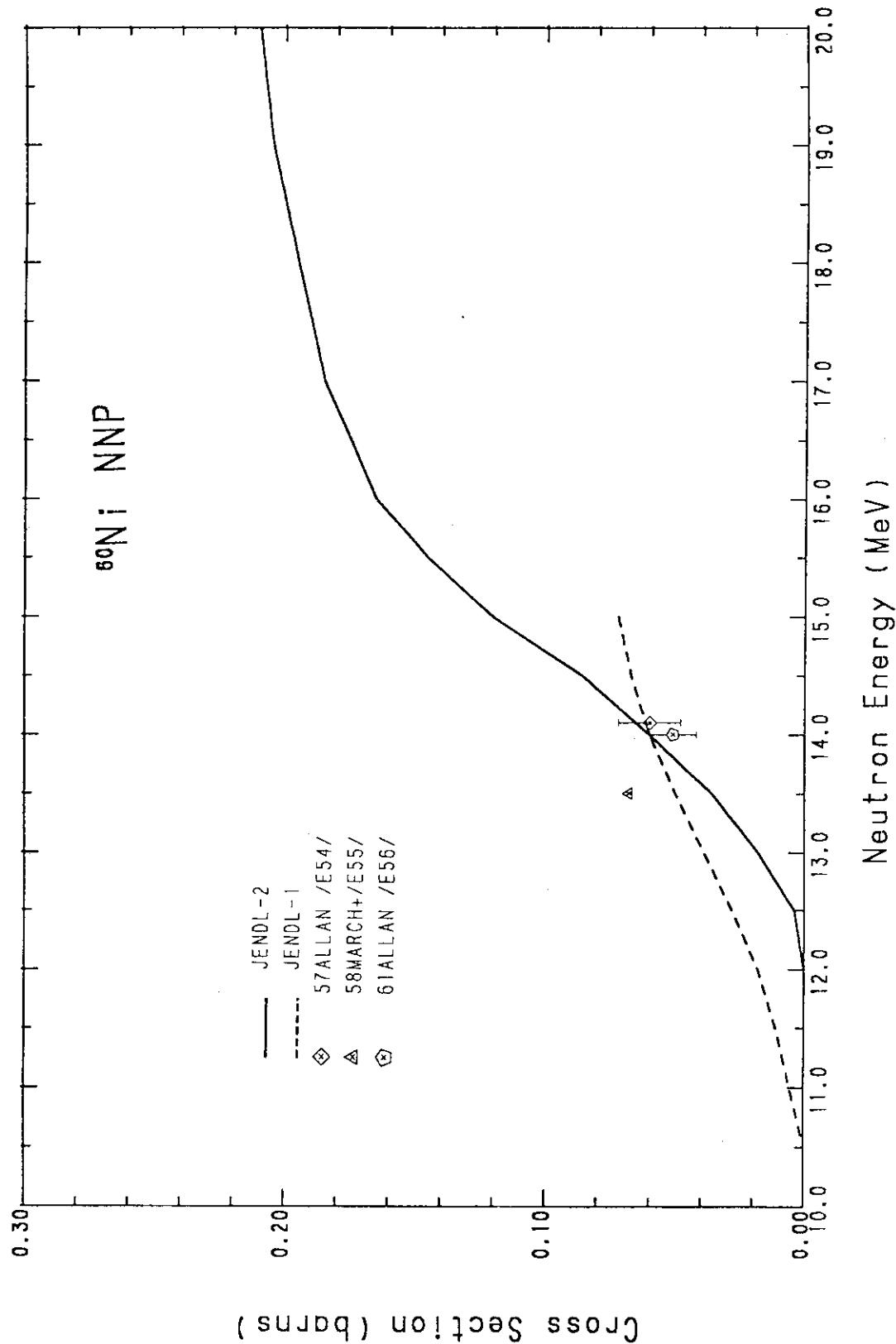


Fig. 18 ($n, n'p$) reaction cross section of ^{60}Ni .

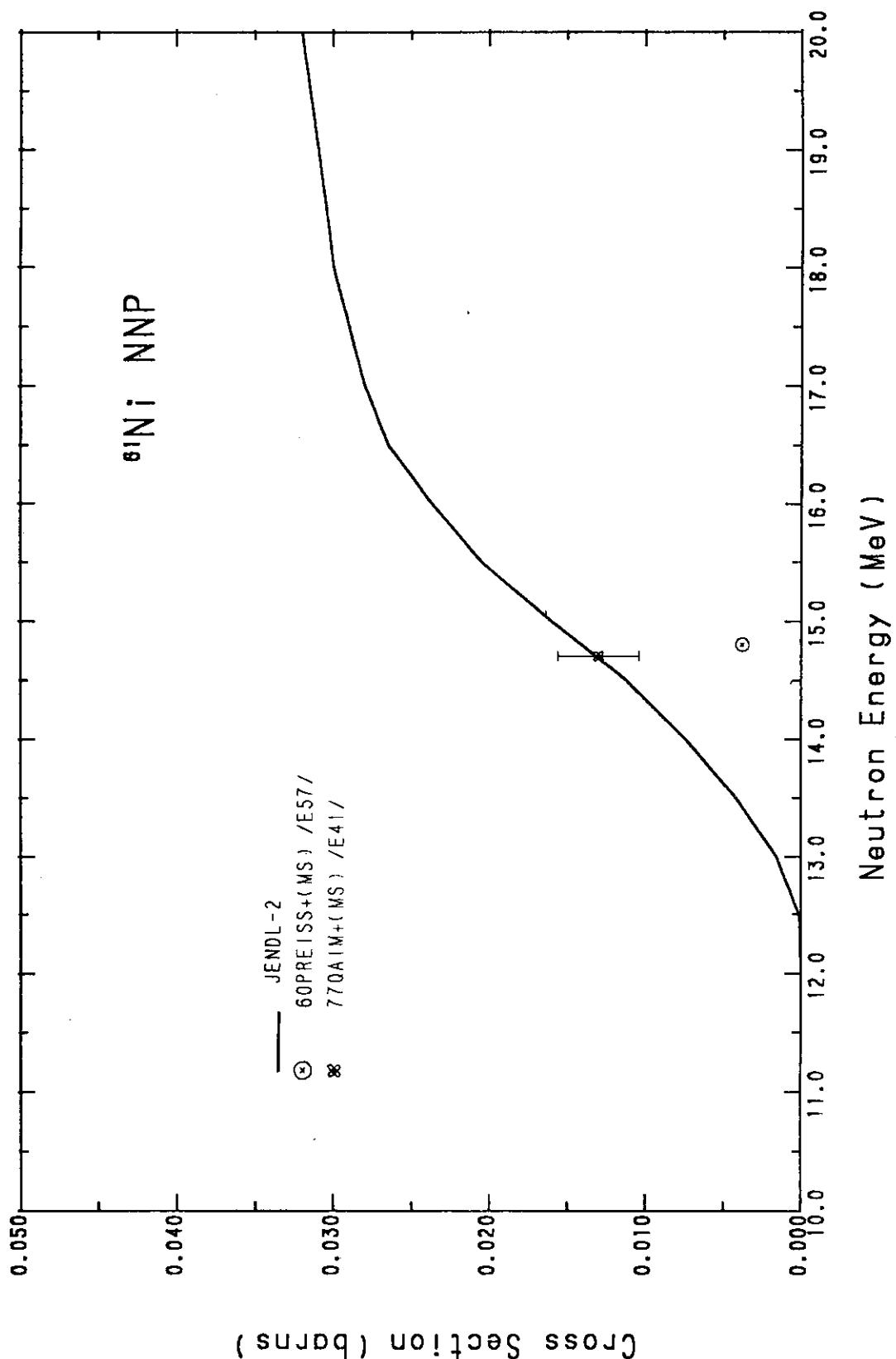
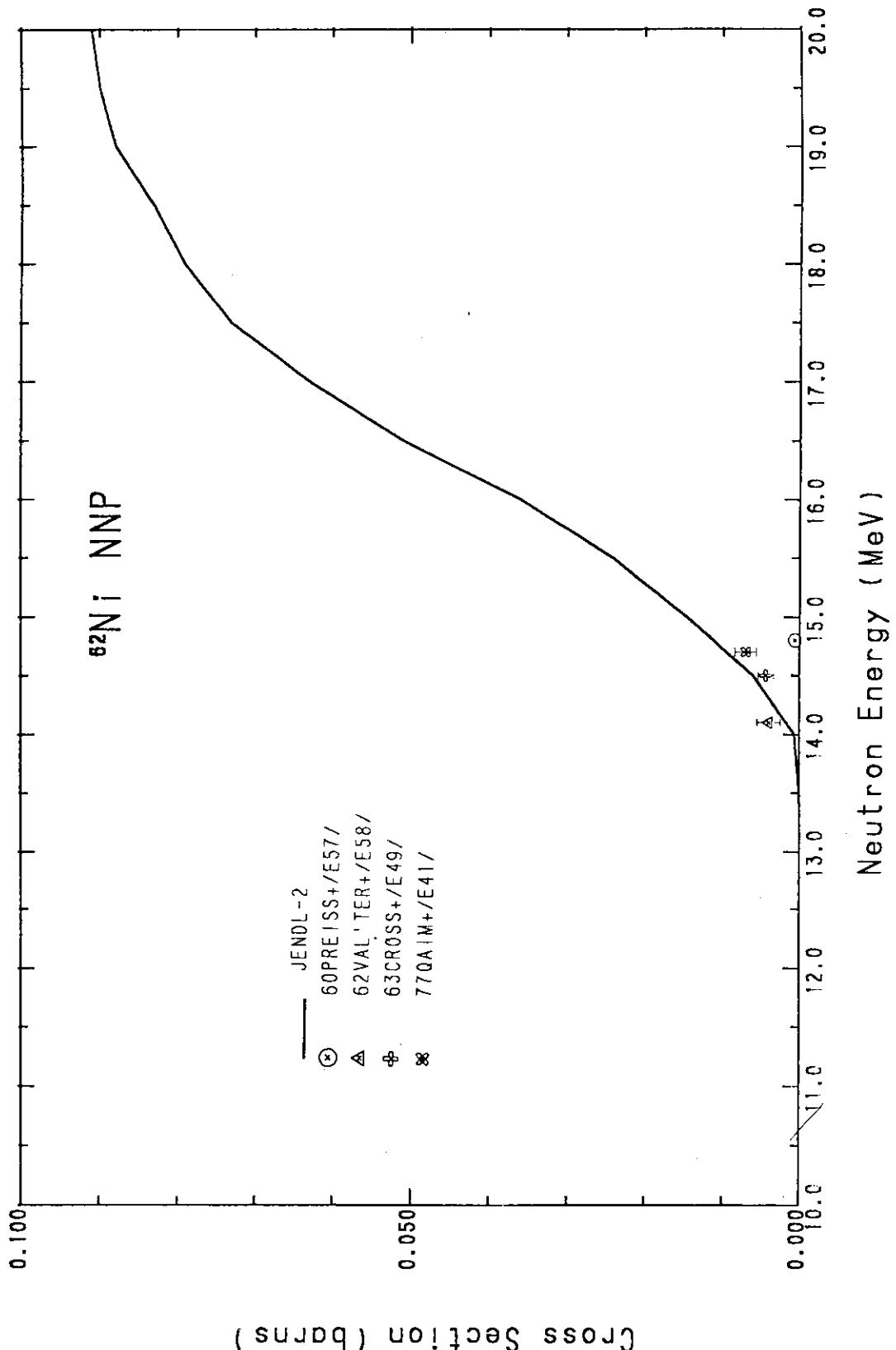
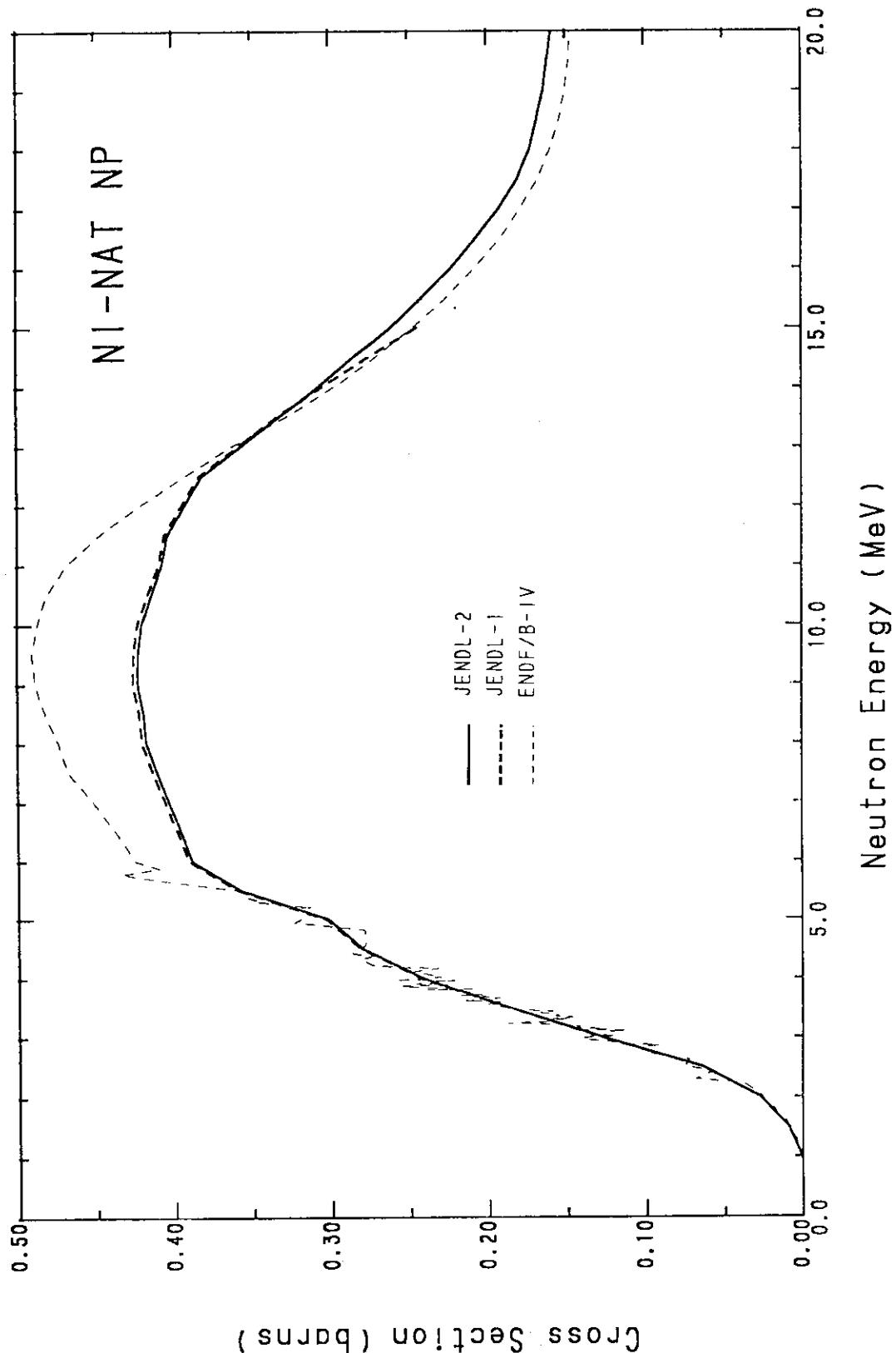
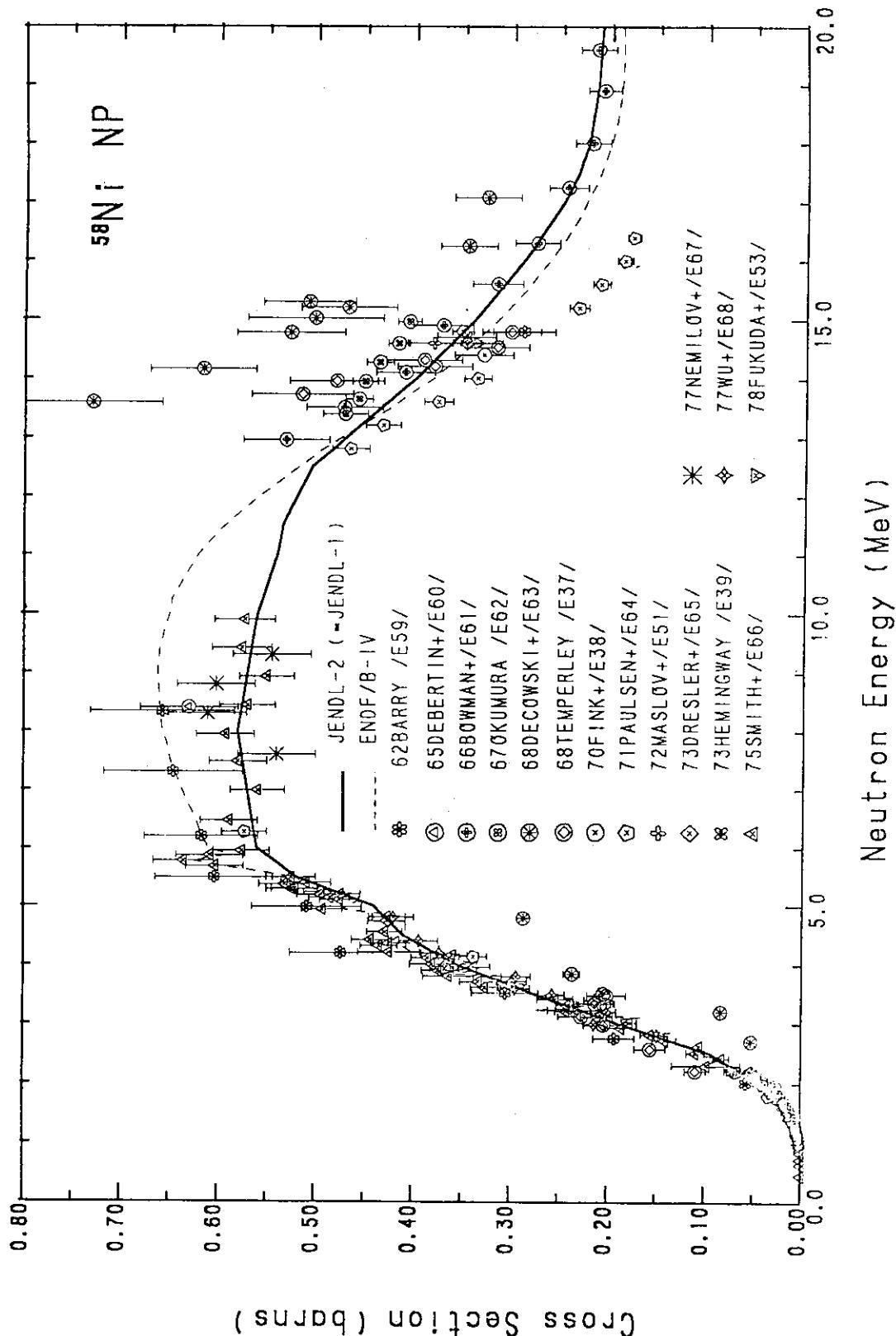


Fig. 19 (n,n'p) reaction cross section of ^{61}Ni .

Fig. 20 (n, n'p) reaction cross section of ^{62}Ni .

Fig. 21 (n,p) reaction cross section of natural nickel.

Fig. 22 (n, p) reaction cross section of ^{58}Ni .

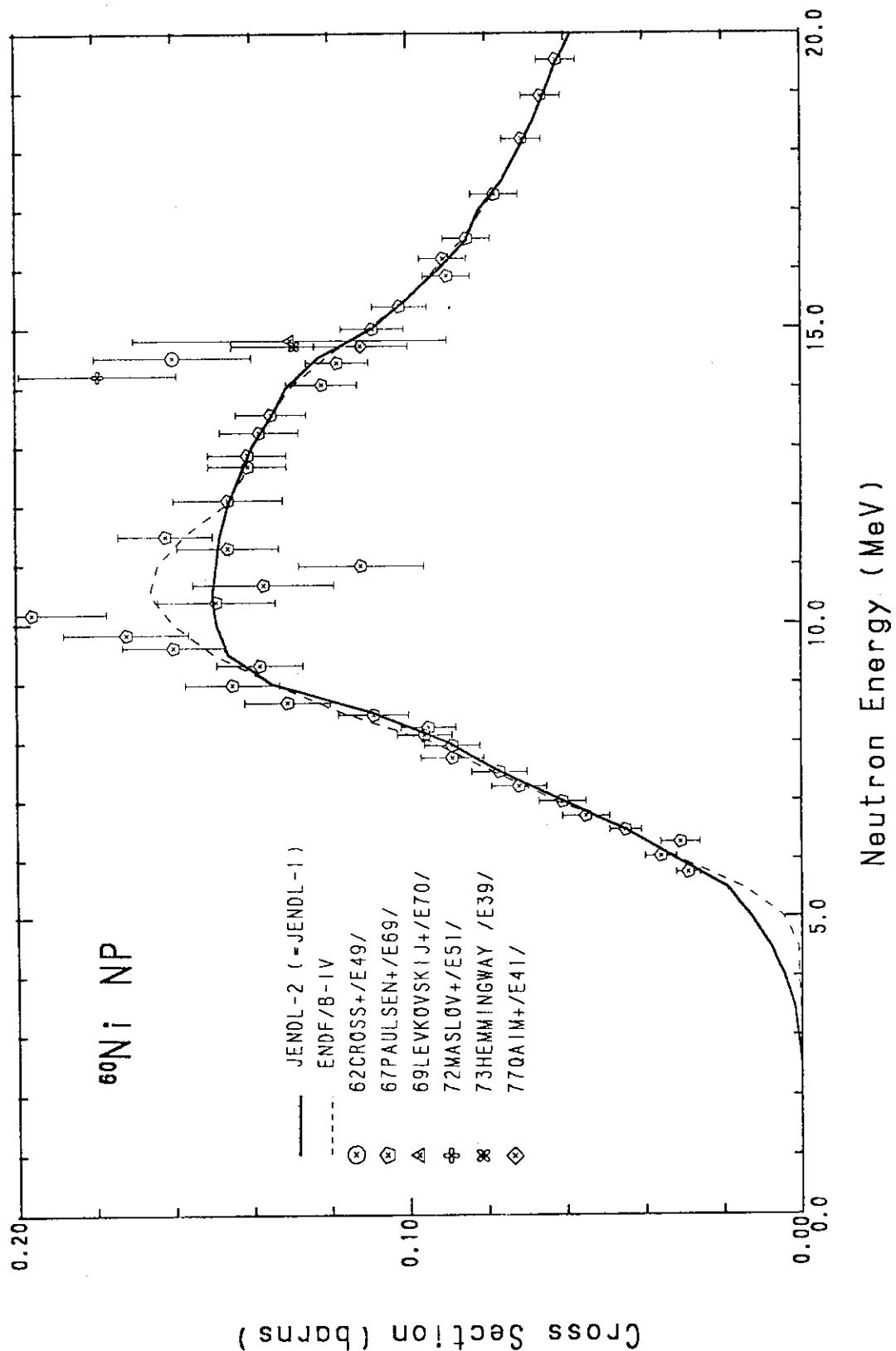


Fig. 23 (n,p) reaction cross section of ^{60}Ni .

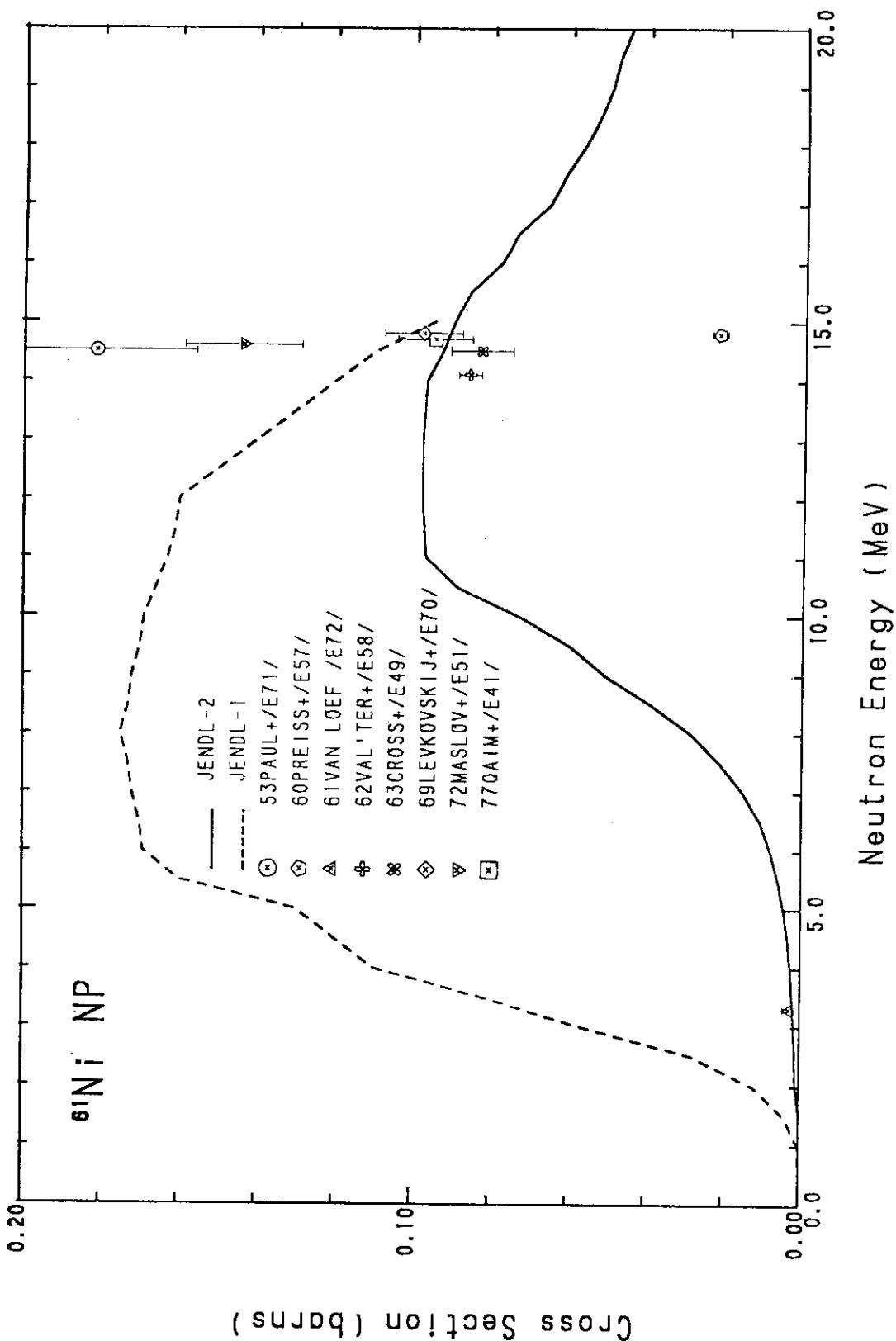


Fig. 24 (n,p) reaction cross section of ^{61}Ni .

