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REVIEW : STUDY OF SINGLE-PELLET INJECTION EXPERIMENTS AND  
DEVELOPMENT OF PELLETT INJECTOR IN JFT-2M

October 1987

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and Seio SENGOKU

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Review : Study of Single-Pellet Injection Experiments and  
Development of Pellet Injector in JFT-2M

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The single pellet injector developed for JFT-2M and the improvement of plasma characteristics in the auxiliary-heated discharges by single-pellet injection are reviewed for the period 1982-1986. The pellet injector is a pneumatic type and the designed pellet size is  $1.65 \text{ mm}\phi \times 1.65 \text{ mmL}$  and  $1 \text{ mm}\phi \times 1 \text{ mmL}$ . The hydrogen, deuterium and mixed ( $\text{H}_2+\text{D}_2$ ) pellets can be produced with good reproducibility. Maximum pellet velocity is about 970 m/s (pellet is deuterium and propellant gas is hydrogen). In the pellet injection experiments into auxiliary-heated (NB, ICRF) divertor or limiter discharges, the plasma confinement time is improved by a factor of 1.4-1.7 compared with the confinement time in the Ohmic discharges. The achieved confinement time is longer than that on the high confinement mode (H-mode) in gas fueled discharges, although the phenomena are transient.

Keywords: Solid Pellet, Pellet Injector, Pellet Diagnostics, Plasma Confinement, L- and H-modes, Neutral Beam Heating, Ion Cyclotron Range of Frequency Wave Heating, Single-Null Divertor, Tokamak

JFT-2Mにおける固体ペレット (Single-Pellet)  
入射実験及びペレット入射装置開発に関するまとめ

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(1987年9月12日受理)

本報告書は、昭和57年から昭和61年度に渡って実施した高性能トカマク開発試験装置 (JFT-2M) 用の1ペレット入射装置の開発と第2段加熱プラズマに高速の固体ペレット ( $H_2$  または  $D_2$ ) を入射して実現したプラズマ特性の改善に関する実験についてまとめたものである。JFT-2M用に開発したペレット入射装置は、圧縮気体の膨張を利用してペレットを銃身内で加速するニューマチック方式を採用している。生成されるペレットの大きさは、直径1.65 mm、長さ1.65 mmと直径1 mm、長さ1 mmである。この装置では水素、重水素および混合 ( $H_2 + D_2$ ) ペレットのいずれをも生成することができる。ペレットの最大射出スピードは、約970 m/sである。中性粒子 (NB) 入射あるいはイオンサイクロトロン周波数帯 (ICRF) 波加熱したダイバータ配位あるいはリミター配位のプラズマへペレットを入射した実験を行った。その結果、粒子をガスパフ補給法により行ったときに得られる通常の閉込めモード (Lモード) にペレットを入射すると、NB及びICRF加熱のいずれにおいてもエネルギー閉込め時間が大幅に改善 (ガスパフ補給法において得られる高閉込めモード (Hモード) における閉込め時間の1.4-1.7倍) される放電モードを見出した。またICRF加熱におけるHモードにペレットを入射した場合にも同程度の閉込めの改善が見られた。以上の実験結果から、固体ペレットによる粒子補給法がプラズマの性能向上に極めて有効であることが実証された。

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## 1. Introduction

Fueling is one of the critical requirements of operating a tokamak fusion reactor in a long pulse or quasi-steady state mode (burn time is long compared to particle confinement time). The solution of this problem, as well as those of providing adequate pumping or exhaust and adequate impurity control, is essential to long pulse operation. Tokamak fusion systems are characterized by small fractional burn up of the fuel and consequently most of the fuel requirement is to meet the particle transport losses from the confinement region.

Injection of a high-speed hydrogen isotope pellet is being considered as a means for fueling future fusion power reactors of the tokamak type. This technique differs from cold-gas inlet at the plasma boundary (i.e. gas puffing). It allows most flexible plasma density profile maintenance as a result of fresh fuel being deposited directly on the innermost flux surfaces. In a future reactor with D-T burn, tritium inventory on the wall is very important problem, which must be solved. The high speed pellet injection should be expected to decrease the tritium inventory on the vacuum chamber wall and the limiter, because the pellet will be well burned.