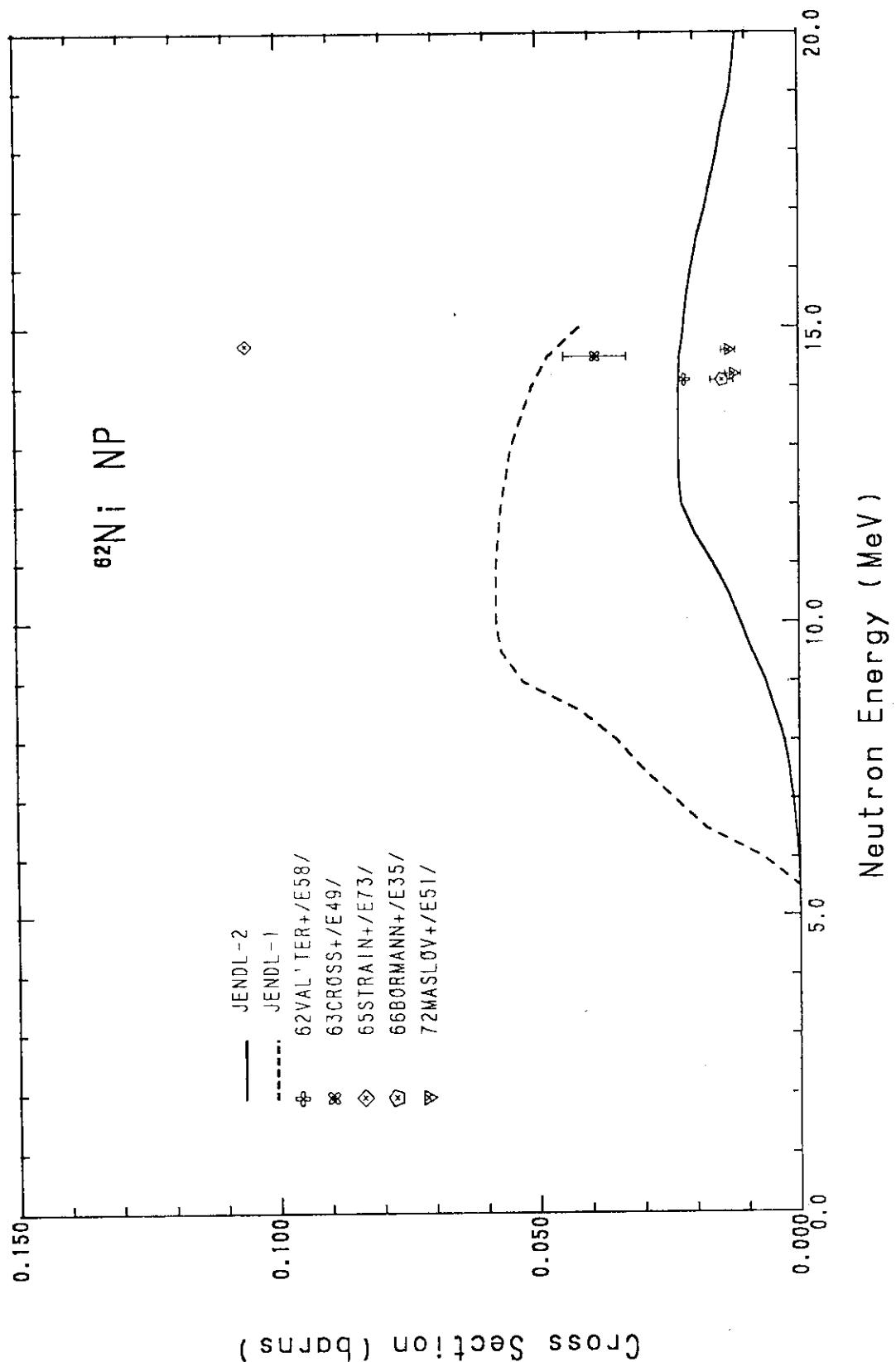


Fig. 25 (n,p) reaction cross section of ^{62}Ni .

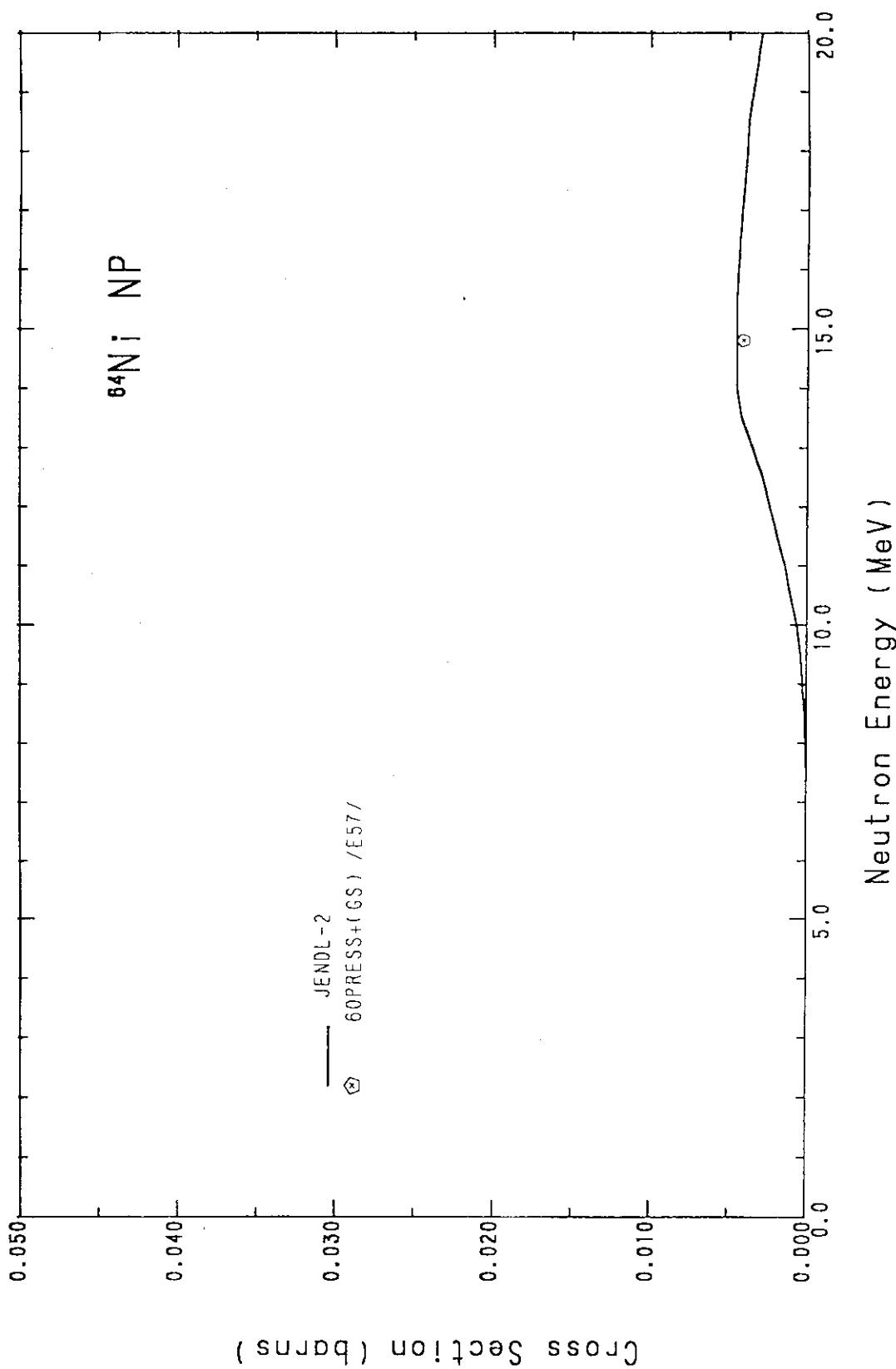


Fig. 26 (n,p) reaction cross section of ^{64}Ni .

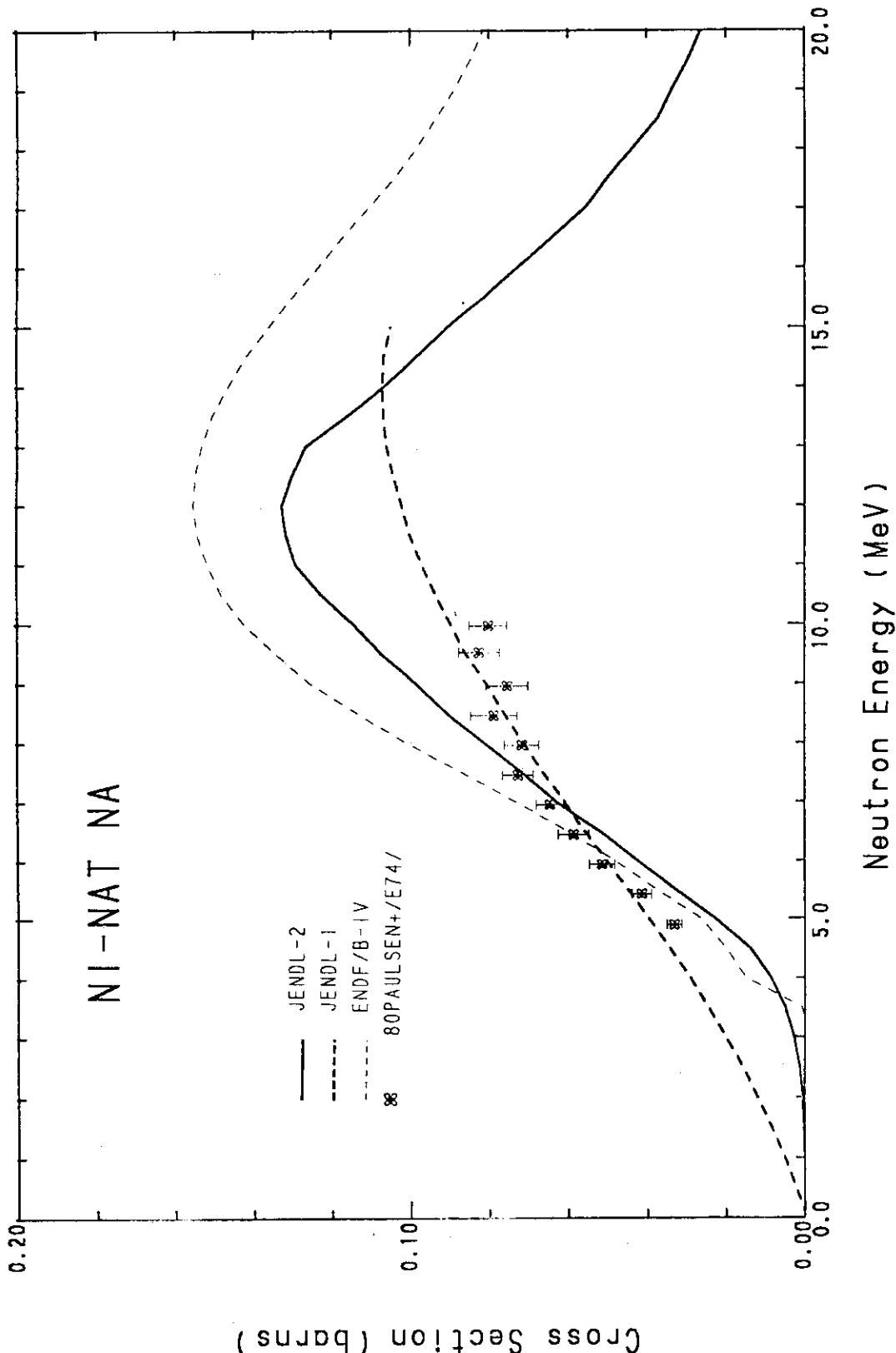


Fig. 27 (n, α) reaction cross section of natural nickel.

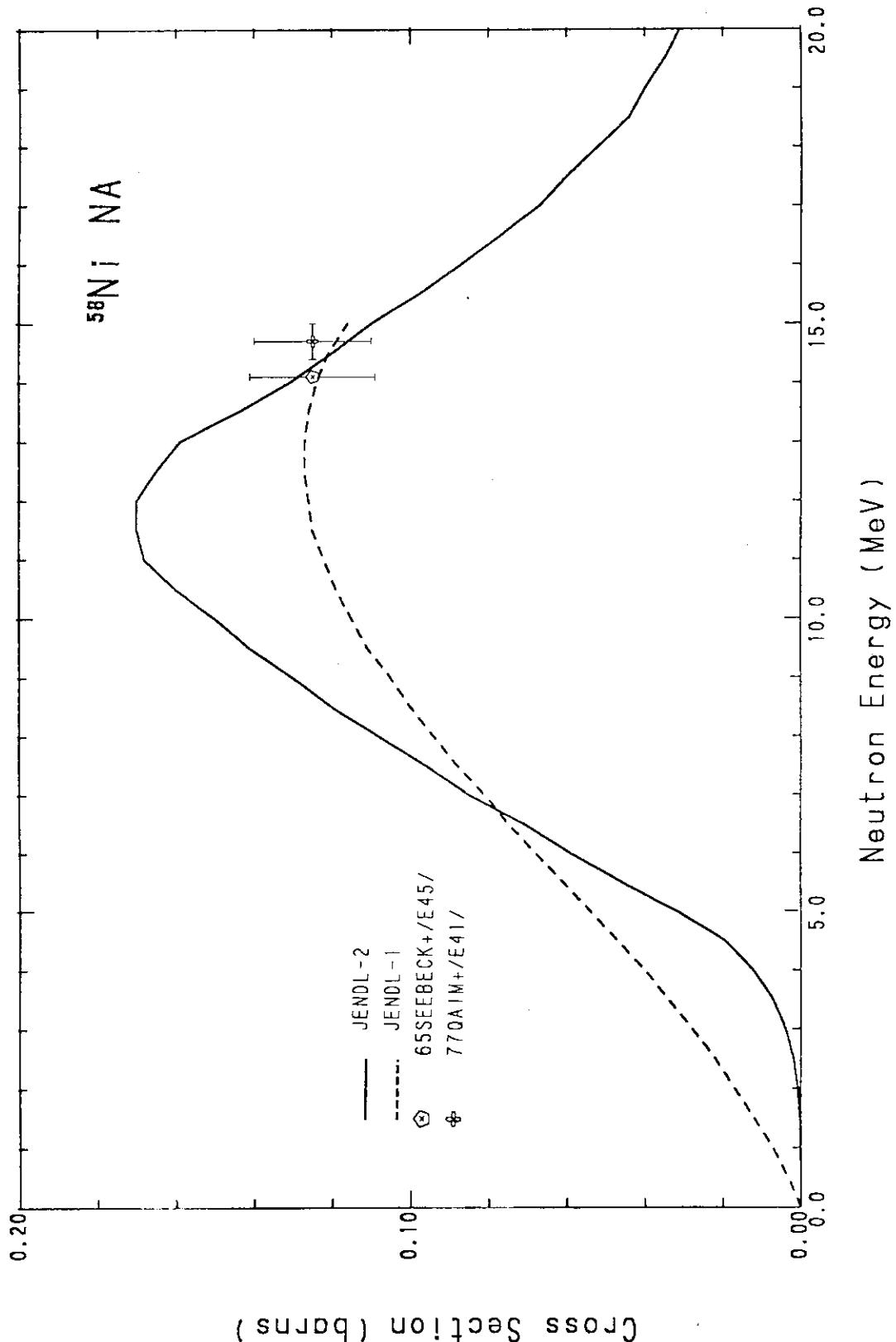


Fig. 28 (n, α) reaction cross section of ^{58}Ni .

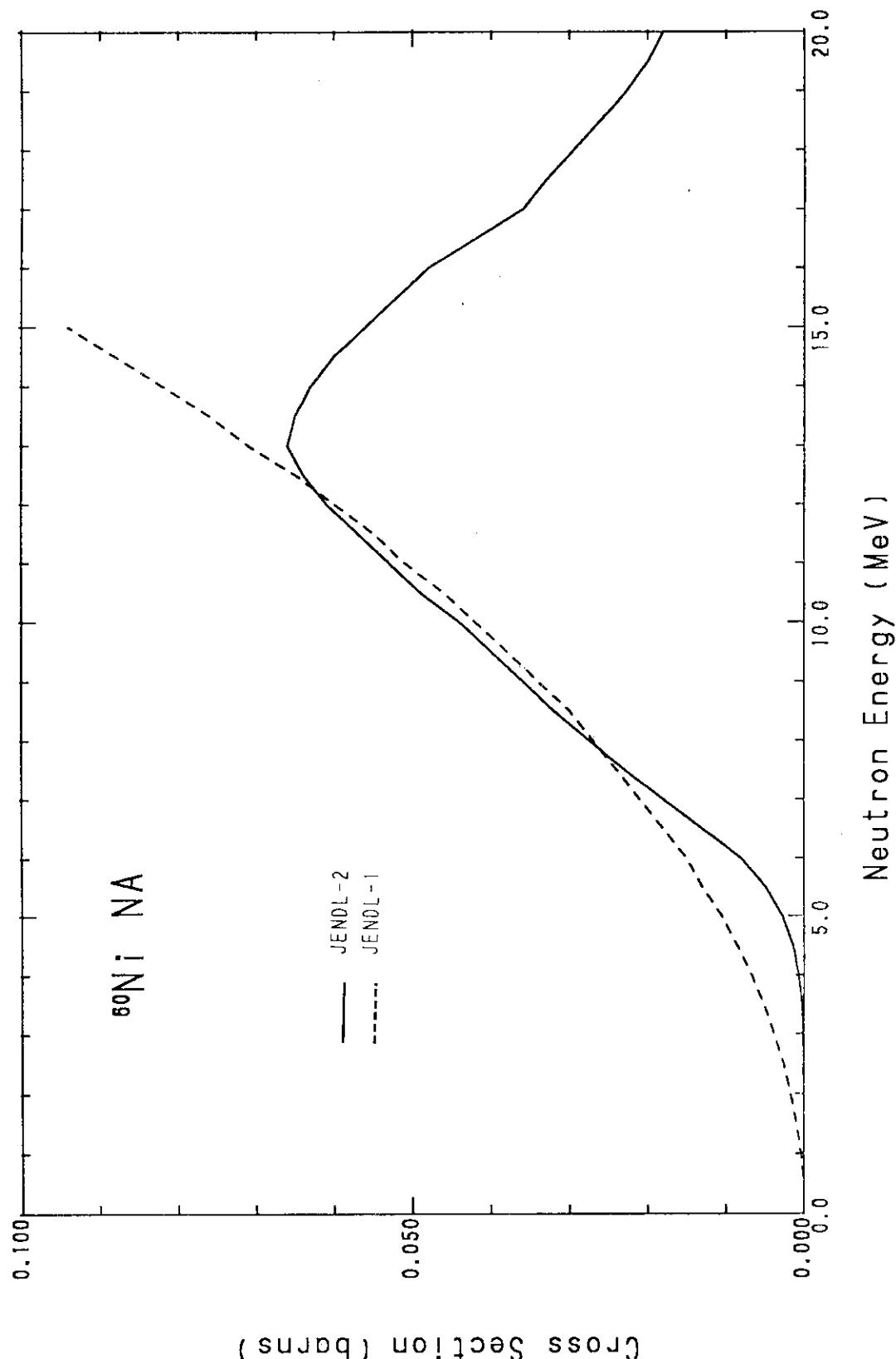


Fig. 29 (n,α) reaction cross section of ^{60}Ni .

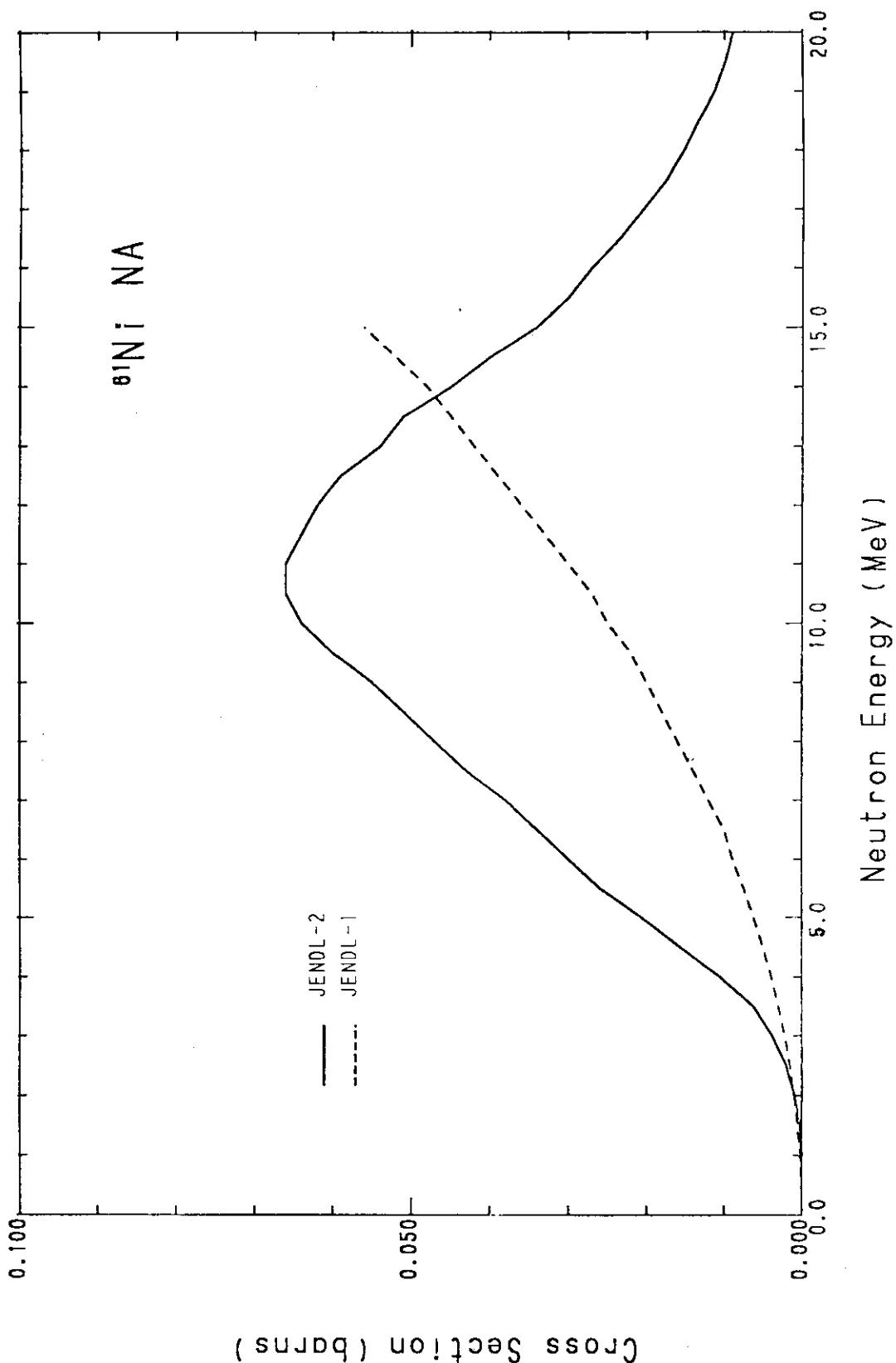


Fig. 30 (n, α) reaction cross section of ^{61}Ni .

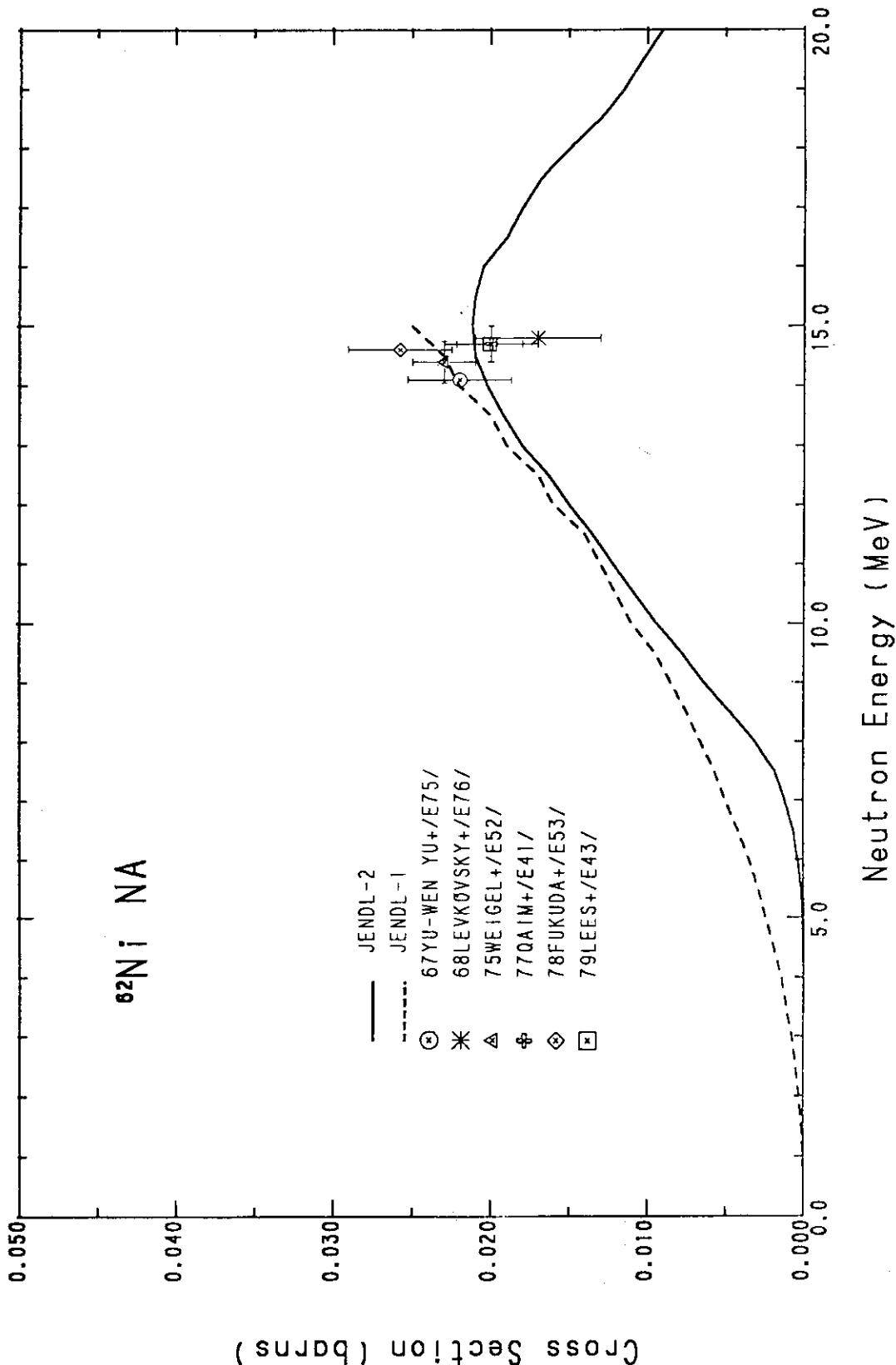


Fig. 31 (n, α) reaction cross section of ^{62}Ni .

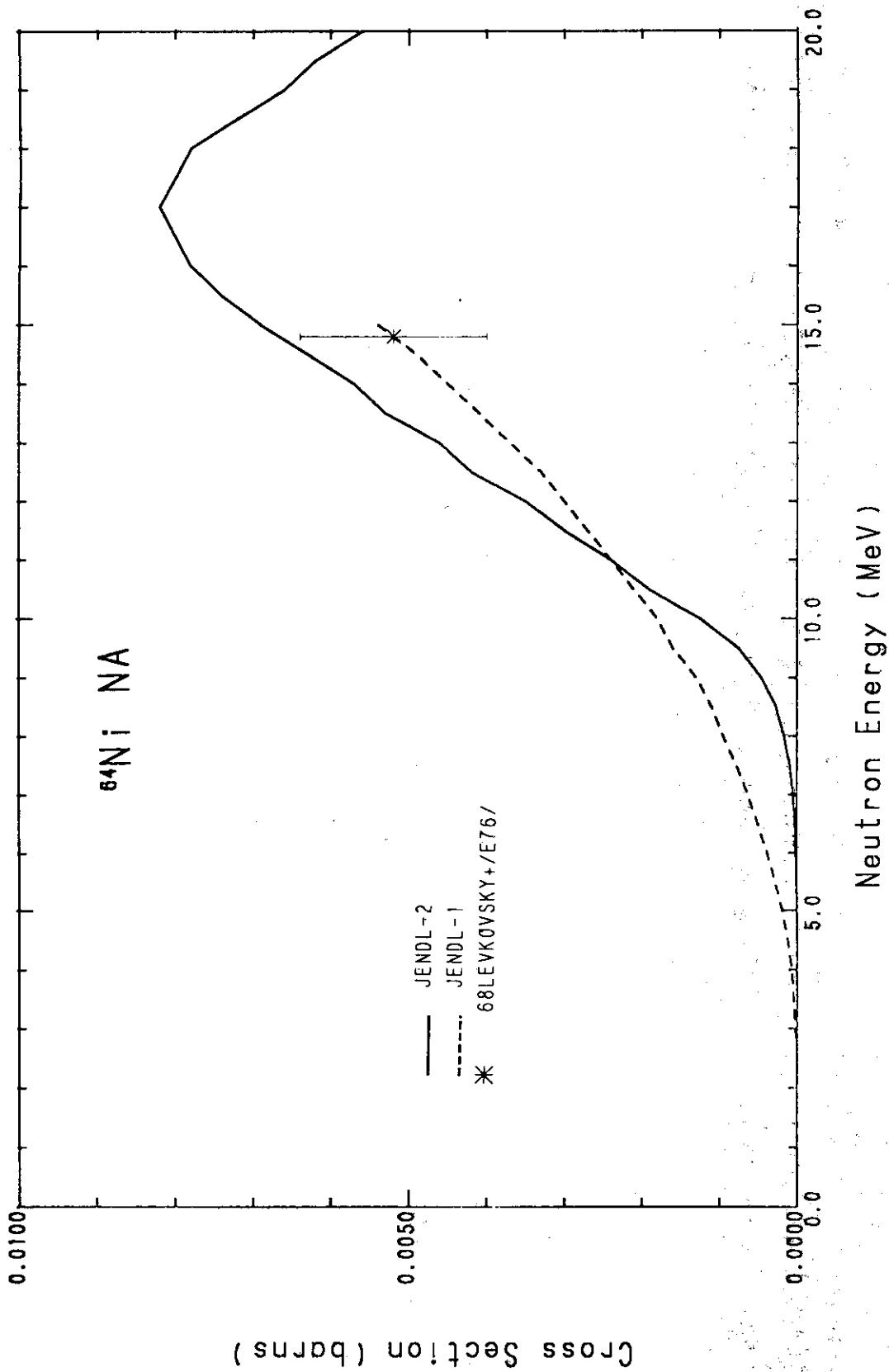


Fig. 32 (n, α) reaction cross section of ^{64}Ni .

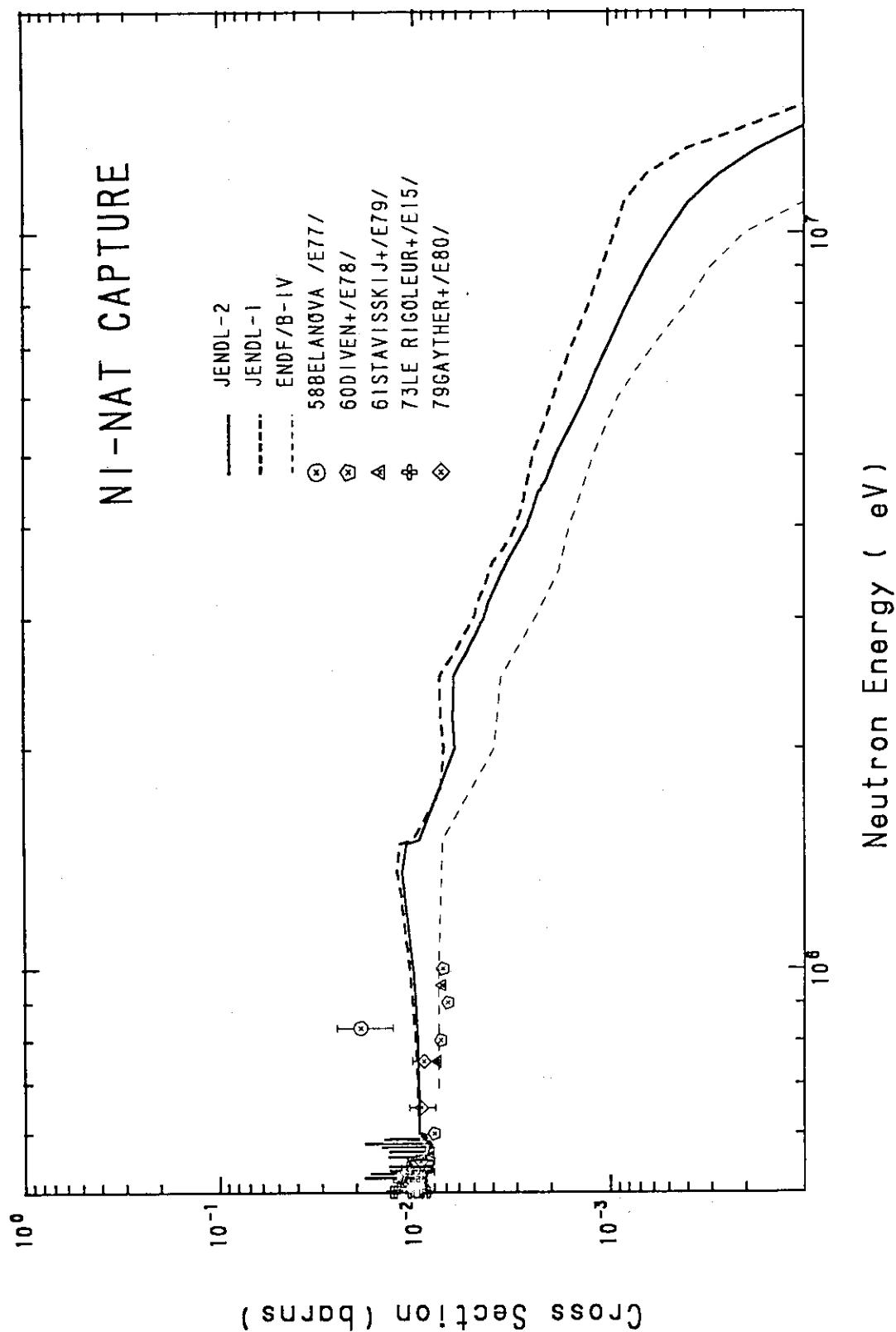


Fig. 33 Capture cross section of natural nickel.

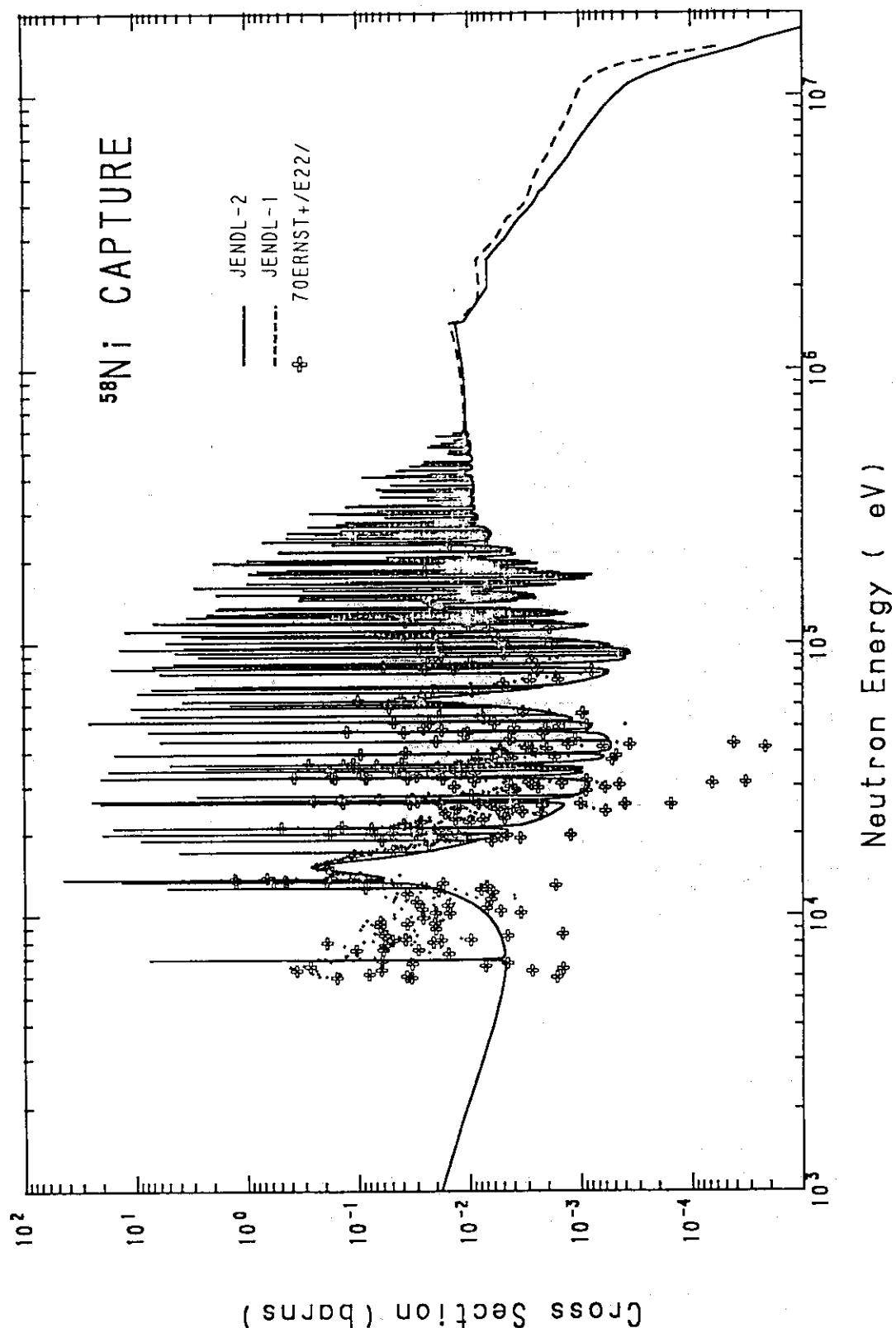
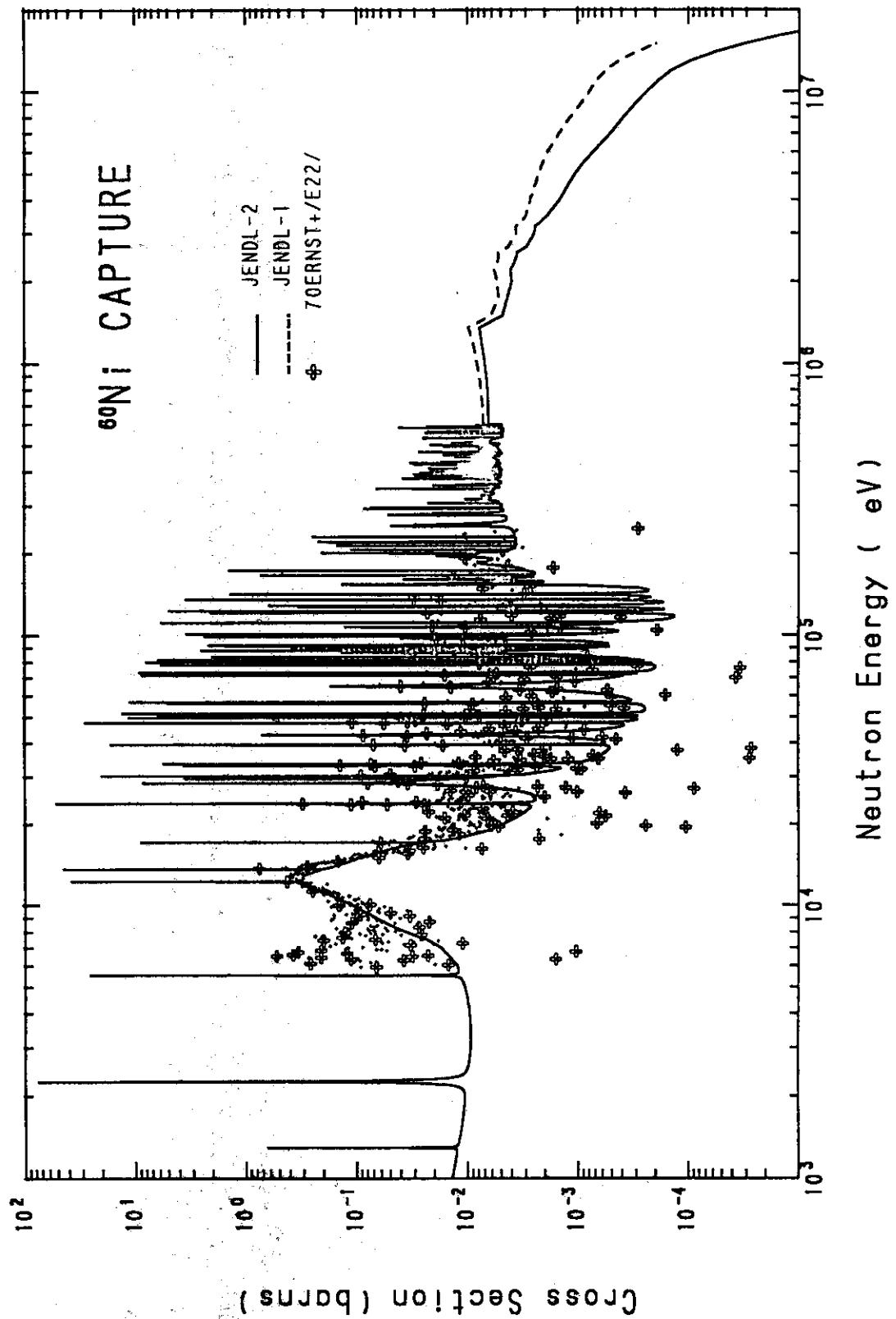
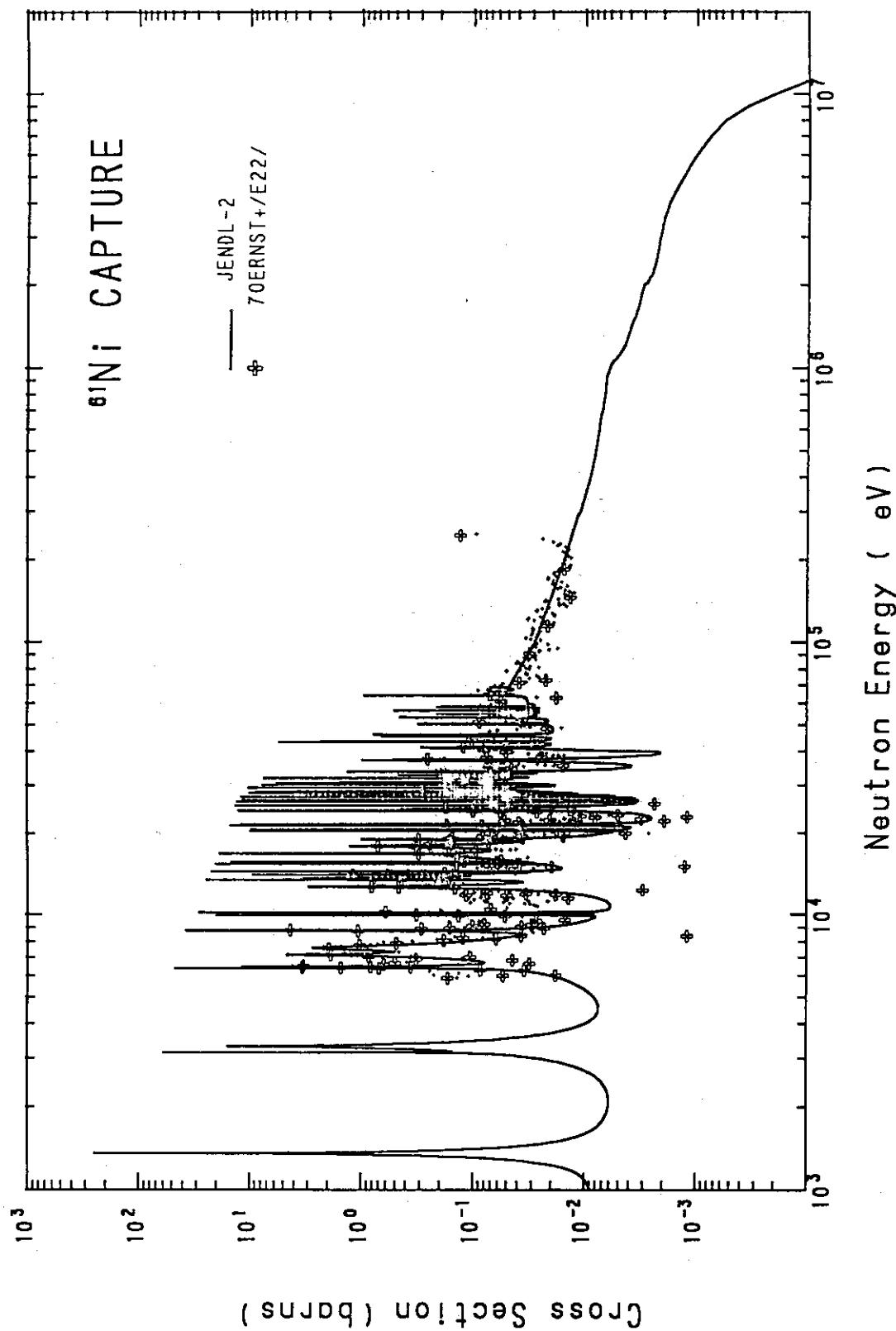
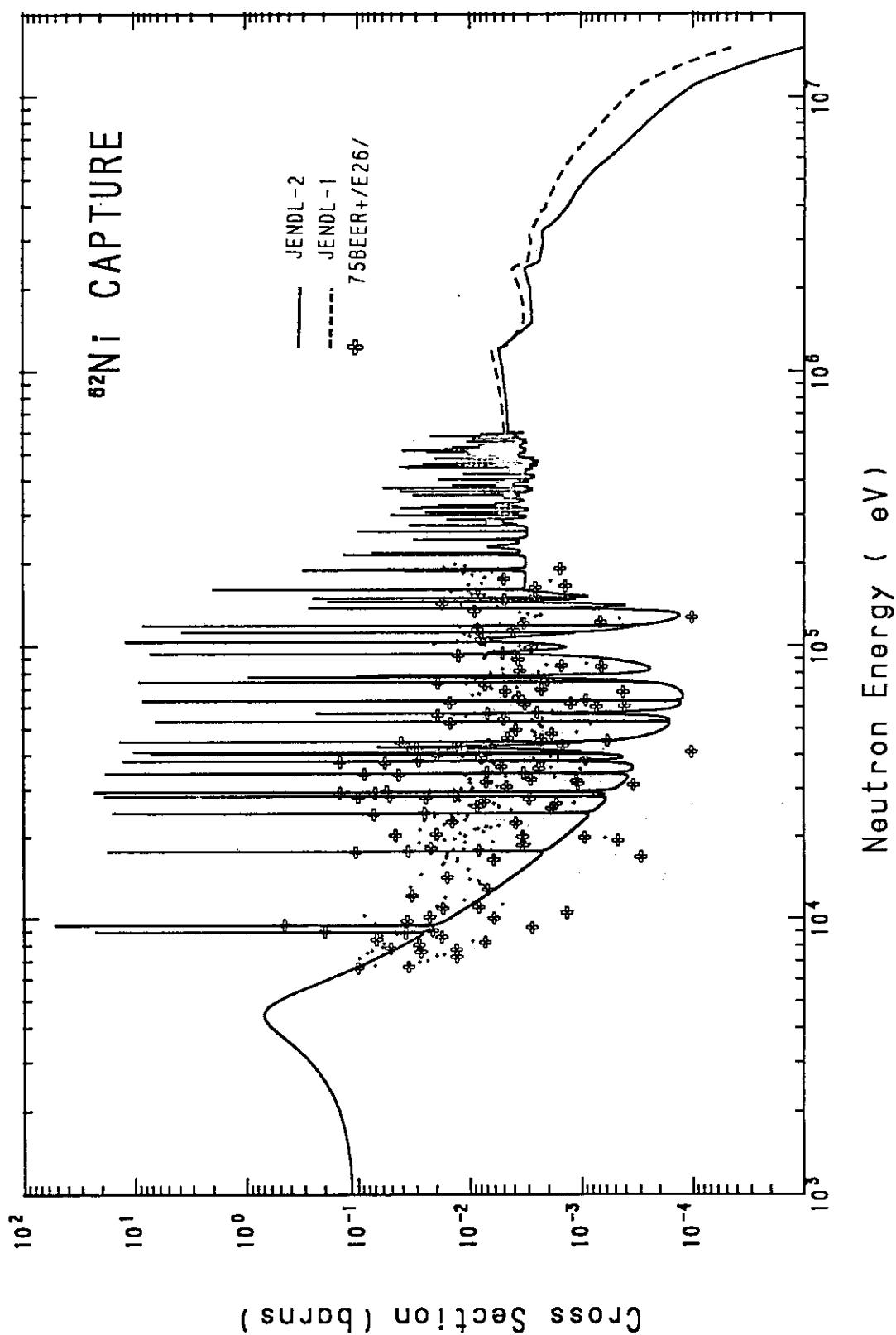
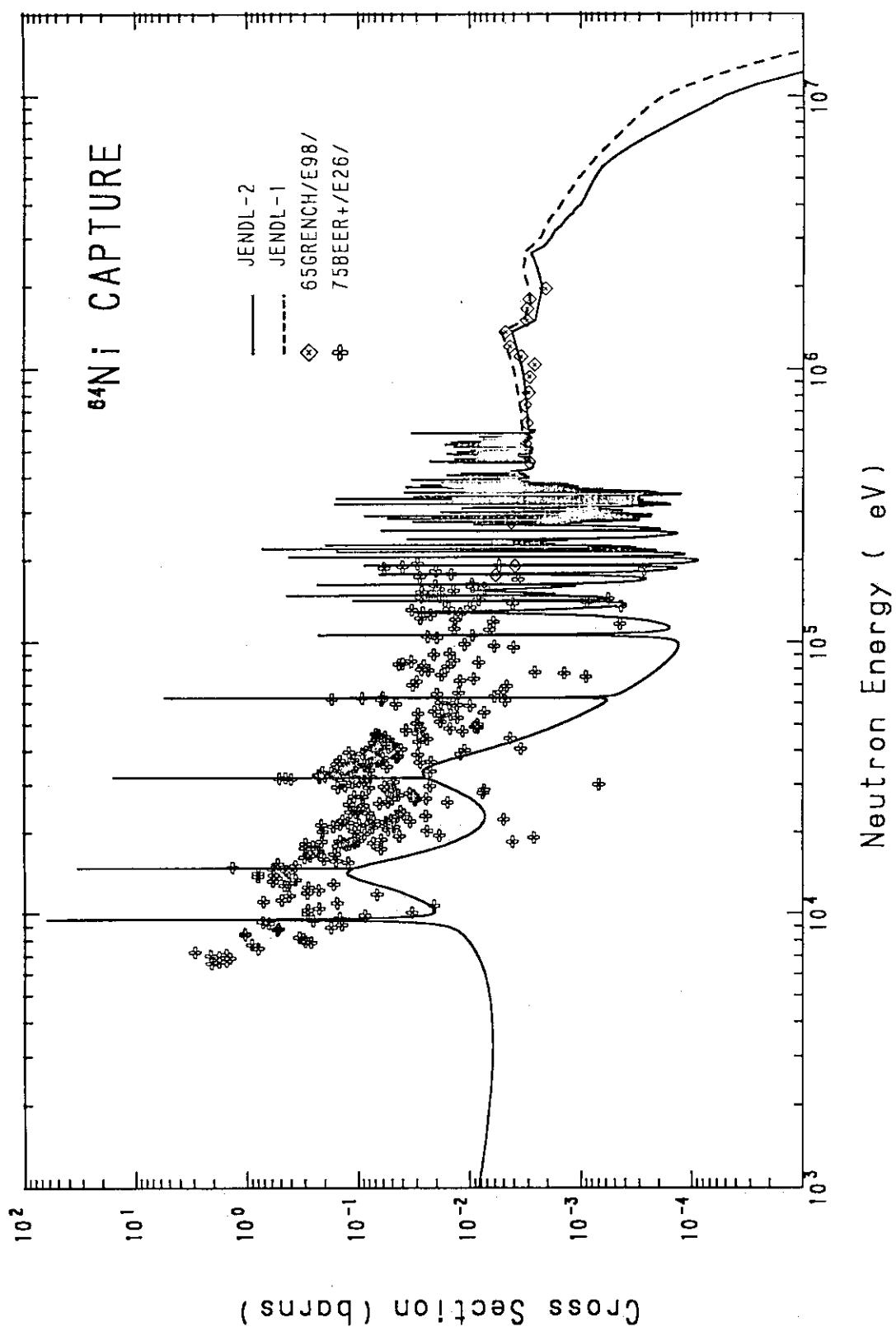


Fig. 34 Capture cross section of ^{58}Ni .

Fig. 35 Capture cross section of ^{60}Ni .

Fig. 36 Capture cross section of ^{61}Ni .

Fig. 37 Capture cross section of ^{62}Ni .

Fig. 38 Capture cross section of ^{64}Ni .

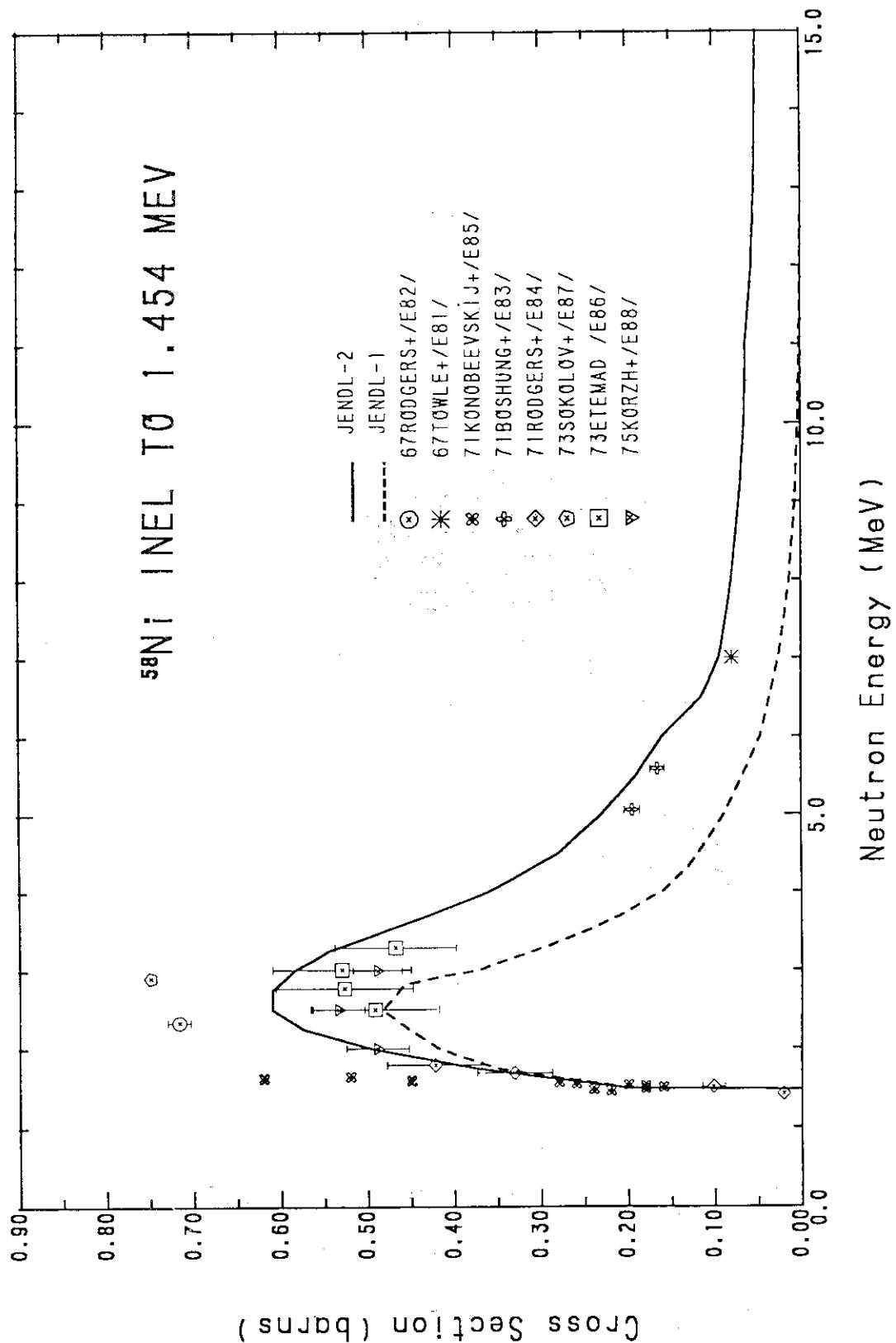


Fig. 39 Inelastic scattering cross section to the first
1.455 - Mev level of ^{58}Ni .