Fueling experiments with a solid hydrogen or deuterium pellet have been performed in many tokamaks, e.g. ISX[1], Alcator-C[2], TFR[3], PDX[4], PLT[5], ASDEX[6], DIII[7], TFTR[8] and JET[9], to study possibility of improvement of plasma characteristics. Especially, in Alcator-C, a plasma with high electron density ( $\bar{n}_e$ ) and high  $n_e(0)\tau_E$  has been produced by pellet fueling ( $n_e(0)$ : electron density at a plasma center,  $\tau_E$ : global energy confinement time). Achieved  $n_e(0)\tau_E = (0.6 - 0.8 \times 10^{20} \text{ sec} \cdot \text{m}^{-3})$  reached the Lawson criterion[2]. The results of pellet injection on TFTR[8] with a large size of a plasma and high temperature have indicated that confinement within the plasma core was improved and the Murakami factor ( $\bar{n}_e R/B_t$ ) obtained with pellets exceeded that which had been achieved by deuterium gas puffing. In JET[9] Ohmic heated discharges, the operational regime is extended by large pellet injection (density limit exceeds by a factor of 2), and increment of electron and ion temperatures is observed in pellet fueled ICRF heating.

In JFT-2M[10], the pellet injection study has been proposed in 1981. Main objects of pellet injection experiments are as follows.

- (1) Improvement of plasma parameters in limiter and divertor

configurations (extension of plasma operational regime).

- (i) production of high density plasmas (increasing of density limit or improvement of Murakami factor)
- (ii) improvement of energy and particle confinements
- (iii) production of high-beta plasmas
- (iv) control of a plasma density (plasma pressure) profile
- (2) Improvement of heating efficiency on the neutral beam (NB) injection and ion cyclotron range of frequency (ICRF) wave heatings.
- (3) Identification of particles (thermal electron, runaway electron, fast ion et al.) mainly contributed to the pellet ablation ---- ablation mechanism.

Also, the development of a pellet injector is as follows.

- (1) Development of a pellet injector with high-velocity to hyper-velocity.
- (2) Improvement of reproducibility of pellet velocity and size of pellet.
- (3) Production of H<sub>2</sub>, D<sub>2</sub> and mixed (H<sub>2</sub>+D<sub>2</sub>) pellets.
- (4) Development of a multi-pellet injector.
- (5) Development of a transfer technique of pellets from the injector to tokamak machine.

Lastly, the possibility of pellet fueling to the large tokamak machine will be found, that is, scaling of pellet fueling to a large machine will be studied.

A fundamental form of an injector has been developed in collaboration with Mitsubishi Atomic Power Industries Inc. in 1982[11]. In the next year, the single pneumatic-gun type injector (pellet size in the carrier: 1 mm $\phi$   $\times$  1 mmL) for JFT-2M was constructed[10]. In Ohmic heated plasmas, electron density rise of about  $0.7 \times 10^{19} \text{ m}^{-3}$  by this pellet was obtained. And after that, the large pellet (hole size in the carrier is 1.65 mm $\phi$   $\times$  1.65 mmL) has been used on experiments[10]. Up to now, maximum line averaged electron density rise of about  $4.5 \times 10^{19} \text{ m}^{-3}$  was achieved and significant improvements of energy and particle confinements were performed in Ohmic and auxiliary-heated (NB, ICRF) plasmas with a pellet fueling[12]. In these discharges, electron and ion temperatures near a plasma center gradually recovered for about 30-40 ms after pellet injection, and the rise of radiation loss power (including the charge exchanged particle loss) and level of loss power are relatively low. It is necessary to optimize the timing of pellet injection during



auxiliary heating in order to get an improvement of confinement. Pellets can penetrate deeper in ICRF heating than in NB heating[10].