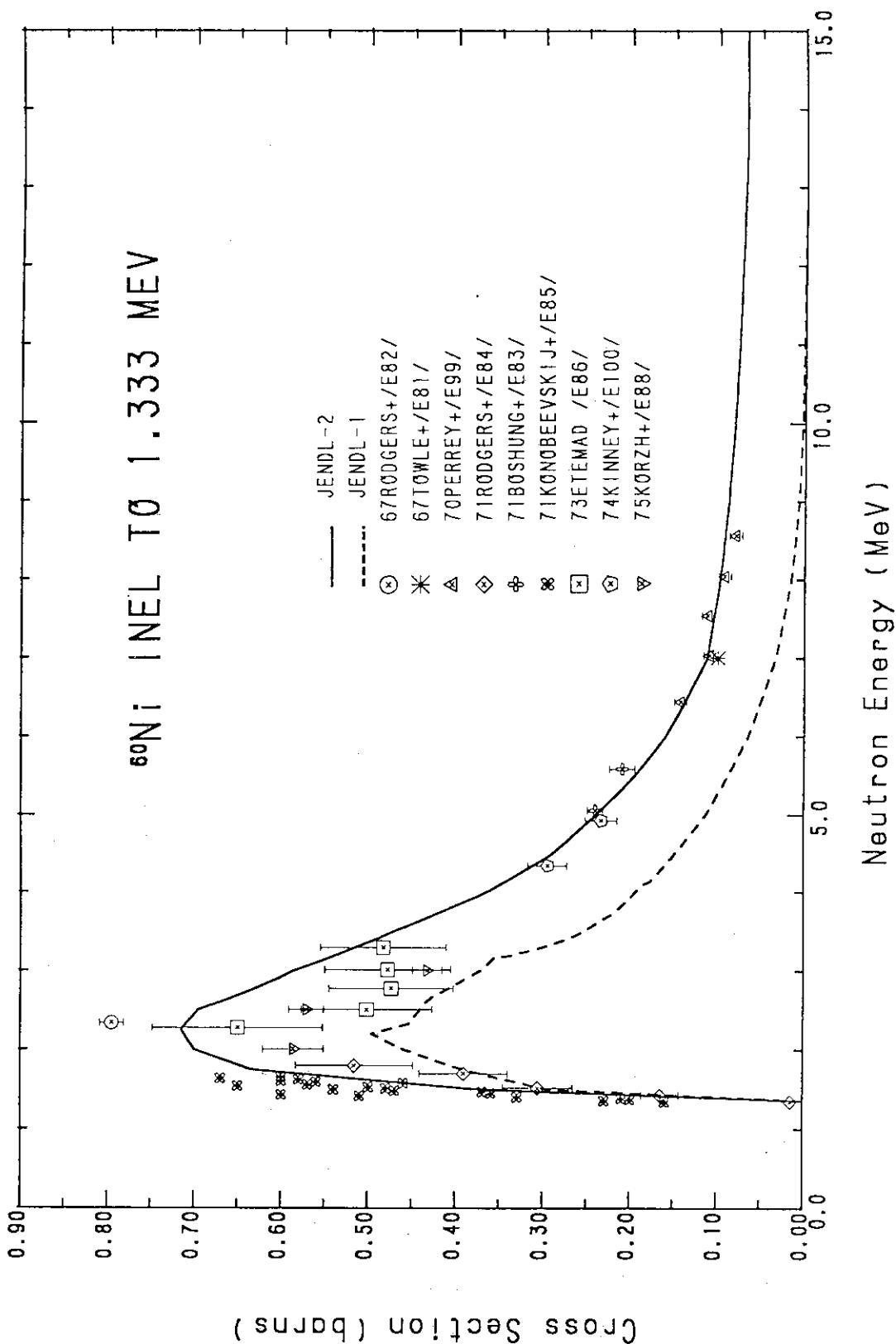


Fig. 40 Inelastic scattering cross section to the first 1.333-MeV level of ^{60}Ni .

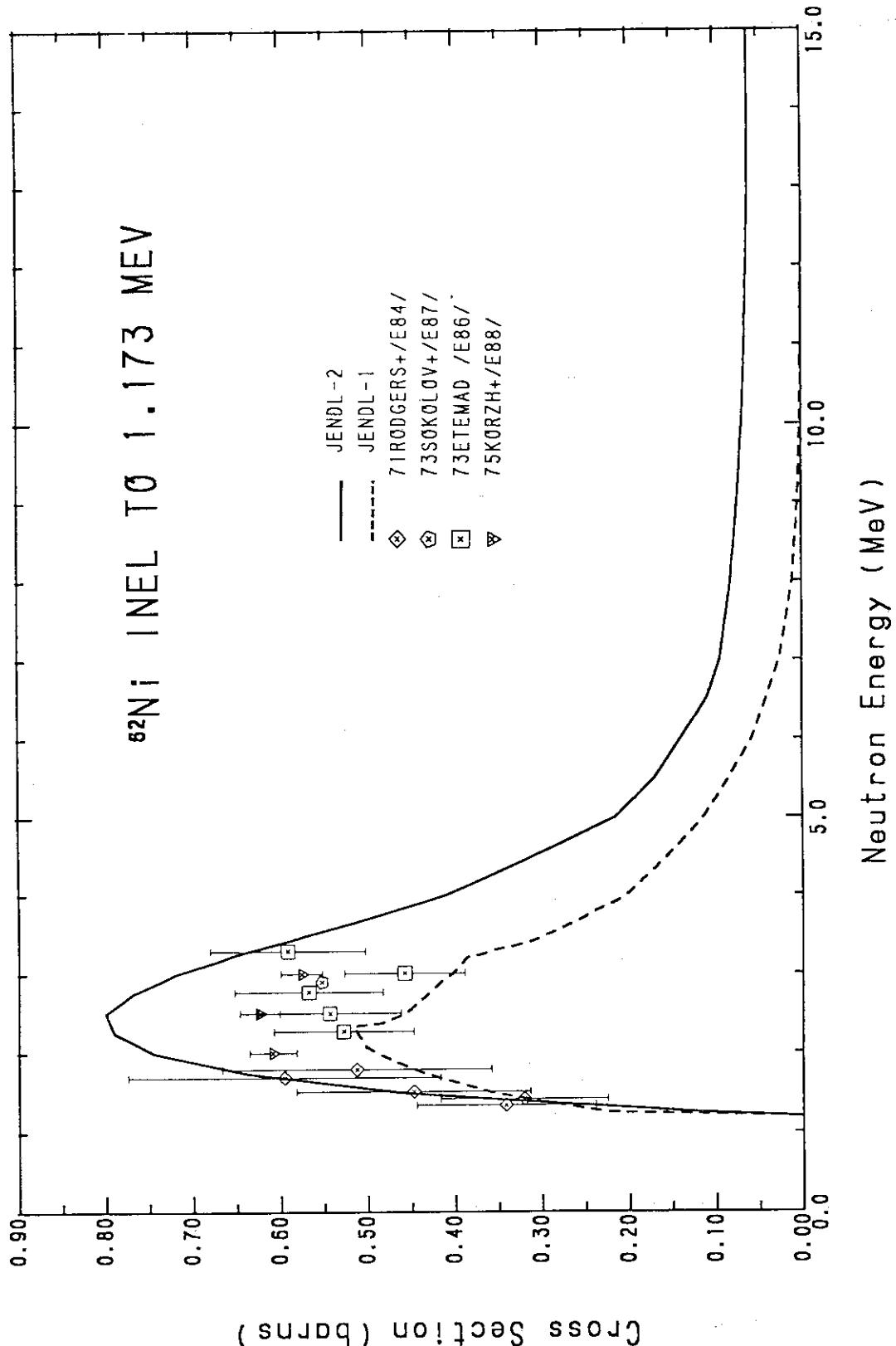


Fig. 41 Inelastic scattering cross section to the first 1.173 - MeV level of ^{62}Ni .

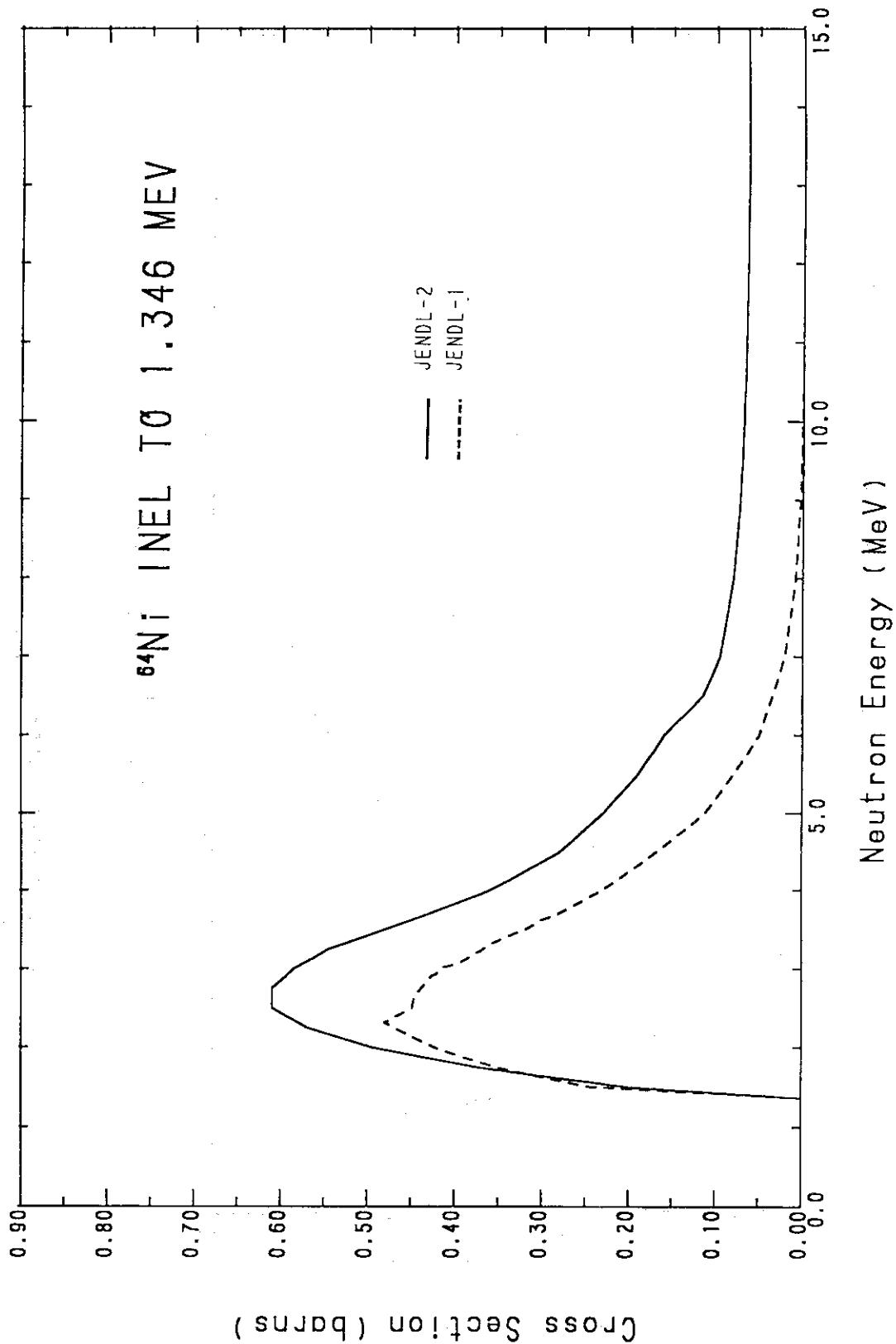


Fig. 42 Inelastic scattering cross section to the first 1.346-MeV level of ^{64}Ni .

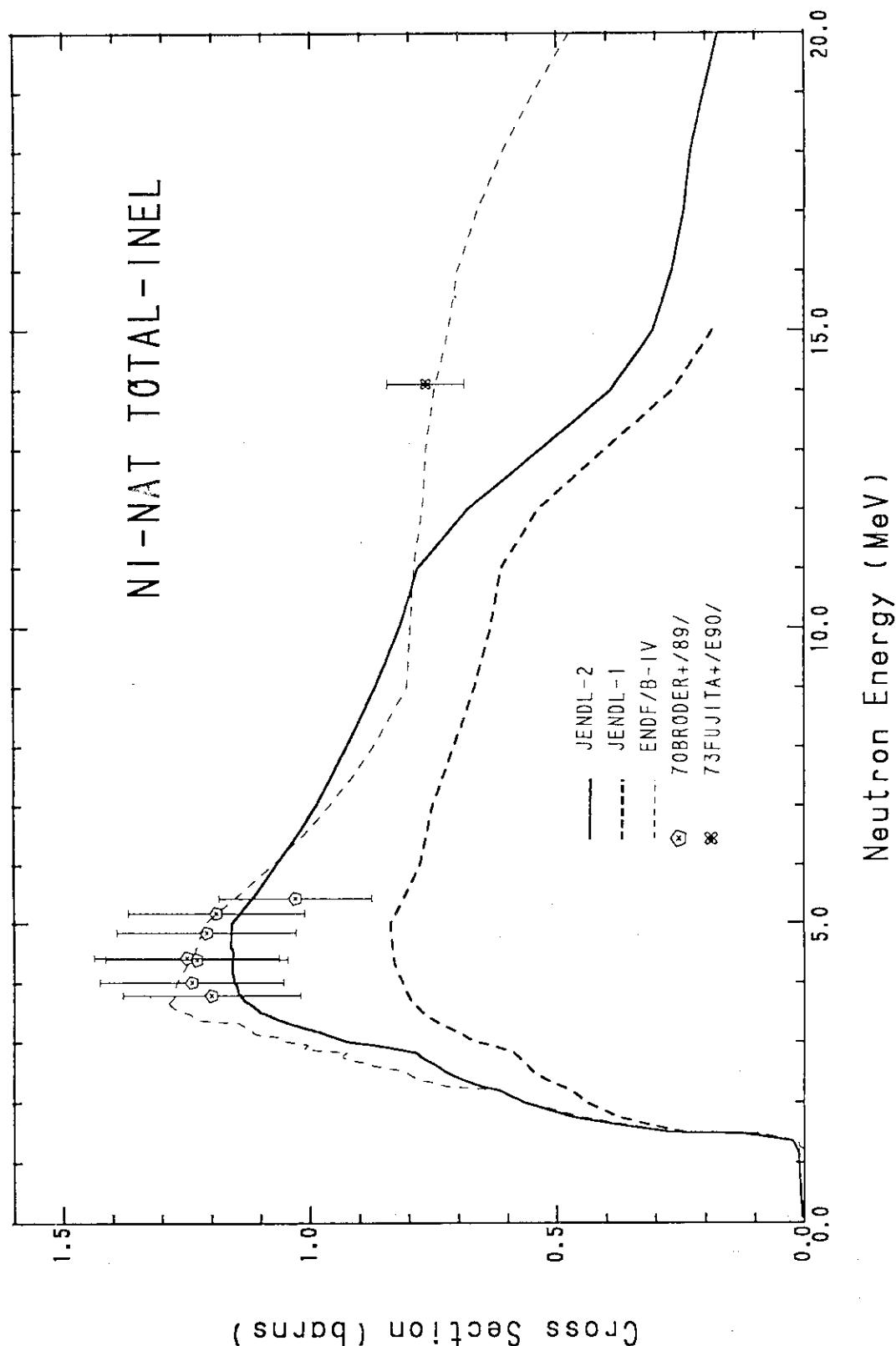


Fig. 43 Total inelastic scattering of natural nickel.

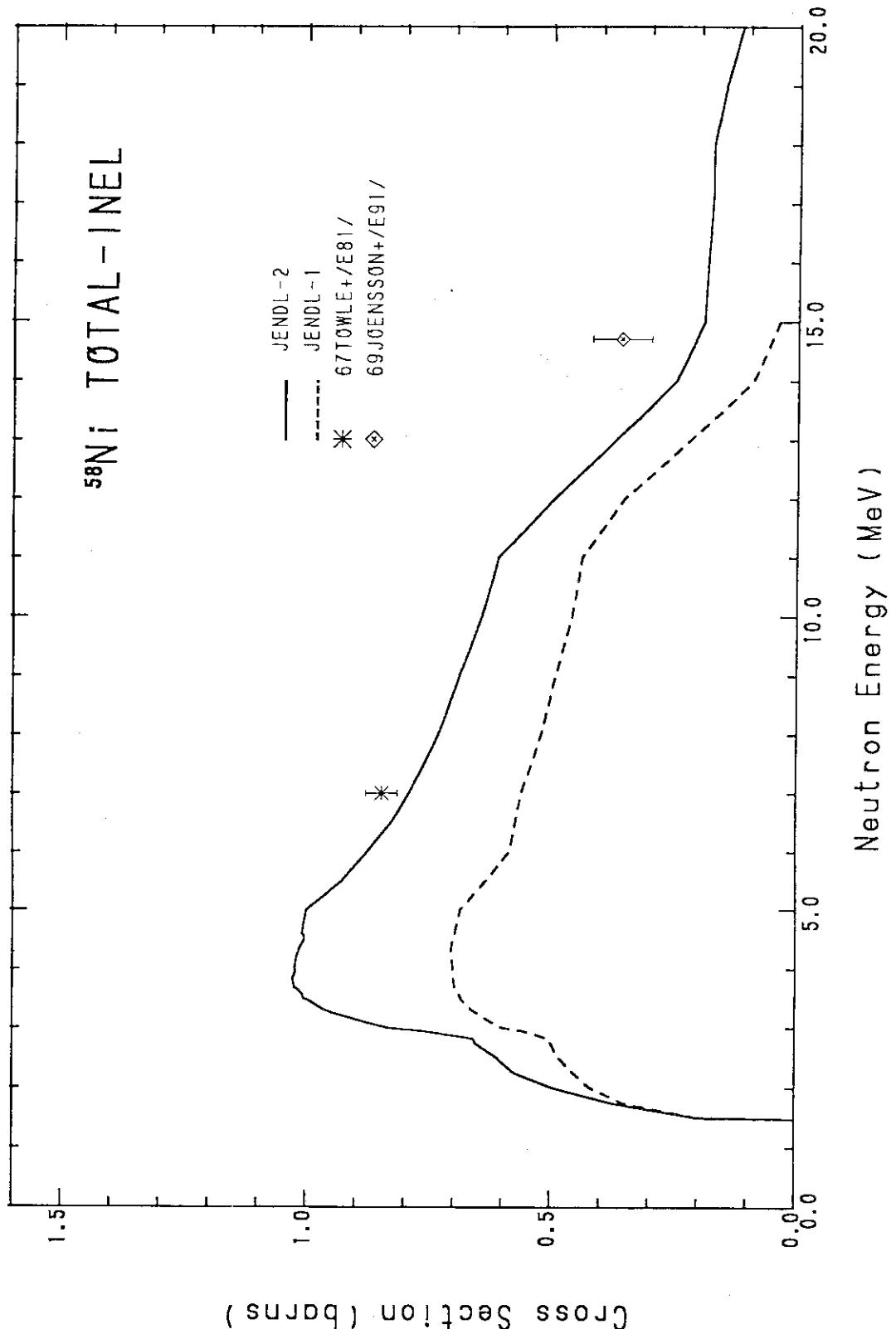


Fig. 44 Total inelastic scattering of ^{58}Ni .

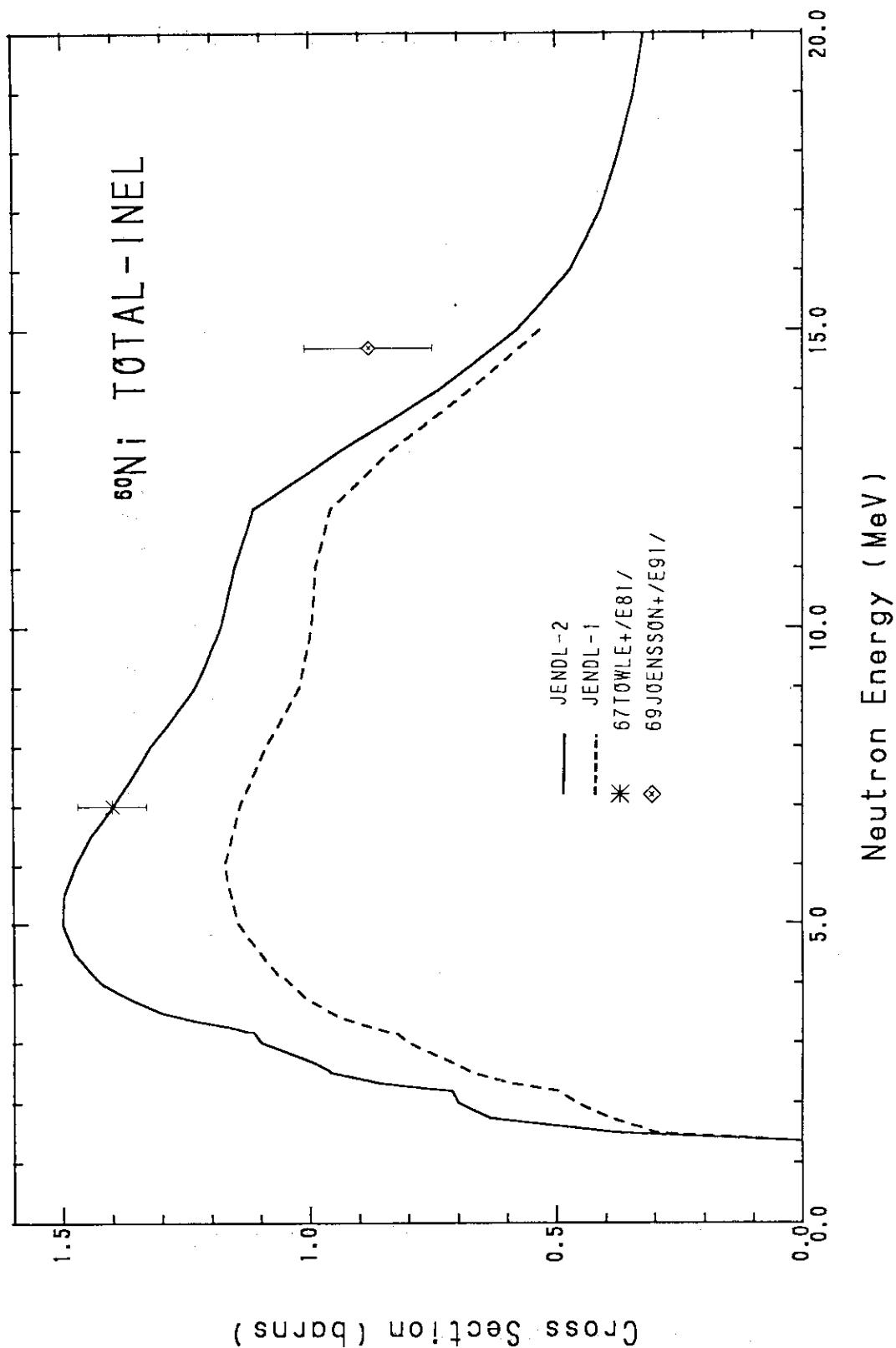


Fig. 45 Total inelastic scattering of ^{60}Ni .

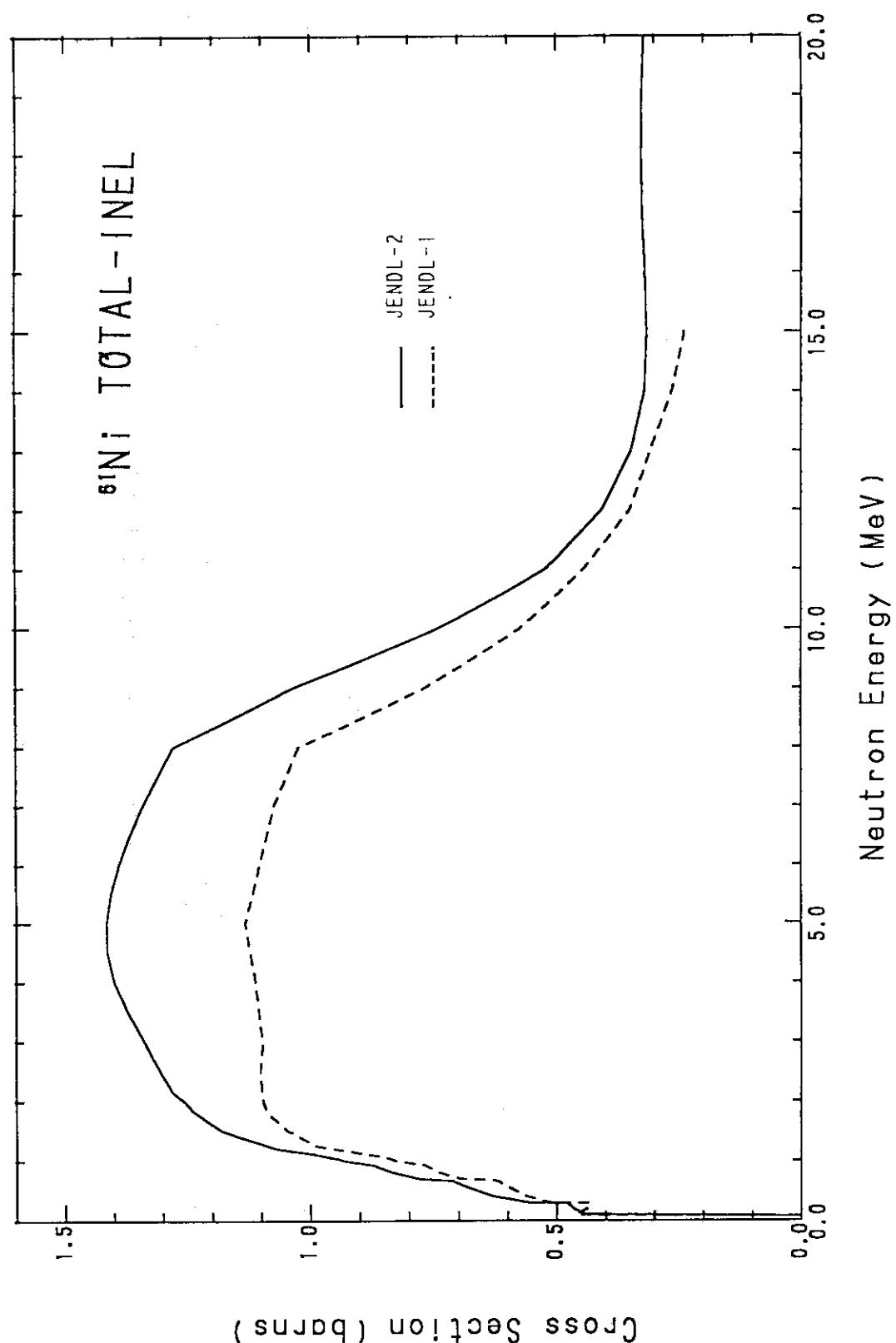
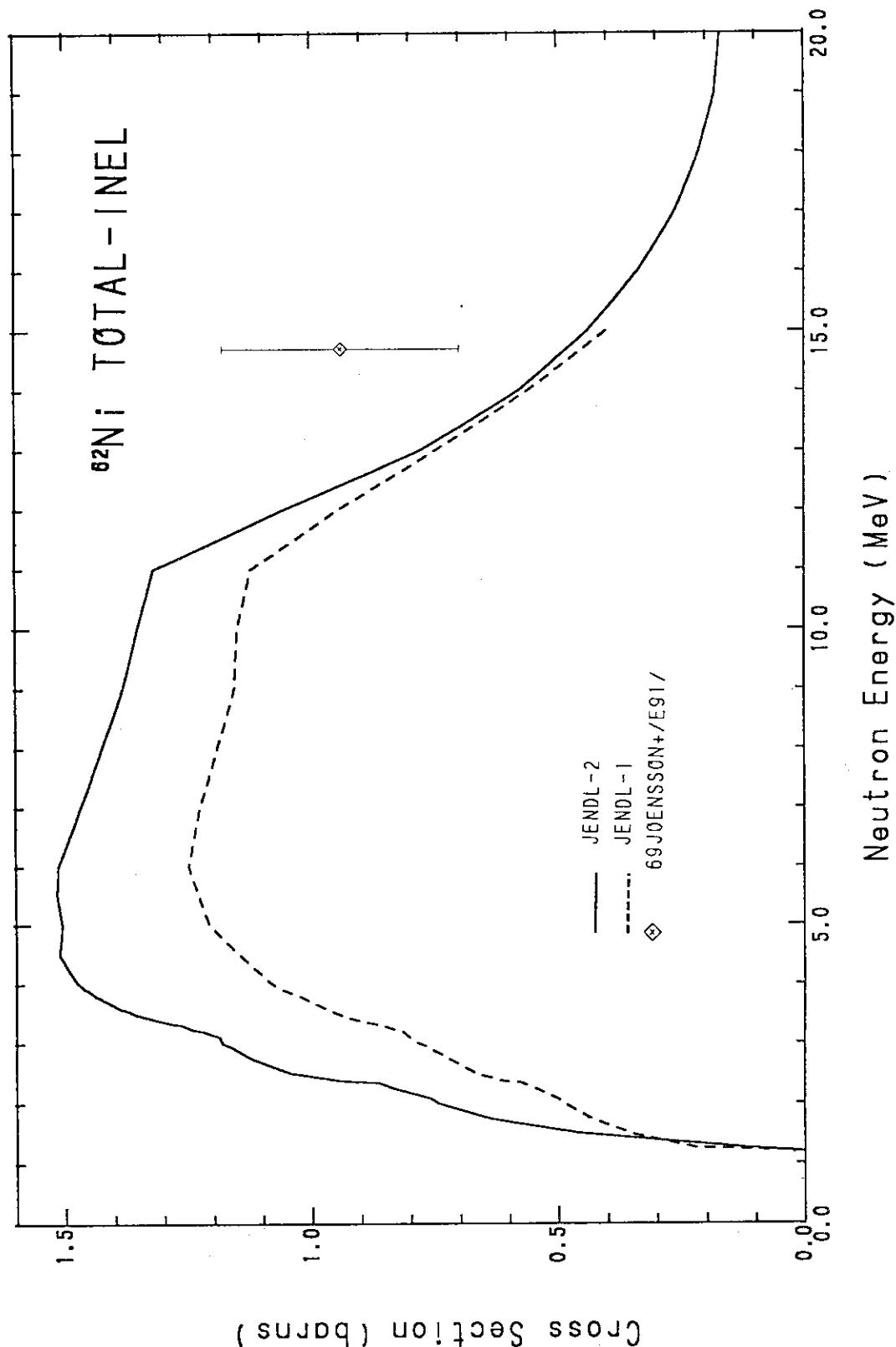
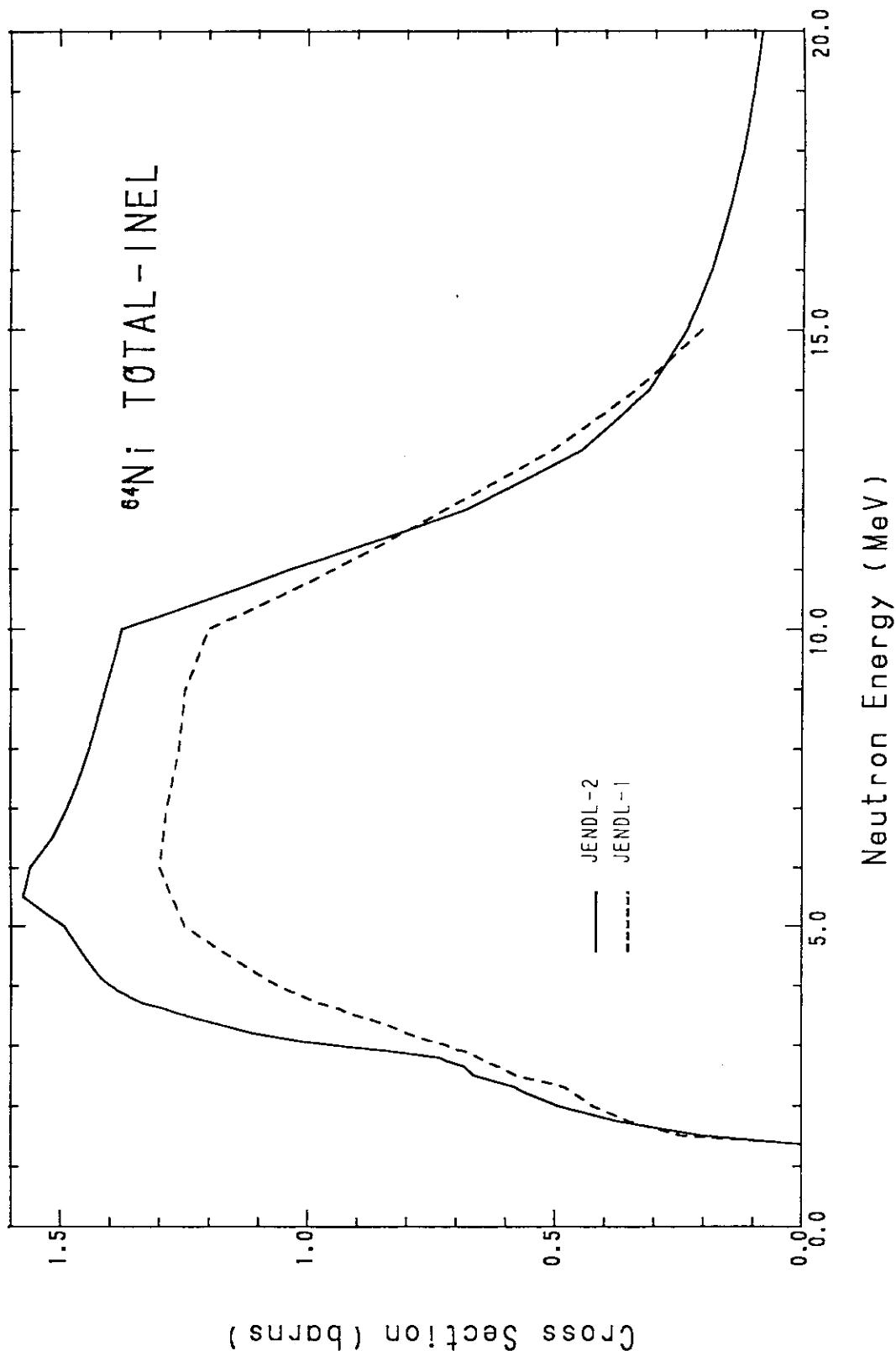


Fig. 46 Total inelastic scattering of ^{61}Ni .

Fig. 47 Total inelastic scattering of ^{62}Ni .

Fig. 48 Total inelastic scattering of ^{64}Ni .

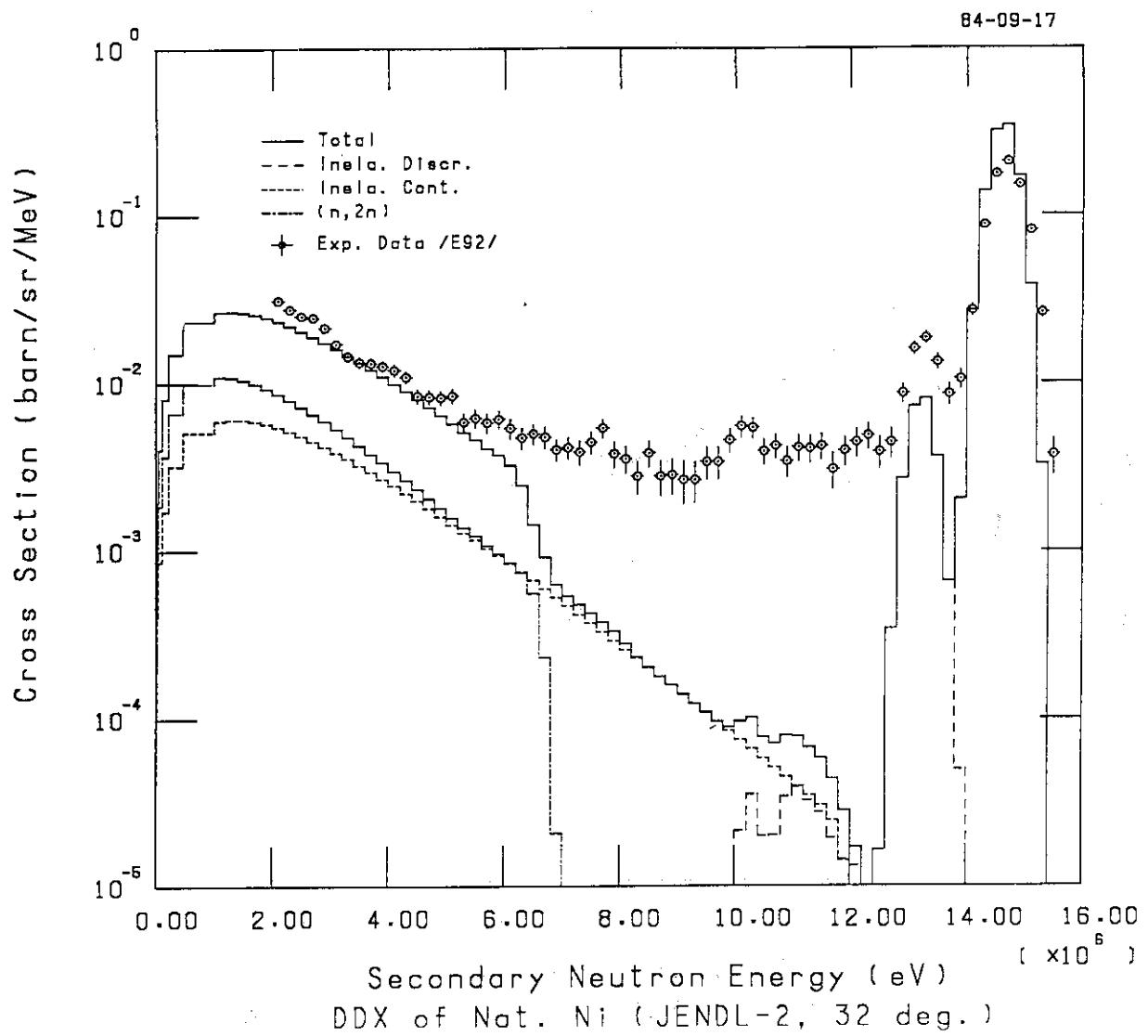


Fig. 49(a) Energy-angle double differential cross section of natural nickel at 32°

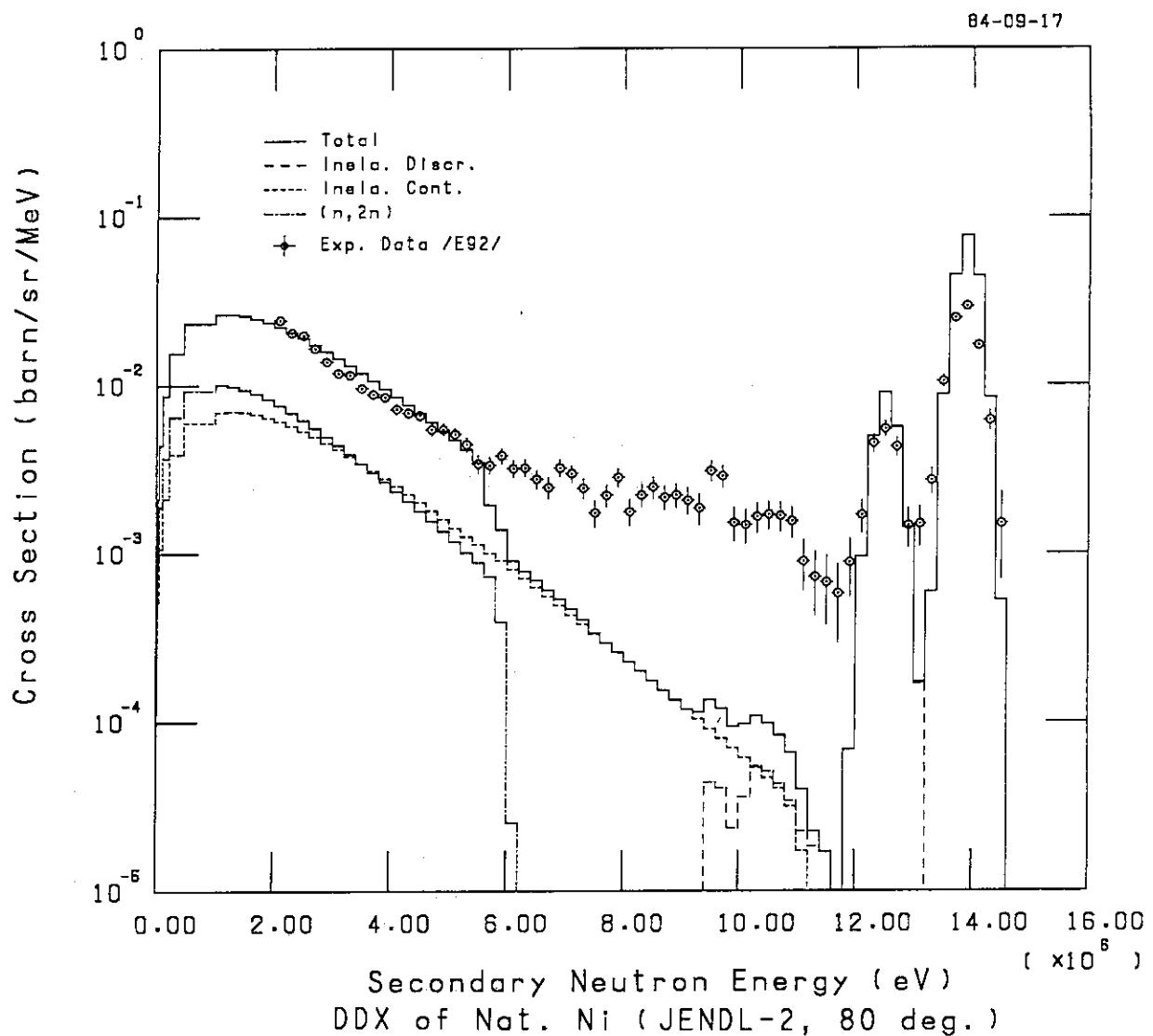


Fig. 49(b) Energy-angle double differential cross section
of natural nickel at 80°

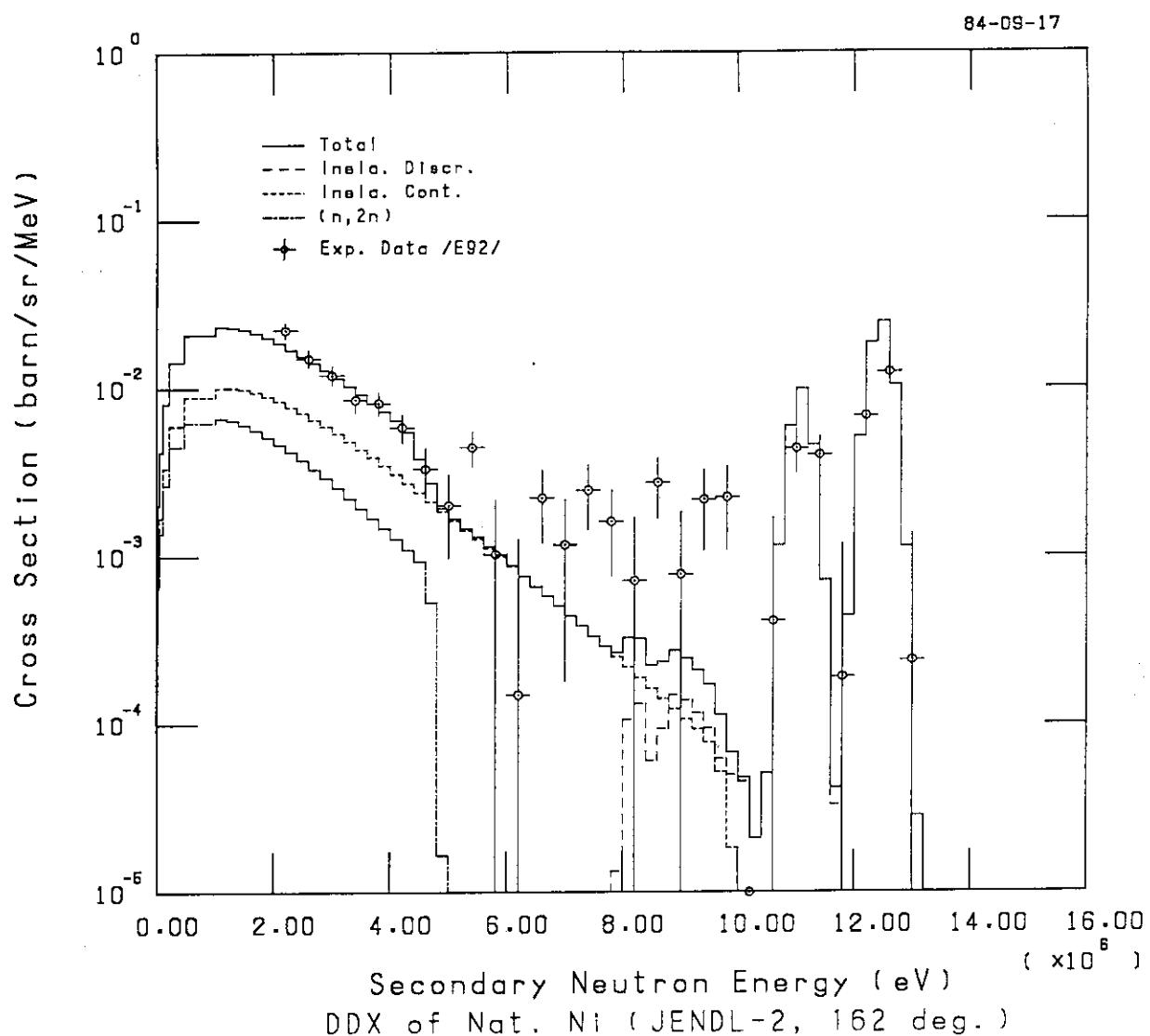
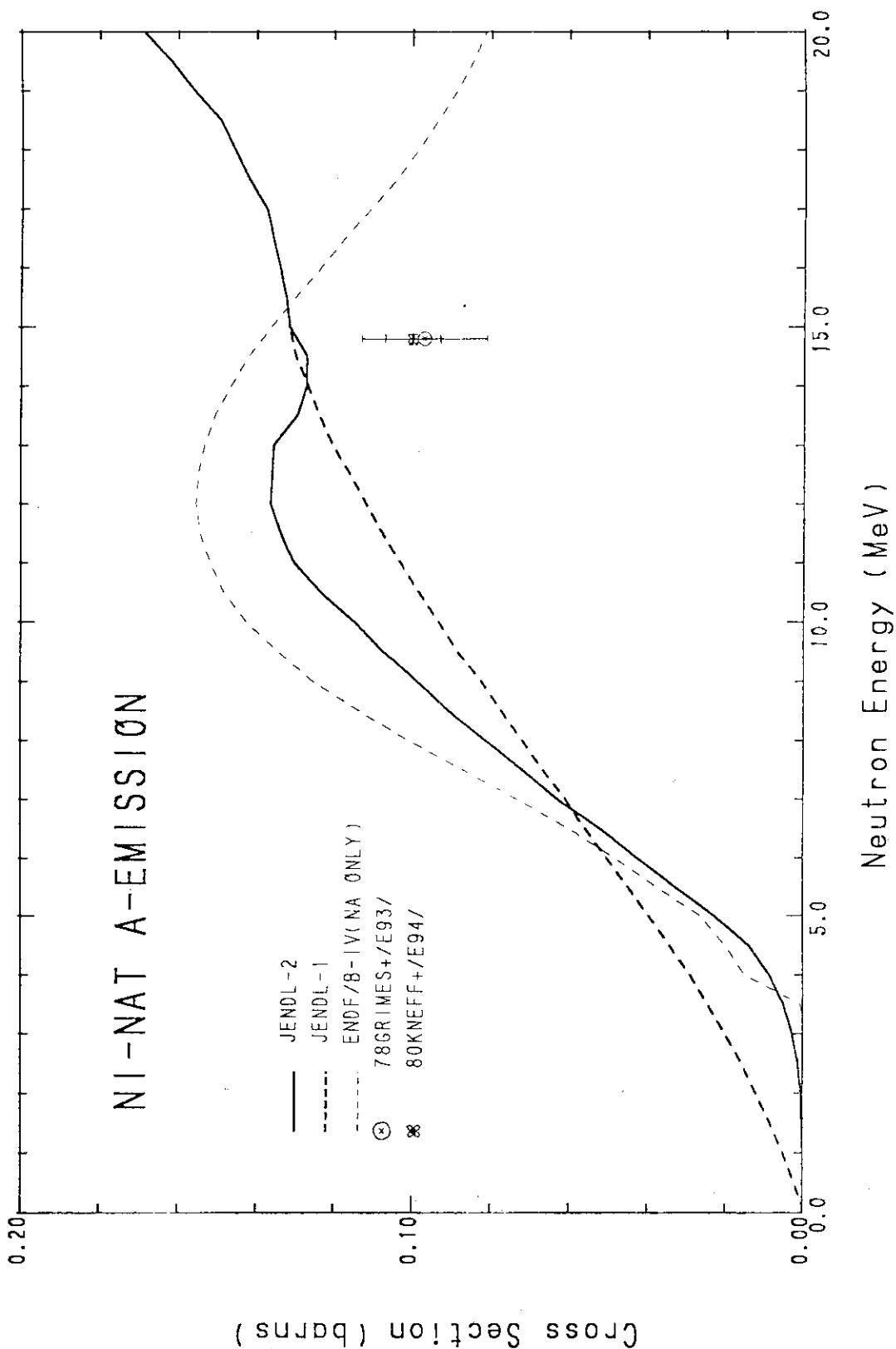


Fig. 49(c) Energy-angle double differential cross section of natural nickel at 162°

Fig. 50 α -emission cross section of natural nickel.

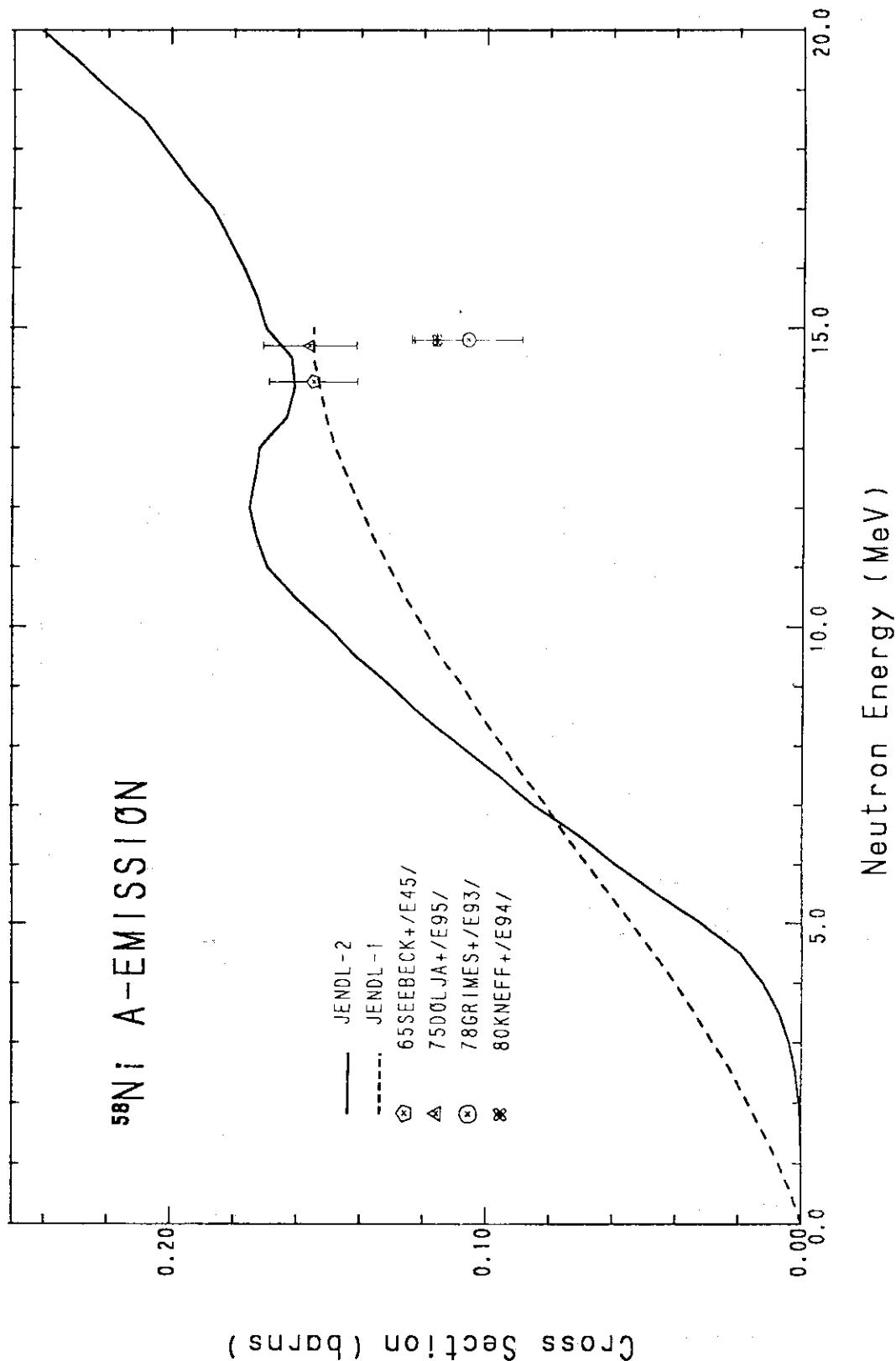
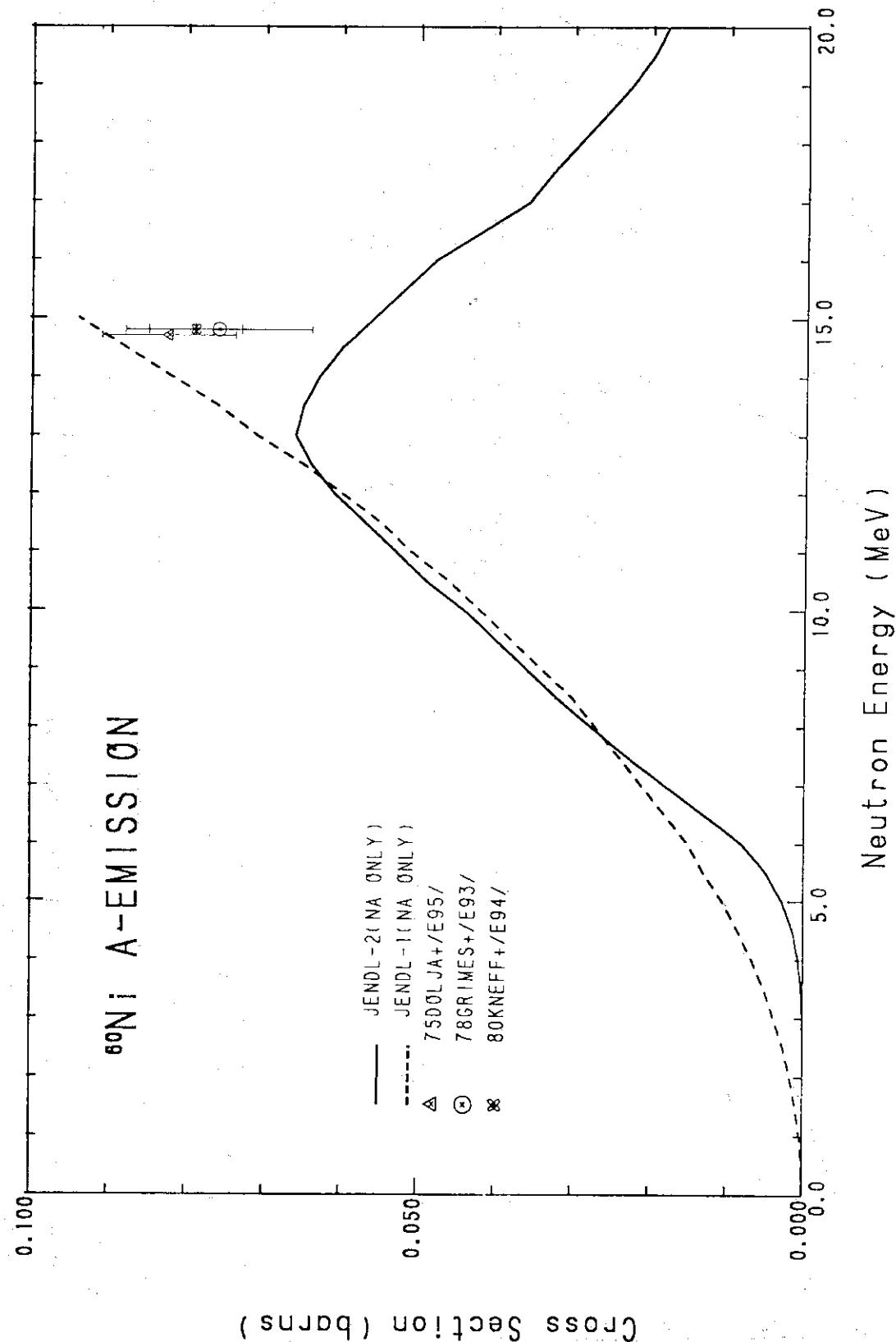
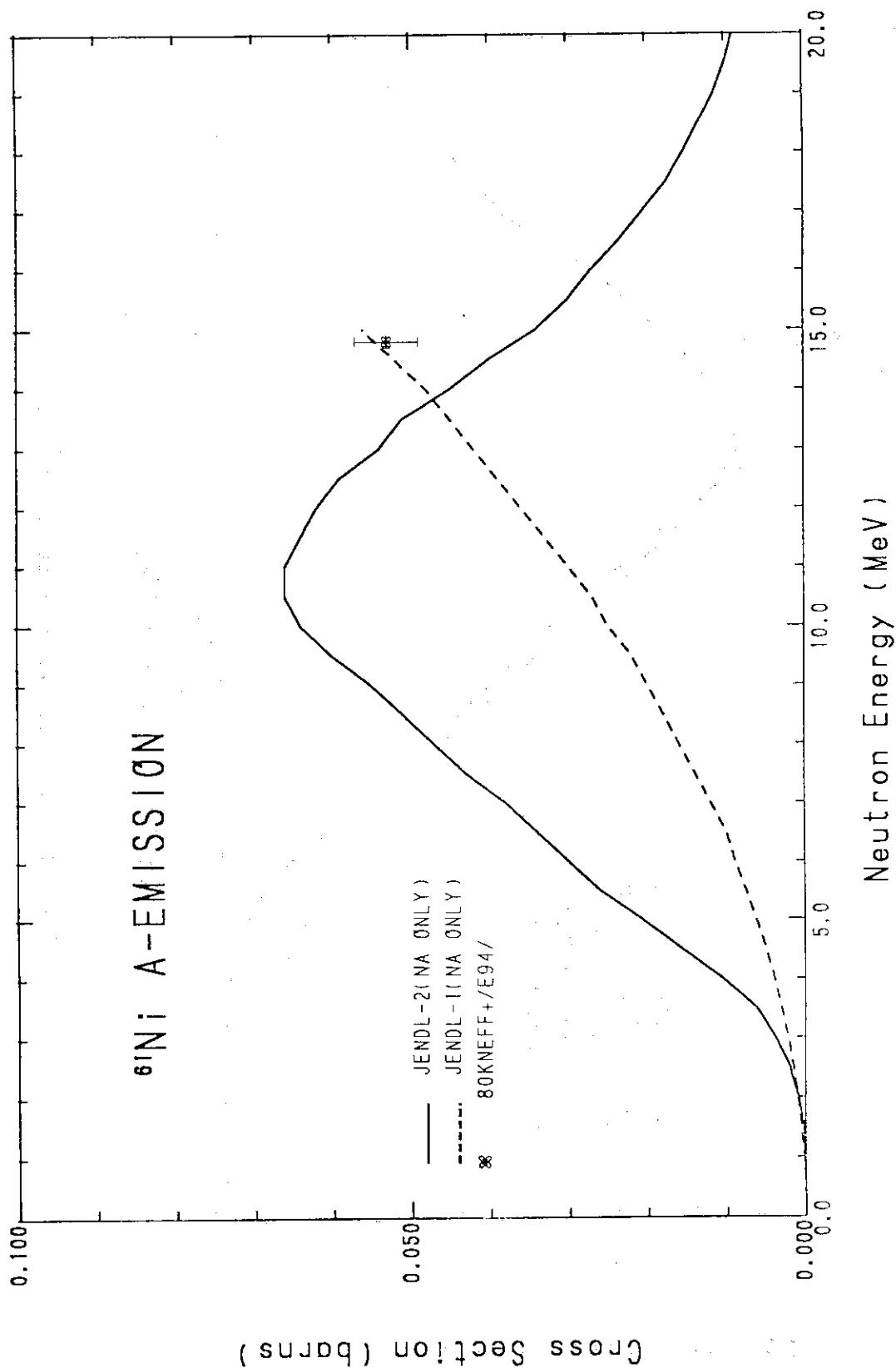
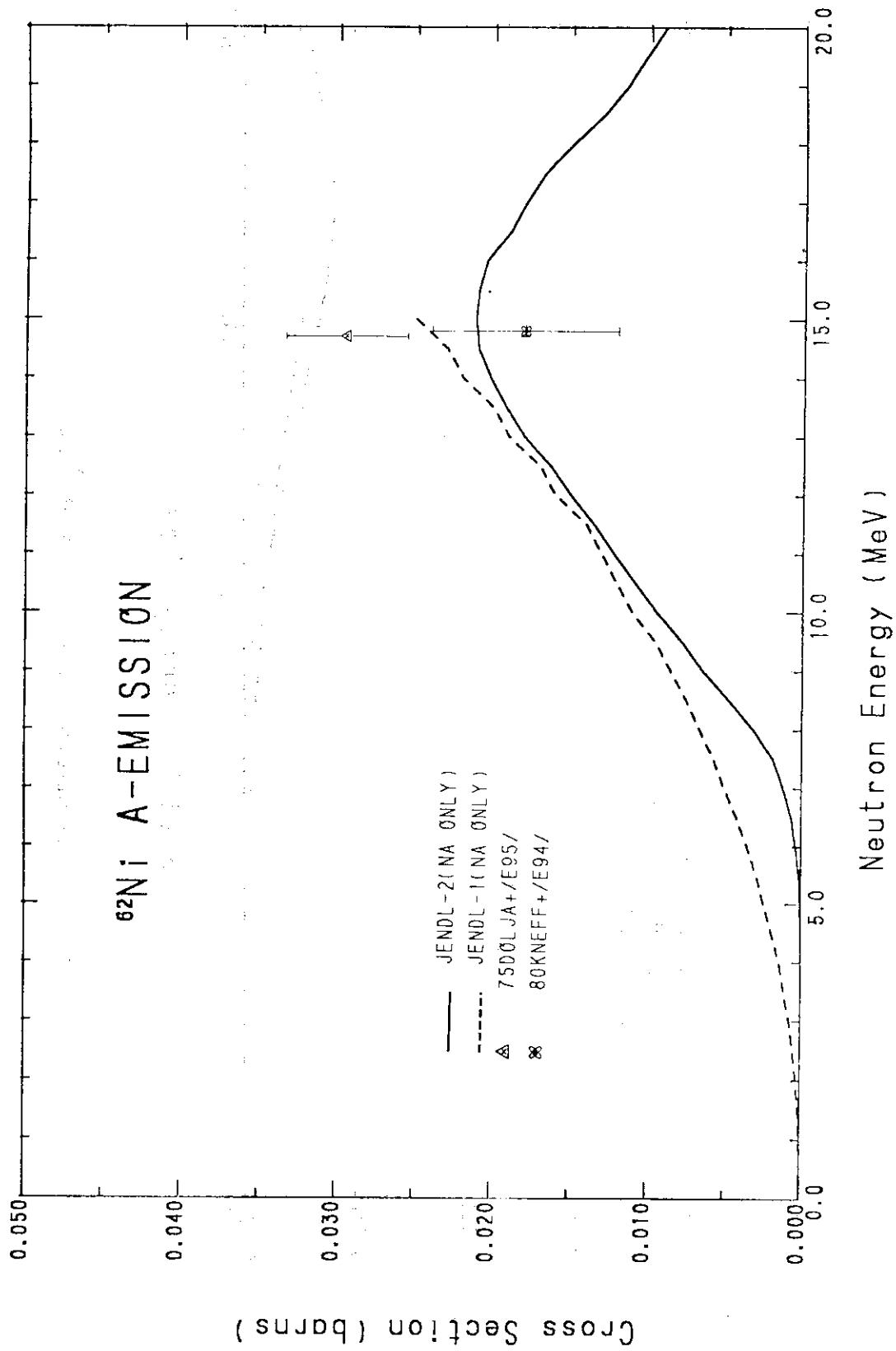
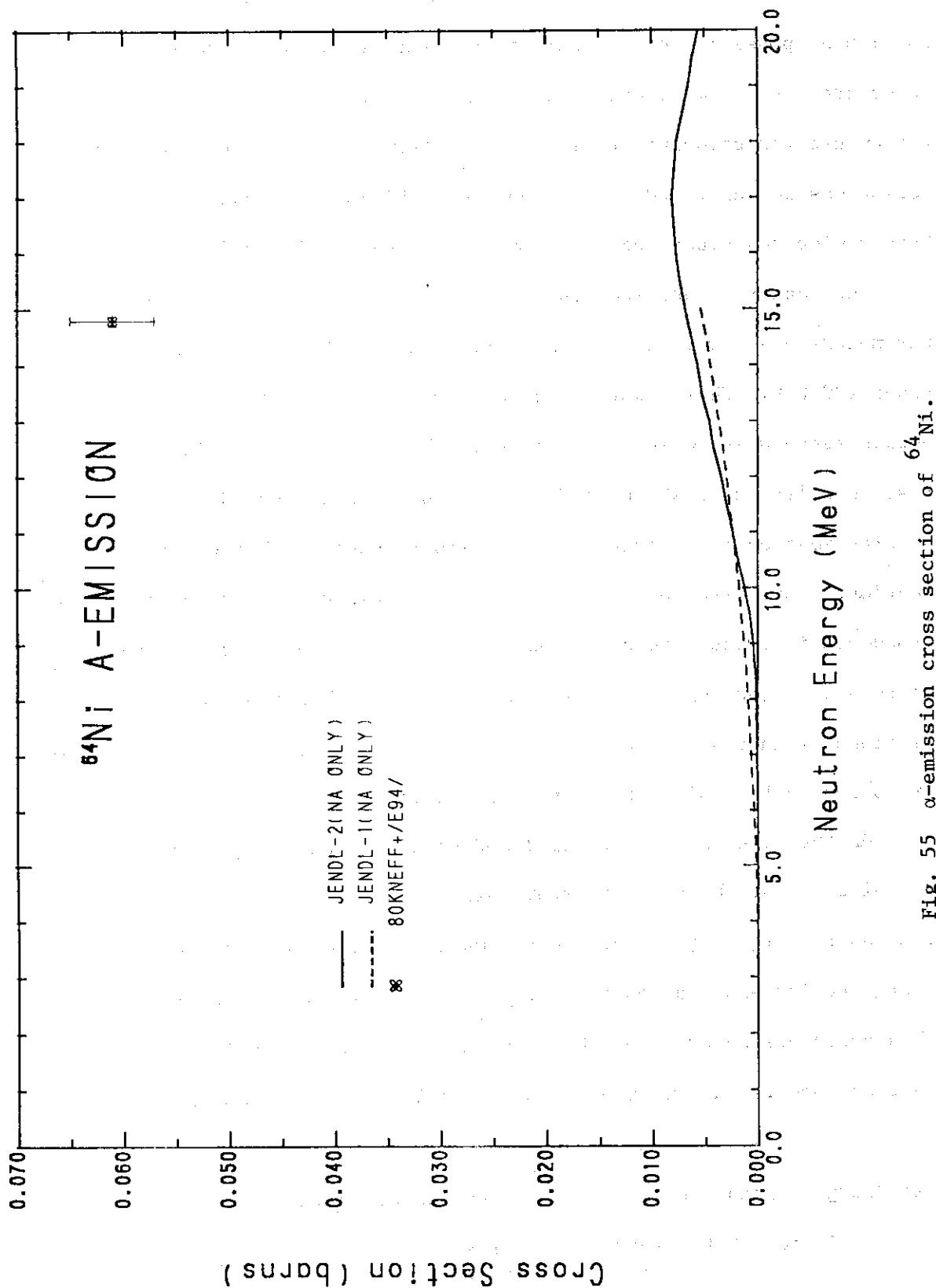


Fig. 51 α-emission cross section of ^{58}Ni .

Fig. 52 α -emission cross section of ^{60}Ni .

Fig. 53 α -emission cross section of ^{61}Ni .

Fig. 54 α -emission cross section of ^{62}Ni .

Fig. 55 α -emission cross section of ^{64}Ni .

Appendix Background Cross Sections

It is desirable to give the resonance parameters up to the energy as high as possible for the structural materials, since the resonance structure has an important role in the self-shielding effects even in the unresolved resonance region up to a few MeV. Hence the resonance region was set up to 600 keV except for ^{61}Ni in the present evaluation. Such a wide resonance region, however, causes the following problems.

The capture cross sections calculated from these resonance parameters are lower than the measured data in the energy range above about 200 keV. This underestimation comes from the level missing of the p-wave resonances, which is obvious in the staircase plotting of resonance levels as shown in Figs. A1 ~ A5. We corrected this underestimation by applying a slight smooth positive background cross section.² Some disagreement between the calculated and the experimental cross sections was also corrected by the background cross section instead of adjusting the resonance parameters. The capture cross sections of natural nickel with and without the background cross section are shown in Fig. A6 with the measured data.

On the other hand, the total and elastic scattering cross sections calculated from the present parameters are underestimated slightly in the lower energy region and overestimated considerably in the higher energy region above a few tens of keV. Hence we have investigated why this anomalous behavior of the total and elastic scattering cross sections occurs, and we have found the following two reasons:

A) Energy Dependence of Effective Scattering Radius

In the ENDF/B format, the effective scattering radius R is

required to be constant through the resolved resonance region. For a wide energy range such as up to 600 keV, however, the effective scattering radius is not constant but energy dependent. The optical model calculation shows that the radius of Ni isotopes decreases considerably with increase of the neutron energy as shown in Figs. A7 ~ A11. The radius decreases often down to a factor of 0.7 at 600 keV. It is therefore evident that the constant radius approximation causes considerable overestimation in the higher energy region. However, this effect is not sufficient to explain the overestimate in the higher energy region.

B) Truncation Effect of Finite Resonances

The resonance shape of the elastic scattering cross section is asymmetric as shown in Fig. A12 because of the interference between the resonance and potential scattering. Hence its contribution is positive in the higher off-resonance energy region and negative in the lower energy region. Consider an energy point. If there are many resonances both in higher and lower energy region as in the case of actual nuclei, the positive and negative contributions cancel out at this energy point.

In the evaluated data file, however, we take a finite number of resonances. Hence all the contributions of distant resonance levels are positive near the upper boundary of the resonance region, and are negative near the lower boundary. This situation is schematically shown in Fig. A12.

In order to know how much this effect is, the cross section of ^{58}Ni was calculated by removing the resonance levels below 400 keV. The results are compared with those without removal of levels in

Fig. A13. The cross section value is reduced more than 20% in the off-resonance energy region. It is found that the truncation effect is as much as the effect of the energy dependence of the effective scattering radius described above.

It is revealed from the present study that the overestimation of the total and elastic scattering cross sections in the higher energy region is inevitable if we use the constant scattering radius. How should this overestimation be corrected? Applying the background cross section is a common way. In the present case, however, the background correction is very difficult particularly for the isotopes from the following reason: The overestimation becomes more than 3 barns at the off-resonance regions above 400 keV. On the other hand, the cross section minimum due to the interference often becomes as low as 0.5 barns. Therefore a smooth negative background correction causes negative cross section values at the energies of the cross section minima.

Consequently the background cross section must have strong energy dependence. It is a hard job to determine such an energy dependent background cross section, as so many resonance levels exist in the energy region considered.

To avoid this difficulty, we adopted the energy dependent effective scattering radius by modifying the ENDF/B format for internal use. We found that the overestimation could disappear with the following energy dependent radius:

$$\begin{aligned}
 R \text{ (fm)} &= 8.11 - 5.9 \times E_n \text{ (MeV)} \quad \text{for } {}^{58}\text{Ni}, \\
 &= 7.0 - 5.0 \times E_n \text{ (MeV)} \quad \text{for } {}^{60}\text{Ni}, \\
 &= 6.4 - 8.3 \times E_n \text{ (MeV)} \quad \text{for } {}^{61}\text{Ni}, \\
 &= 7.66 - 4.29 \times E_n \text{ (MeV)} \quad \text{for } {}^{62}\text{Ni}, \\
 &= 7.37 - 3.7 \times E_n \text{ (MeV)} \quad \text{for } {}^{64}\text{Ni}.
 \end{aligned}$$

The present radius is also shown in Figs. A7 ~ A11. The solid line in Fig. A13 shows the cross section calculated with the energy dependent radius.

The energy dependent radius is not allowed, however, in the current ENDF/B format. We made a proposal^{A1)} to modify the ENDF/B format so as to accept the energy dependent effective scattering radius. At present we took the difference between the energy-dependent and constant radius calculations as the background cross section. Consequently, the background cross section has a resonance-like structure. Such a strongly energy-dependent background cross section, however, might distort the Doppler broadened cross section, if it is calculated directly from the resonance parameters and the background cross section. As to the natural nickel, the background cross section was produced by the eye-guide method before the present study. The eye-guide method was possible, because the cross section minimums are not so low as those of the isotopes.

Figures A14-A19 show the background cross section of natural nickel and the isotopes.

References

- A1) Kikuchi Y.: "Nuclear Data for Structural Materials", Proc. IAEA Consultants' Meeting, Vienna, 2-4 Nov. 1983, p.169, INDC(NDS)-152/L (1984).

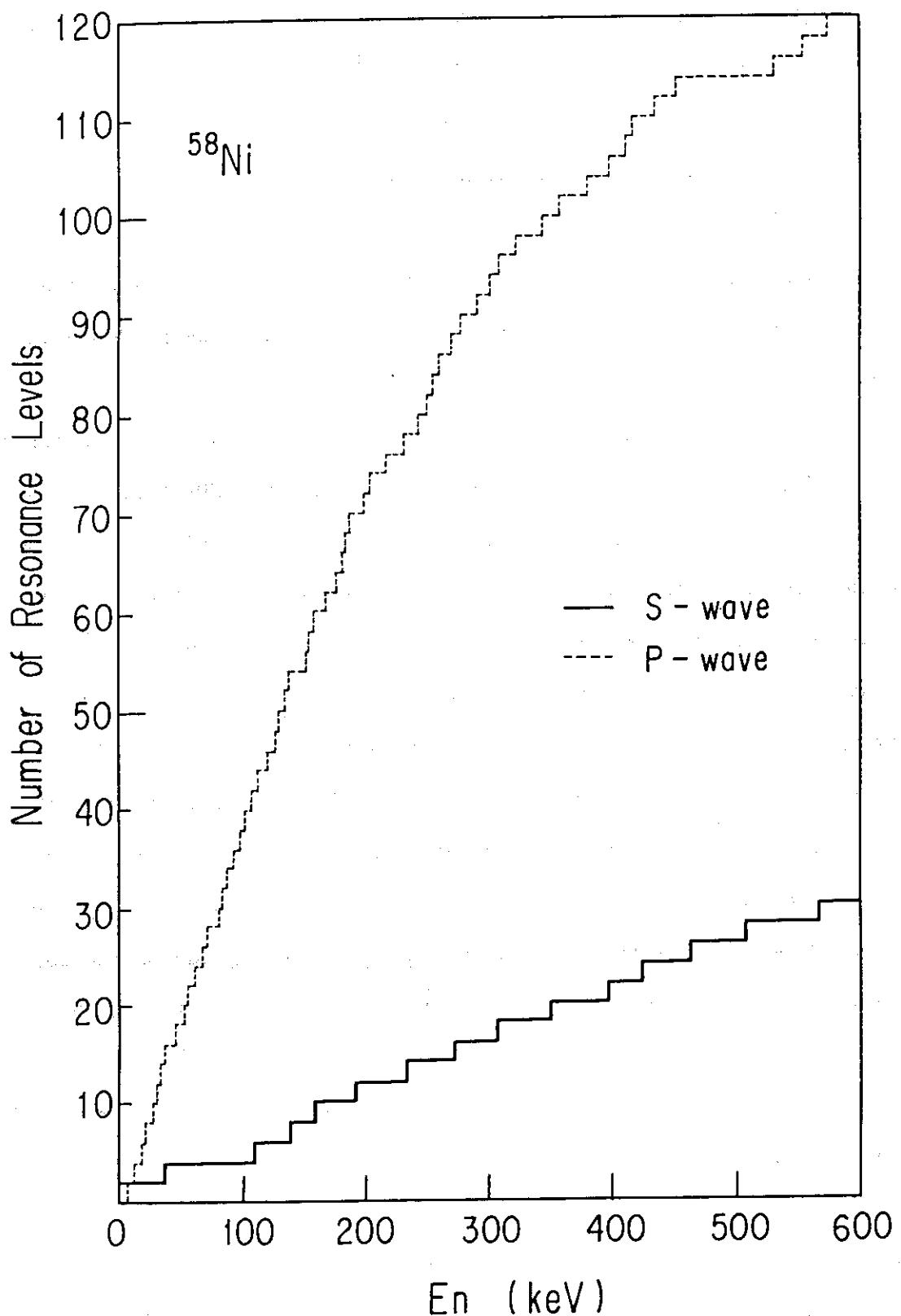


Fig.A.1 Staircase plotting of resonance levels of ^{58}Ni .

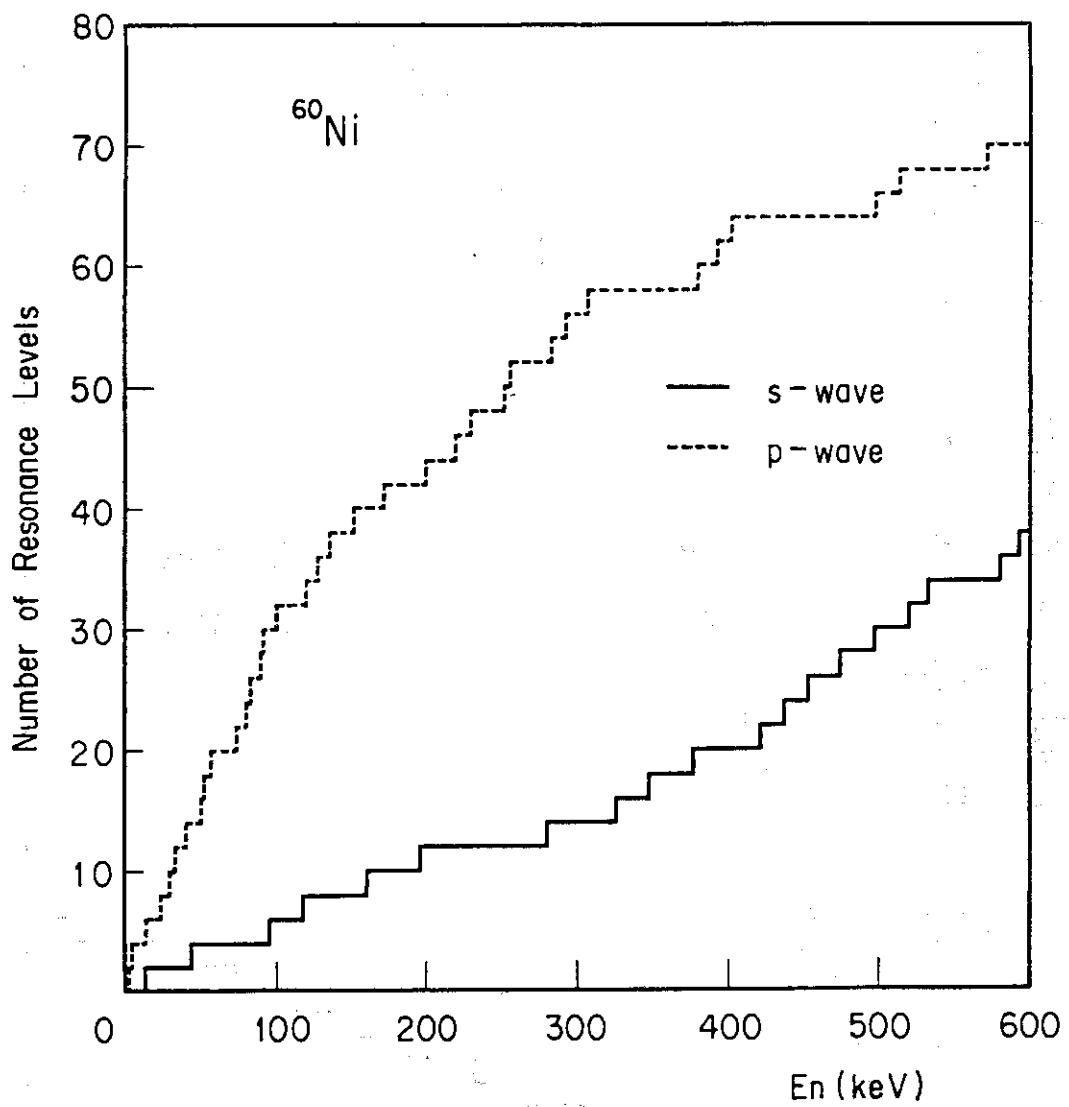


Fig.A.2 Staircase plotting of resonance levels of ^{60}Ni .

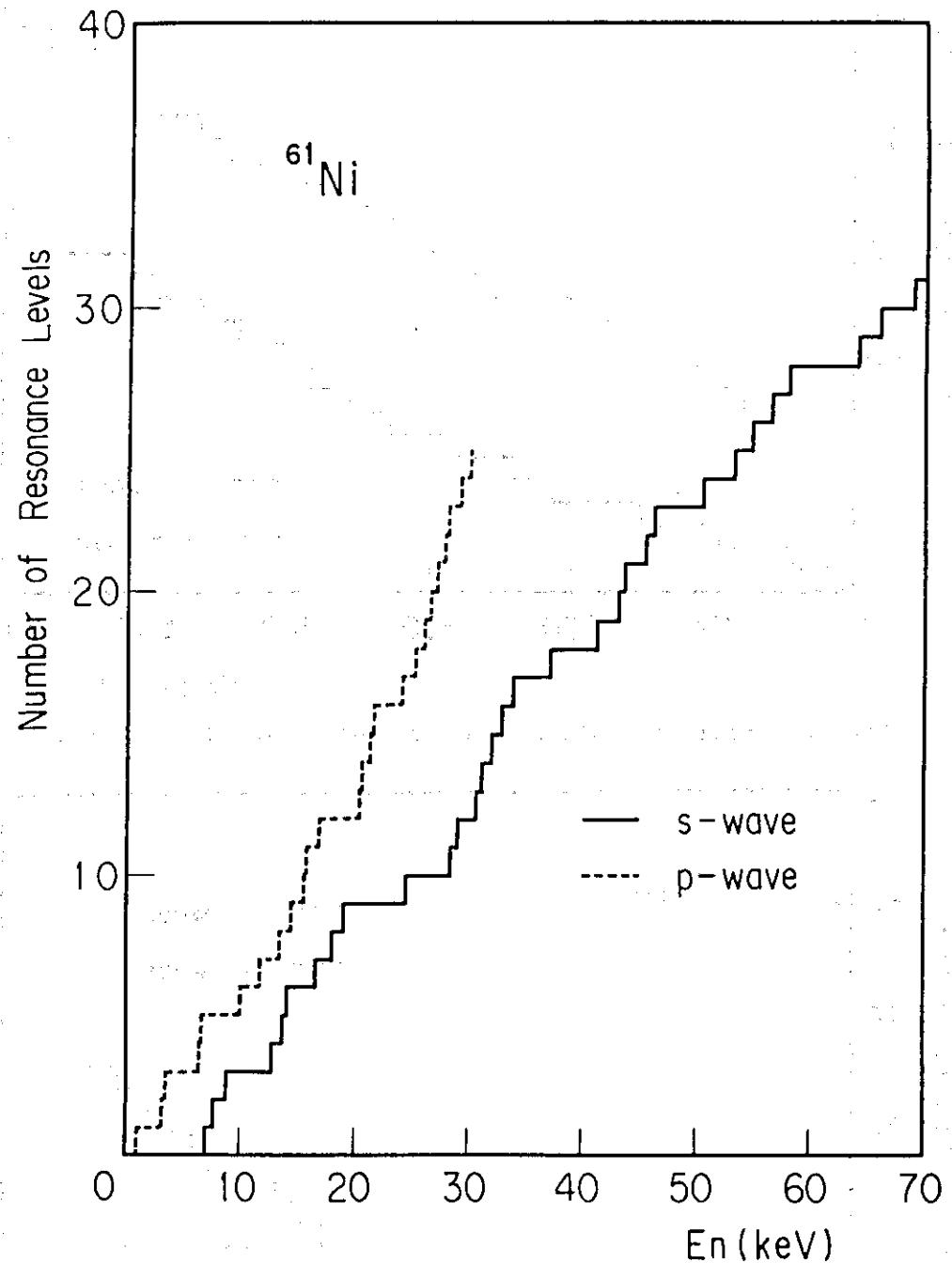
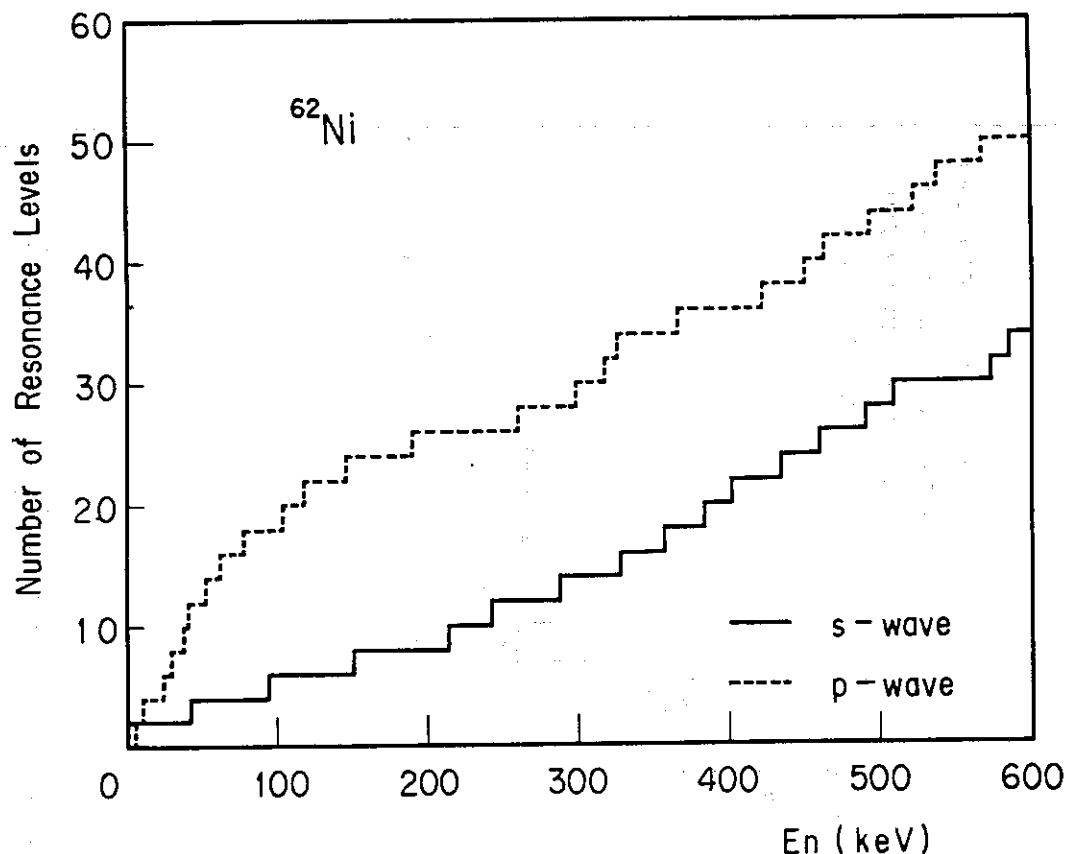
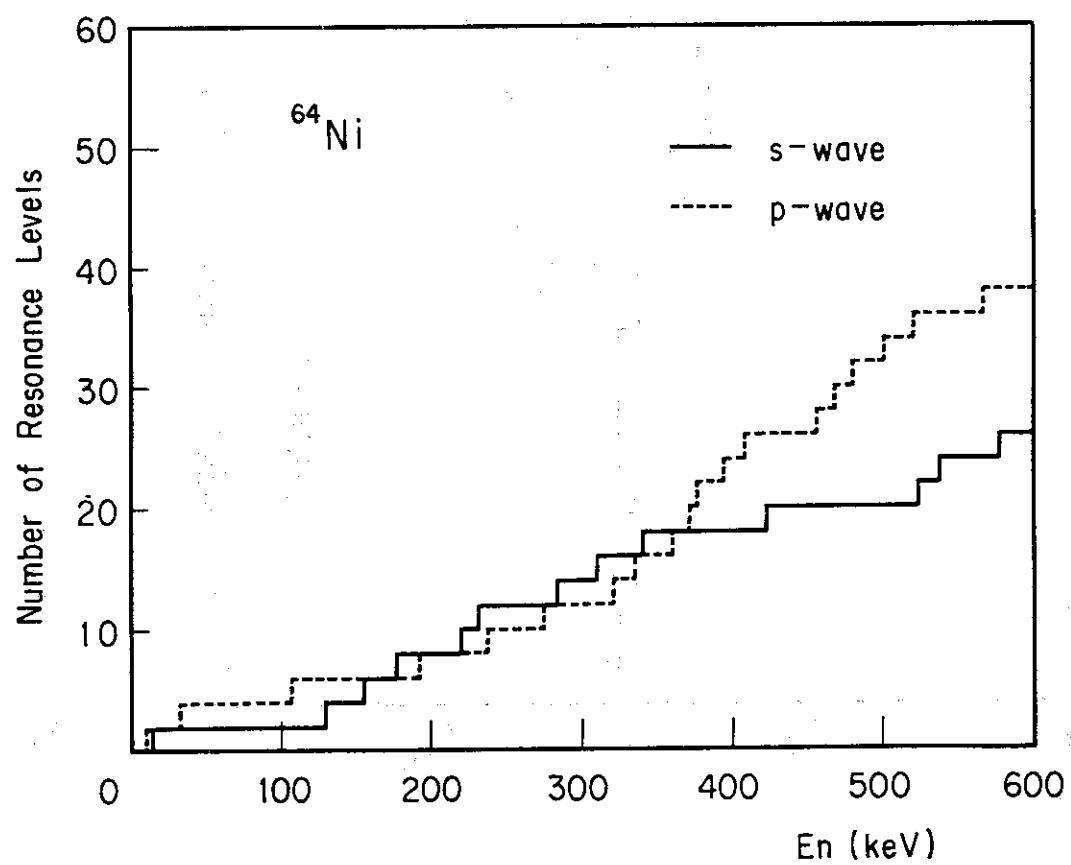


Fig.A.3 Staircase plotting of resonance levels of ^{61}Ni .

Fig.A.4 Staircase plotting of resonance levels of ^{62}Ni .Fig.A.5 Staircase plotting of resonance levels of ^{64}Ni .

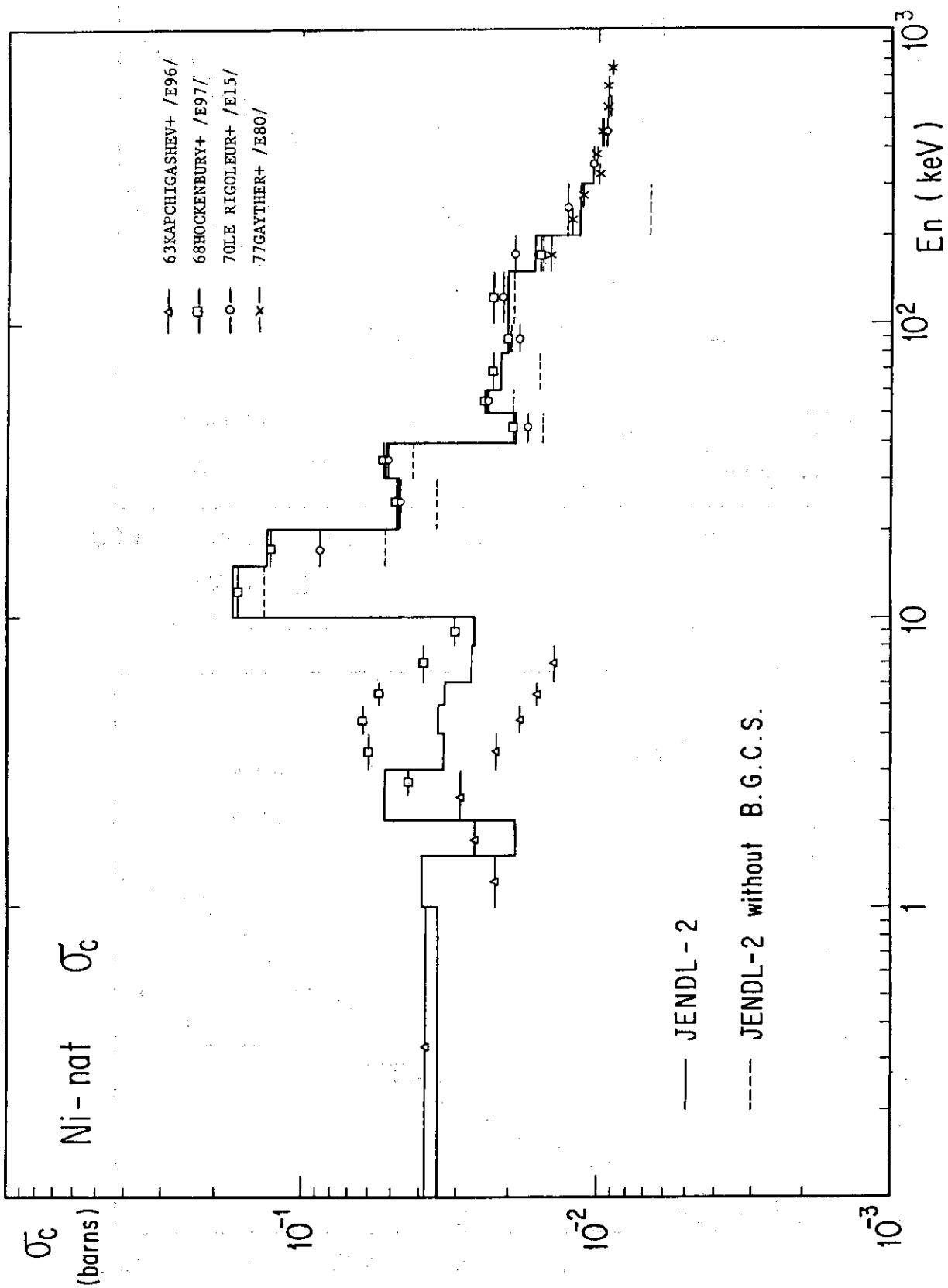
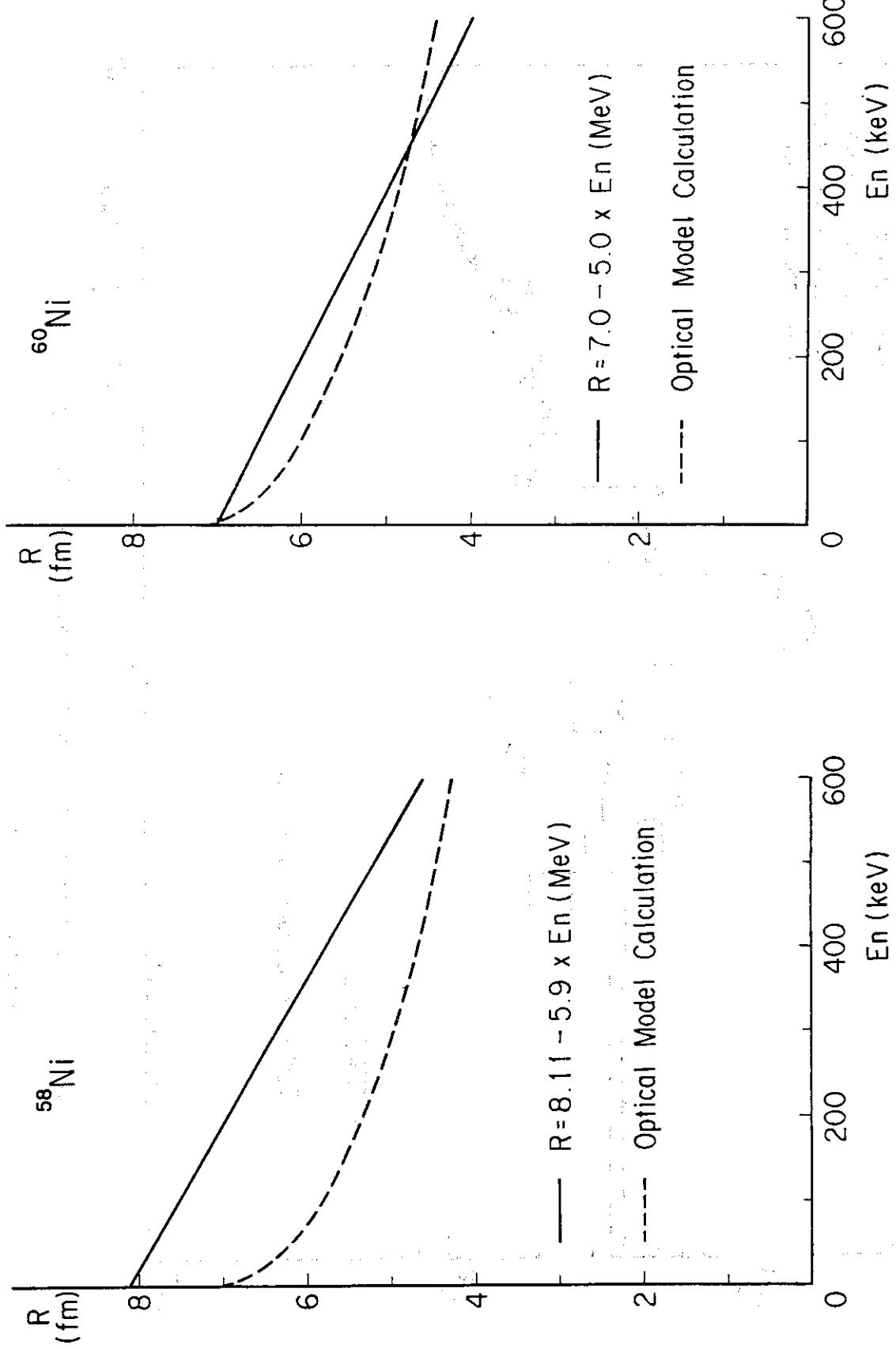


Fig.A.6 Average capture cross sections of natural nickel with and without background cross section.



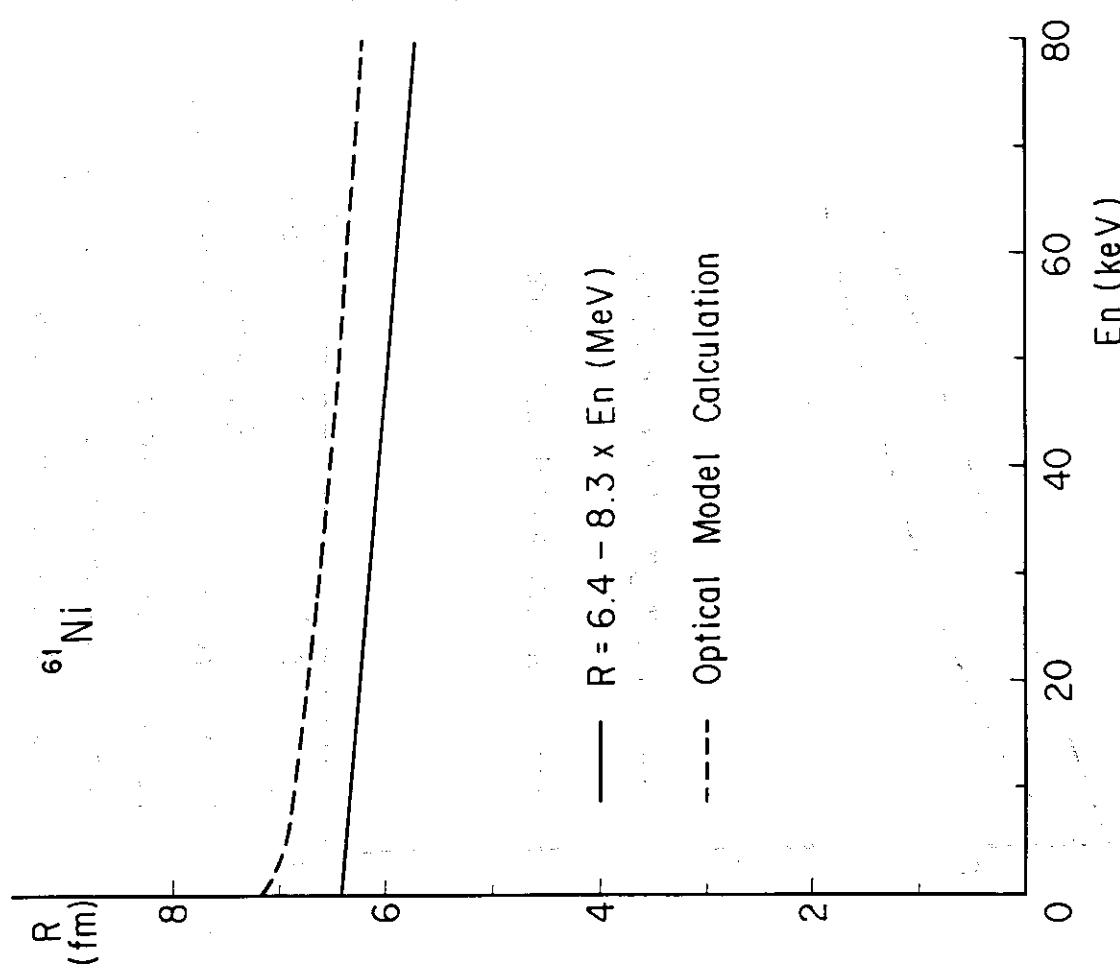


Fig.A.9 Energy dependence of the effective scattering radius of ^{61}Ni . The solid line is the adopted value in JENDL-2. The dashed line is calculated with the optical model.

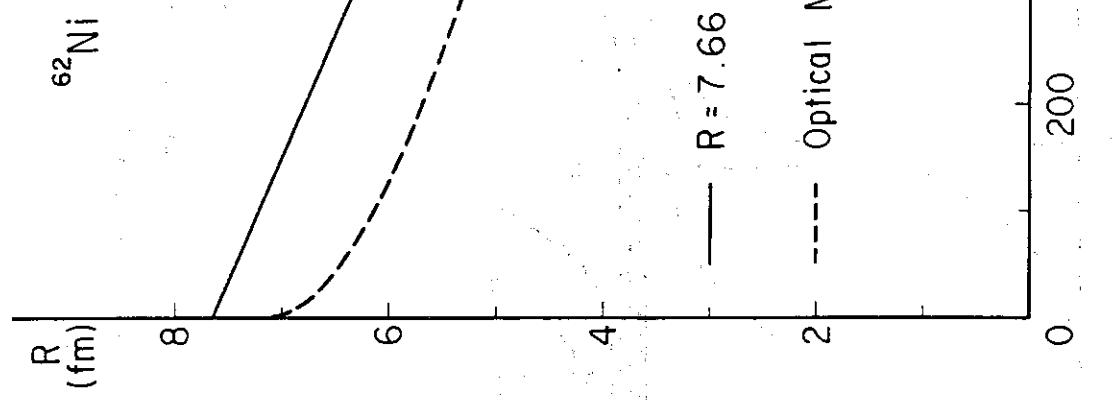
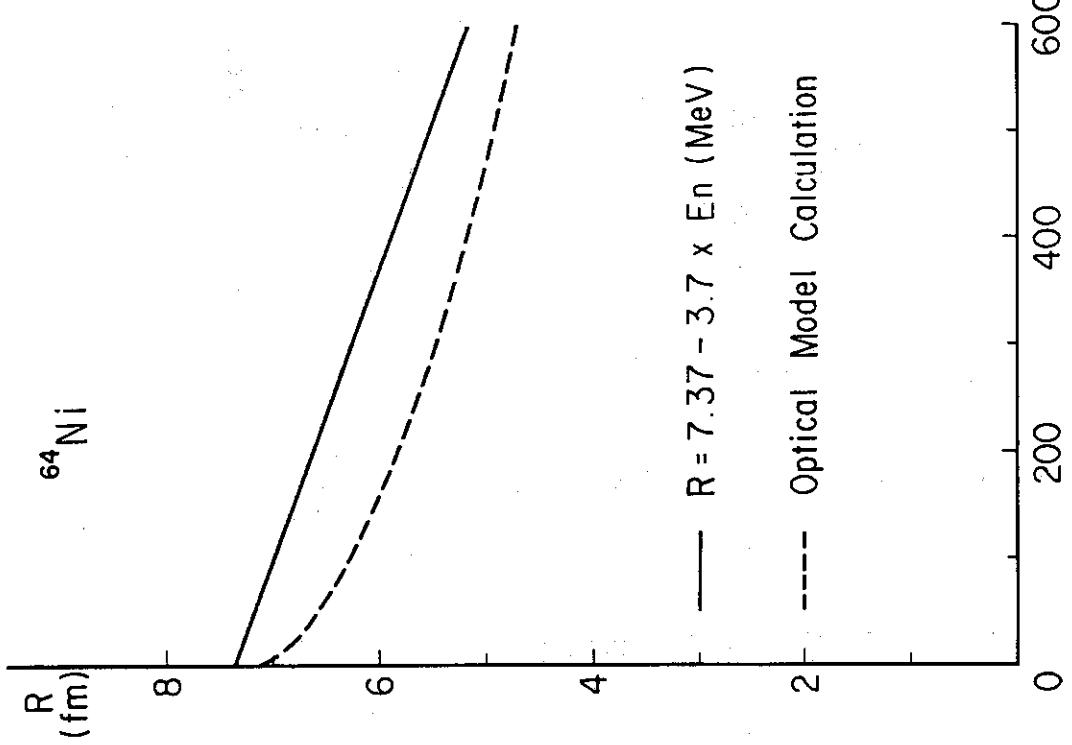


Fig.A.10 Energy dependence of the effective scattering radius of ^{62}Ni . The solid line is the adopted value in JENDL-2. The dashed line is calculated with the optical model.



- 188 -

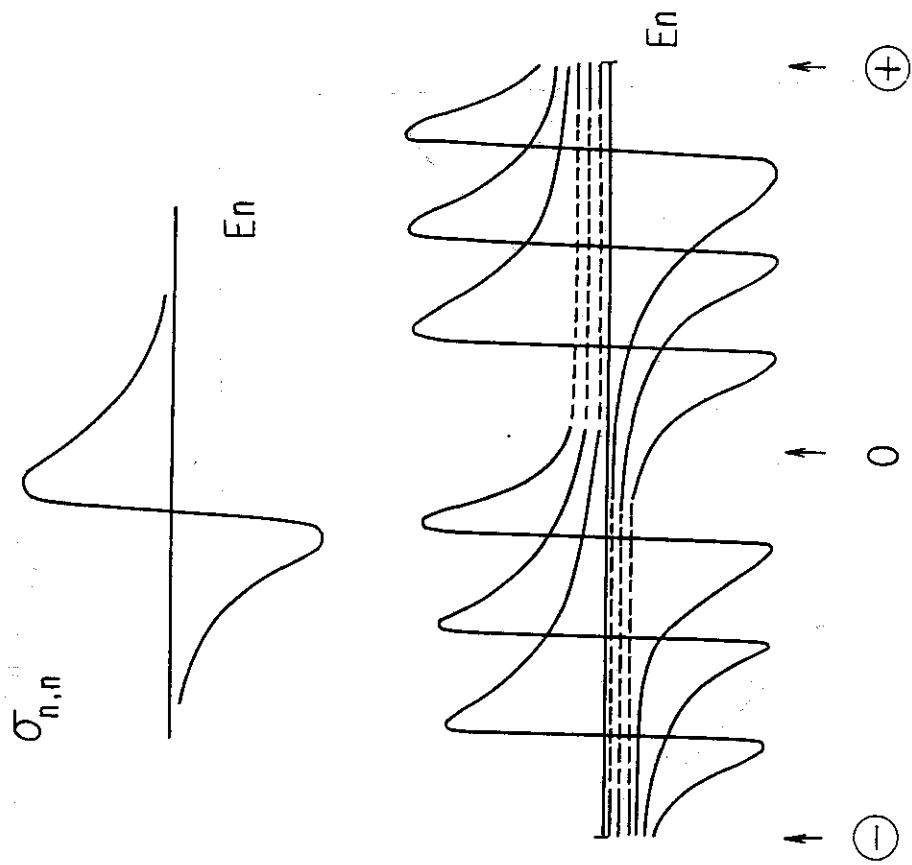


Fig. A.12 Shape of elastic scattering cross section (upper) and schematic view of the contribution from distant levels (lower).

Fig. A.11 Energy dependence of the effective scattering radius of ^{64}Ni . The solid line is the adopted value in JENDL-2. The dashed line was calculated with the optical model.

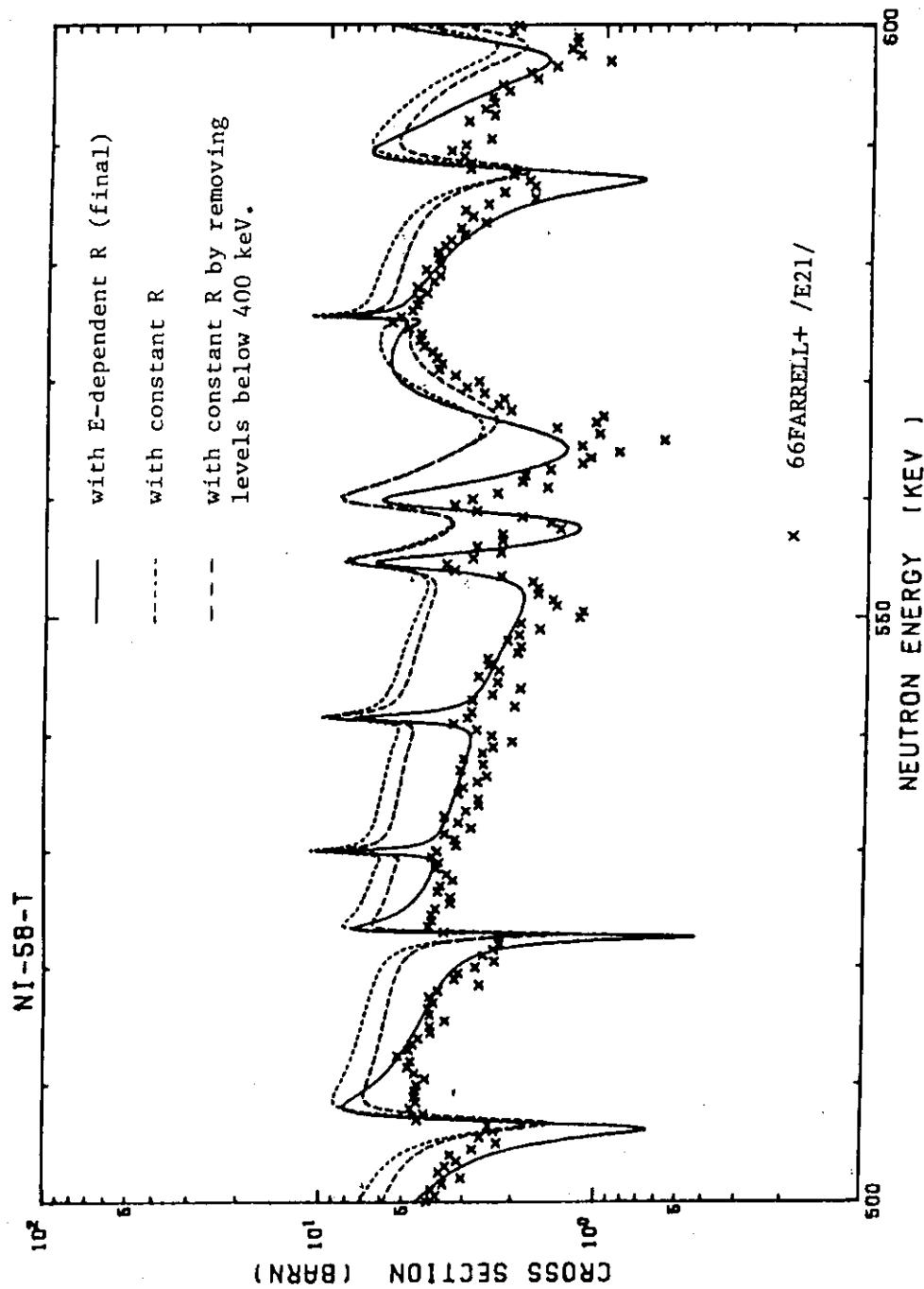


Fig.A.13 Total cross sections of ^{58}Ni . The solid line is calculated with the finally adopted energy dependent effective scattering radius and the dotted line with the constant radius. The dashed line is calculated with the constant radius by removing the resonances below 400 keV in order to know the truncation effect.

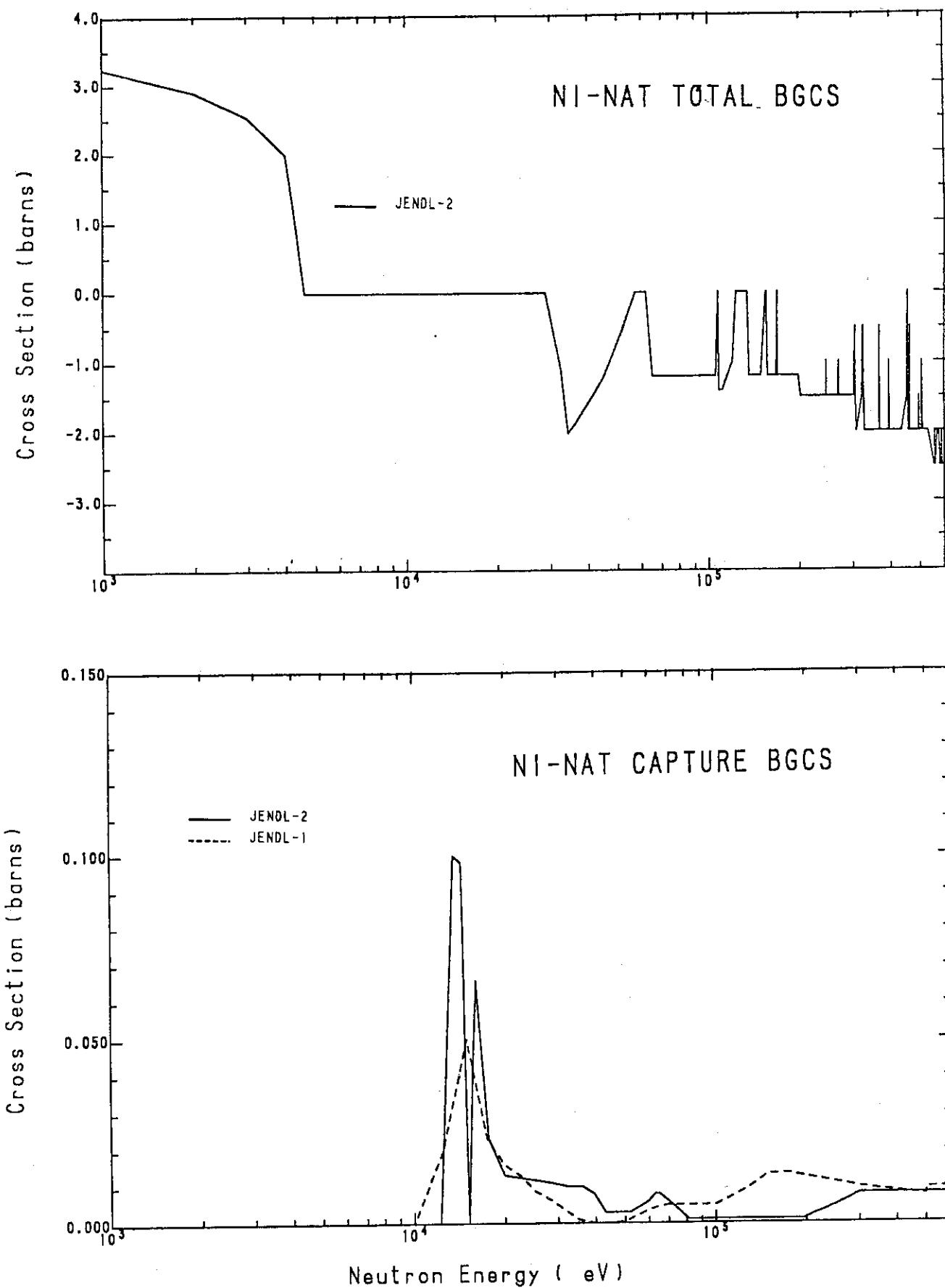


Fig.A.14 The total and capture background cross sections of natural nickel.

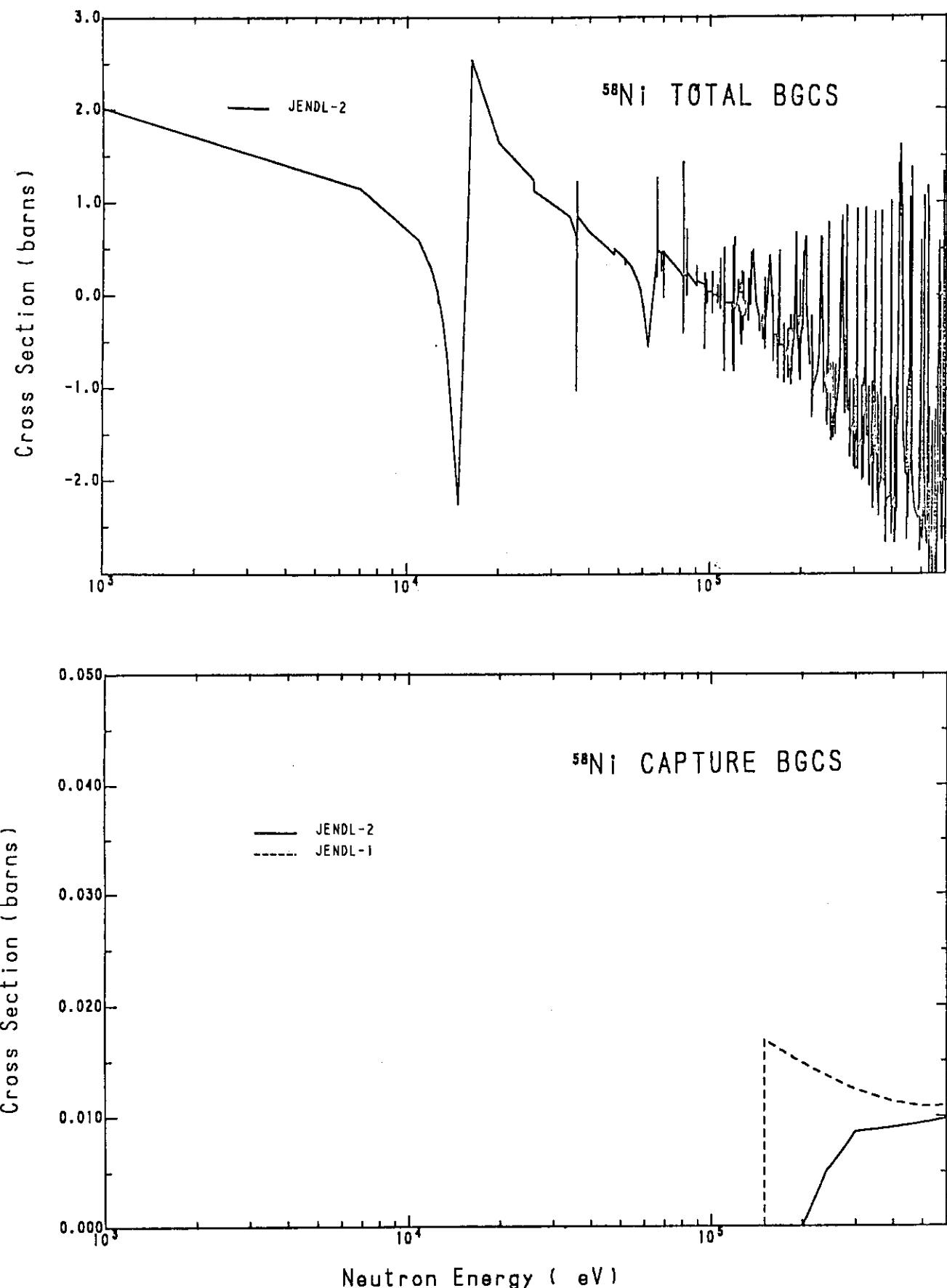
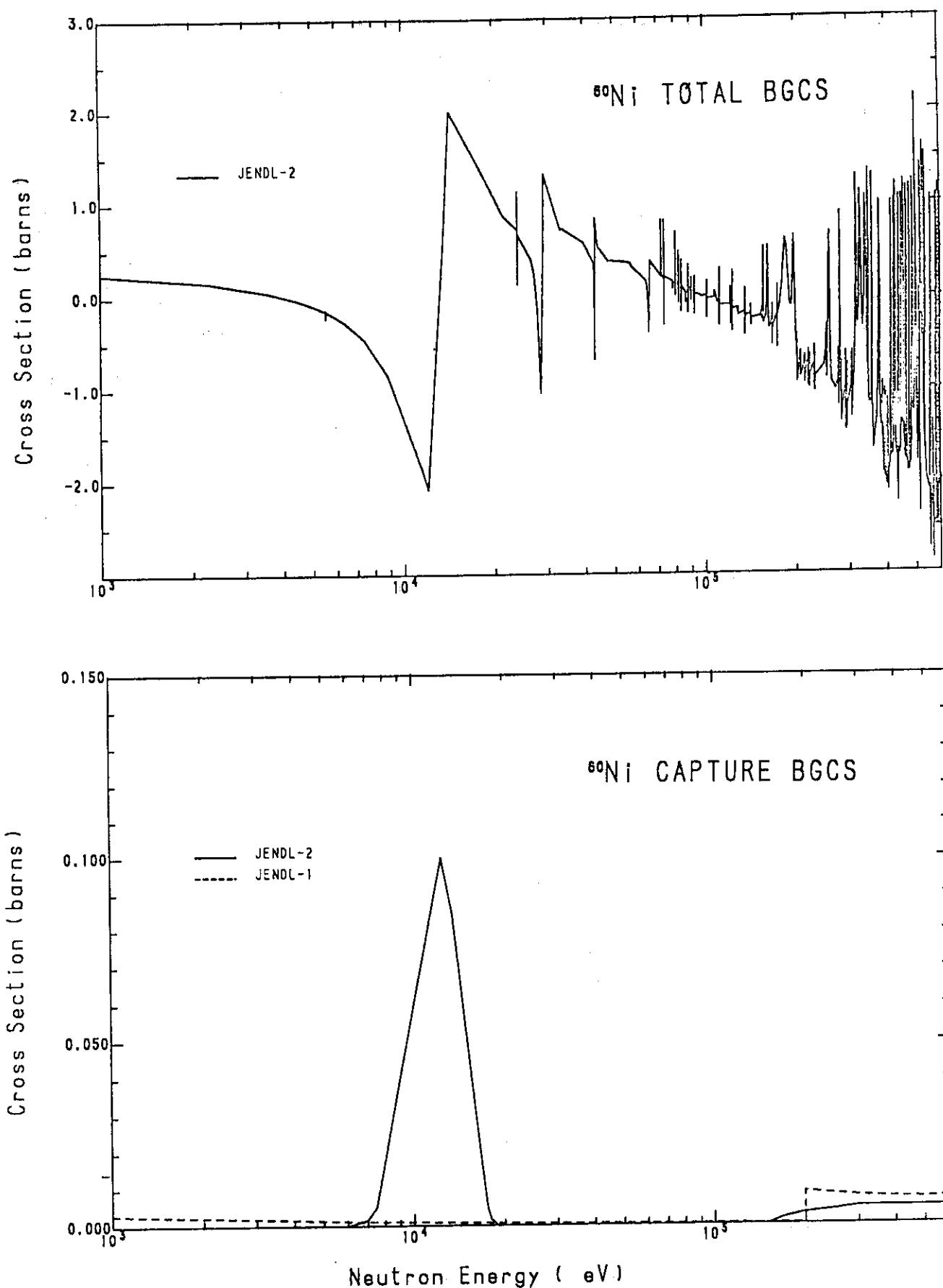


Fig.A.15 The total and capture background cross sections of ^{58}Ni .

Fig.A.16 The total and capture background cross sections of ^{60}Ni .

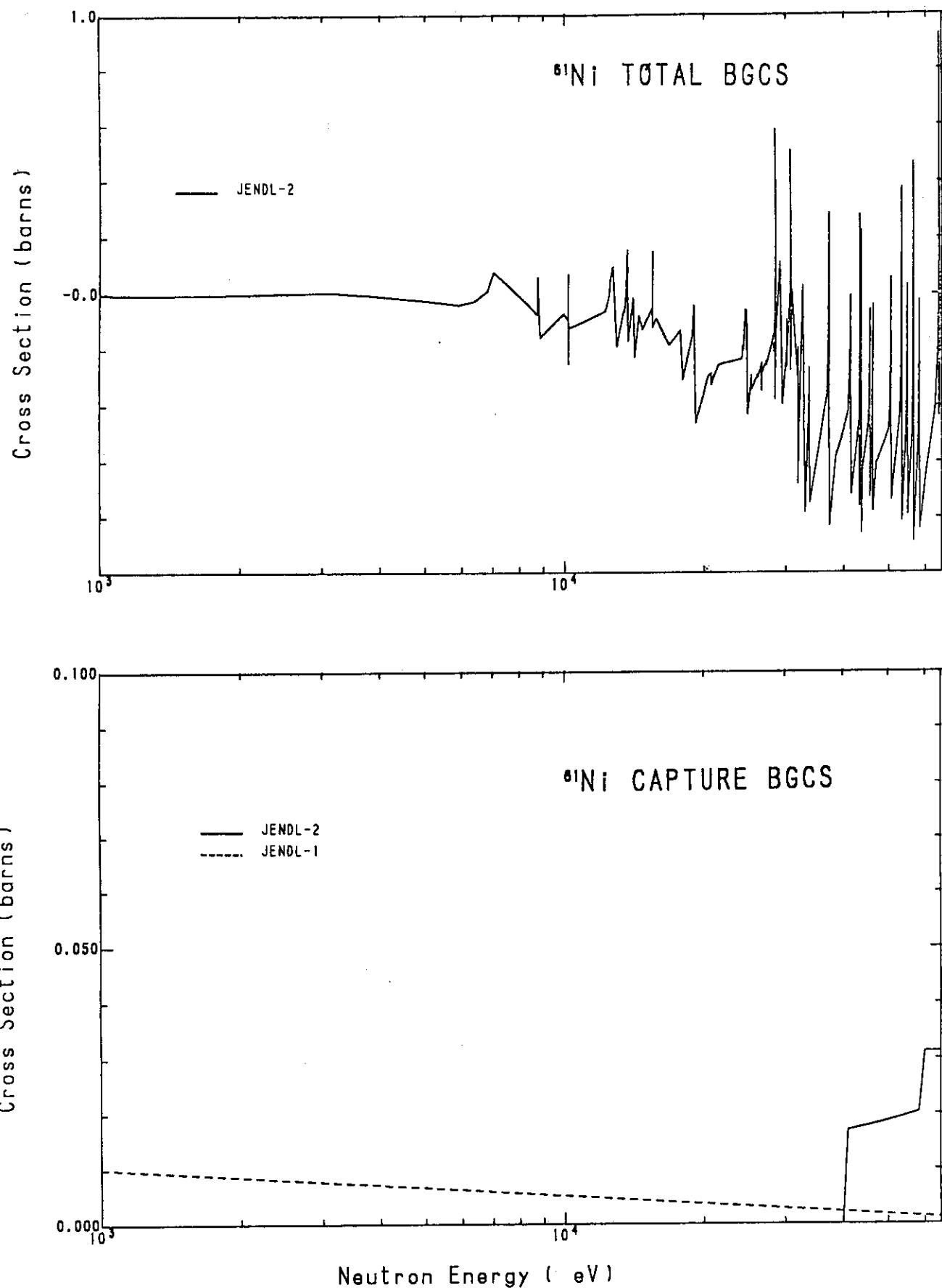
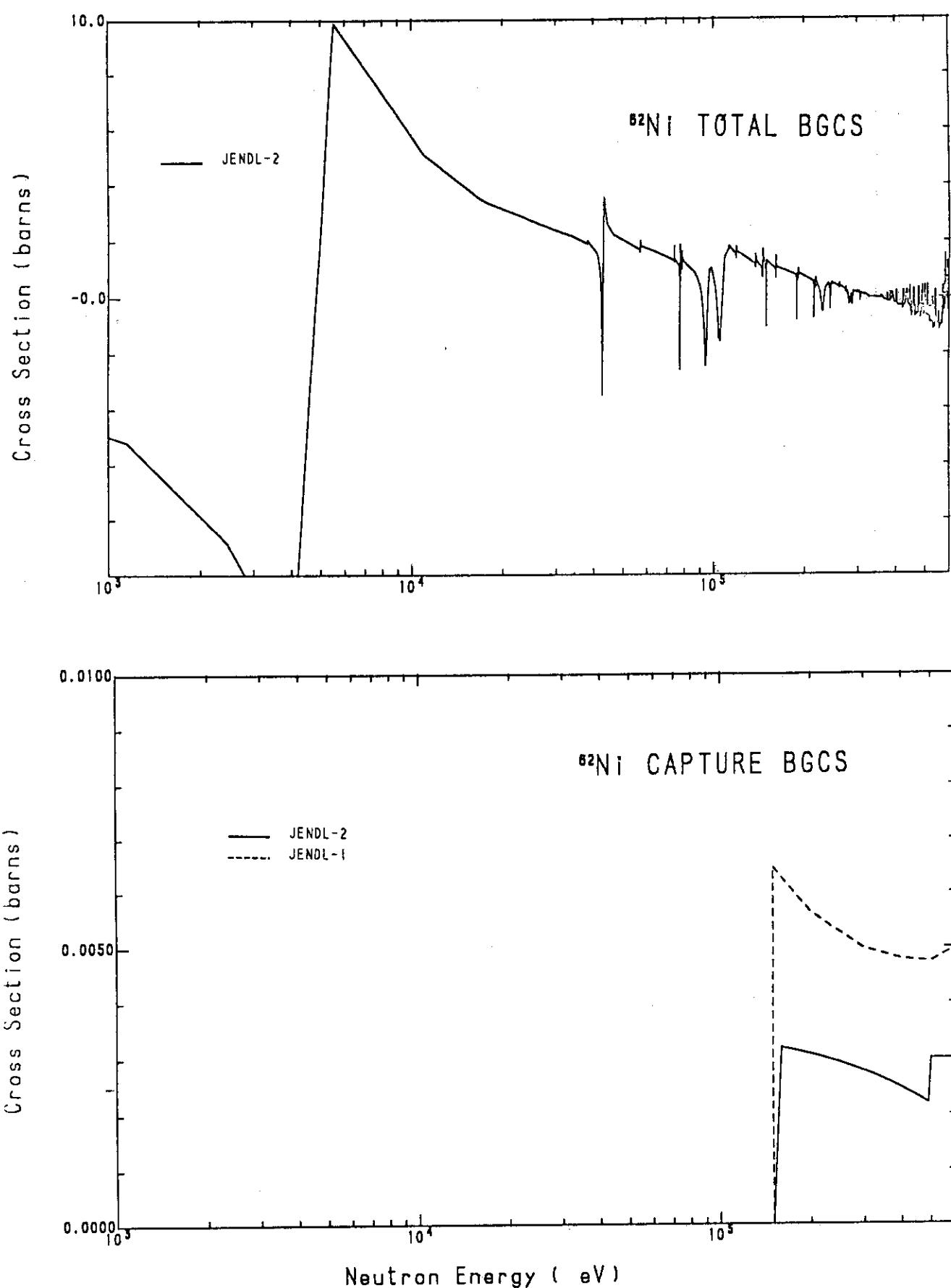


Fig.A.17 The total and capture background cross sections of ^{61}Ni .

Fig.A.18 The total and capture background cross sections of ^{62}Ni .

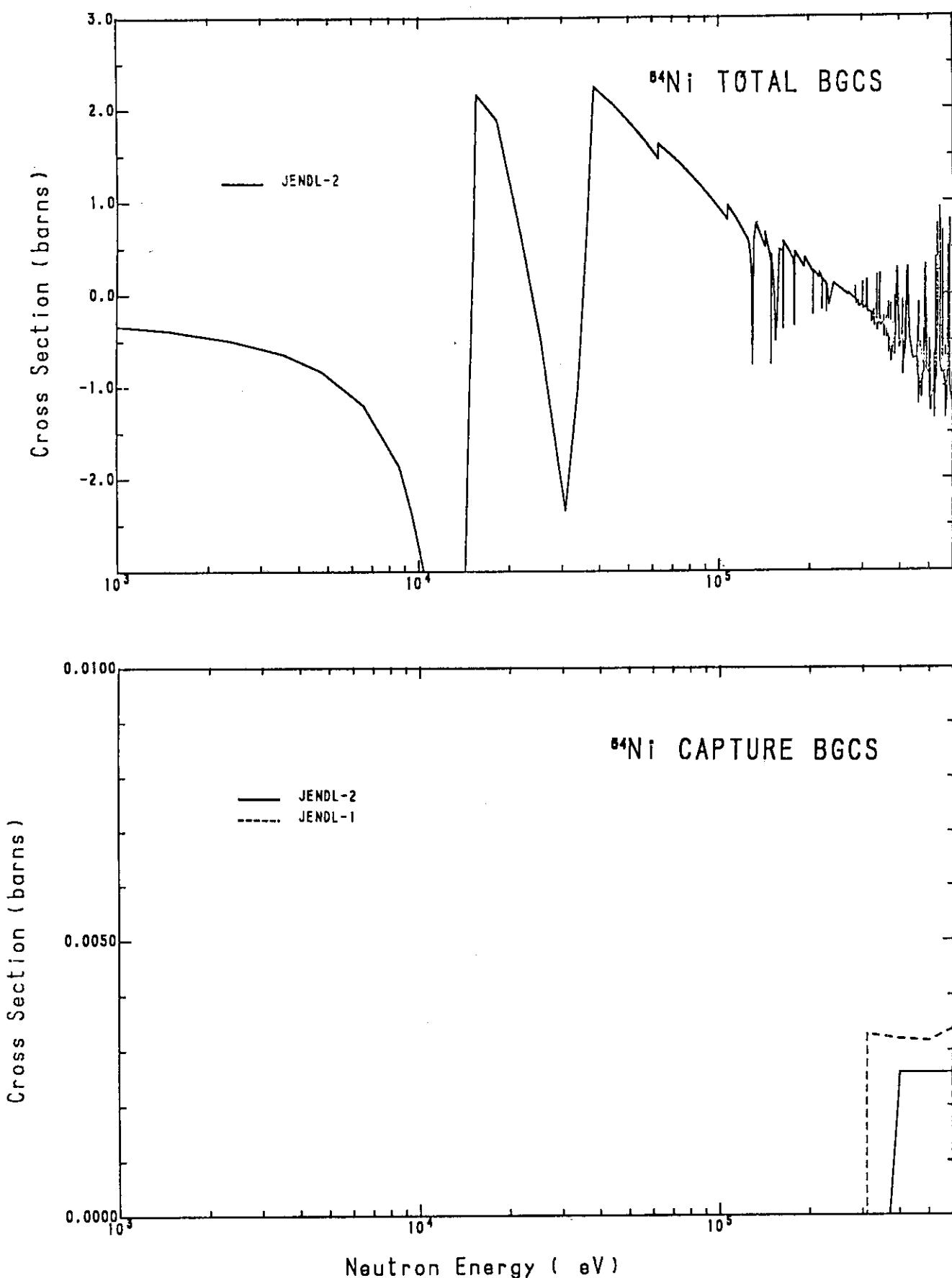


Fig.A.19 The total and capture background cross sections of ^{64}Ni .