This paper is the review of the experimental results with pellet injection and the single-pellet injector developed for JFT-2M from 1982 to 1986. In the next section, a pellet injector system and diagnostic apparatuses are described, and some tested fundamental parameters (pellet velocity, mass, pellet shape, pellet ablation, et al.) are presented in section 3. In section 4, the improved confinement results by pellet injection are reviewed and in the last section, obtained results are summarized and the future plan is described.

## 2. Pellet Injector System[10]

### 2.1 Single-pellet injector

The pellet injector for JFT-2M is basic type developed at Oak Ridge National Laboratory (ORNL). Figure 1 is the schematic diagram of a single pneumatic pellet injector, which indicates the principle of the injector. A stainless-steel gun barrel (two barrels were prepared: 1 mm bore  $\times$  18 mm long and 1.69 mm bore  $\times$  18 mm long) is fixed within the barrel housing made of oxygen free copper (OFC) as shown in Fig. 2. A disk shaped stainless steel pellet carrier is situated between the barrel housing and the OFC main housing. The carrier has two holes drilled opposite each other extending through the face of the disk within which pellets are located. A diameter of the hole in one carrier is 1 mm and that 180° by a pulse motor to transport the pellet at the position in an injection line with the gun barrel. The gun assembly is convectively force-cooled by liquid helium. The pellet is propelled from the mechanism by pressurized helium or hydrogen gas. Working pressure of propellant gas is selected to any value from 0 to 30 kg/cm<sup>2</sup>.

The primary function of the pellet injection line is to isolate the JFT-2M vacuum vessel from the pellet injector. This is accomplished by using a gas delay line as shown in Fig. 3, which consists of the three vacuum chambers and two stainless steel pipes (an inner diameter is 4 mm and 10 mm). The first chamber (low vacuum chamber) has a mechanical booster pump (pumping speed: 500 m<sup>3</sup>/h) and the second chamber (high

auxiliary heating in order to get an improvement of confinement. Pellets can be penetrated deeper in ICRF heating than in NB heating [10].

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vacuum chamber) has a turbo-molecular pump (pumping speed: 100 l/s). The last chamber (drift chamber) is evacuated by a 100 l/s turbo-molecular pump. In addition, fast magnetic gate valves (VACOA model: SOV-050-05-SS), situated between the first and second chambers and between the second and last chambers, close behind the pellet for trapping most of the gas in the first and second chambers. The injection line of a pellet makes an angle of about  $5.2^\circ$  with the major radius of the torus in JFT-2M (Fig. 4).

## 2.2 Diagnostic apparatuses of a pellet

### 2.2.1 Pellet velocity

A pellet velocity is measured by a pair of photo-interruptors, which consist of a photo-diode (Bell & Howell: Silicon-pin diode 509-01) with IR filter and a light emission diode (Toshiba: GaAs emission diode TLN105). A pair of photo-interruptors, which are installed in the vacuum chamber, are put on the injection line separated by a distance of 1 m (Fig. 5).

### 2.2.2 Pellet mass

A pellet mass is periodically monitored from the pressure rise by trapping and evaporating the pellet in the chamber (drift chamber). A pressure sencer is a diaphragm type (MKS Instruments Inc.: Baratron 222BHS-BB10). The number of atomic particles can be calculated by the following equation.

$$N_{H,D} = 1.9307 \times 10^{22} P_d V_d / T ,$$

where  $P_d$ (Torr) is pressure rise in the chamber,  $V_d$ (l) = 4.98 l (volume of the drift chamber) and  $T$ (°K) = 300°K. This method can not be used during the injection experiments into a plasma.

Another method monitored a pellet mass on shot to shot of discharges was developed, which employs a microwave cavity operating at around 8.8 GHz [13]. The microwave cavity is cylindrical, 26 mm in inner diameter and 35.5 mm in inner length. This shows a strong resonance at

8.8 GHz. The mode is the  $TM_{010}$ . Q factor of the cavity is about 3175. The change in resonant frequency by a pellet, dielectric sample, is given by

$$|f_0| = 11.1 f_0 P m_p / (M V_c)$$

where  $f_0$  is the unperturbed resonant frequency, P is the molar polarisability of a pellet (1.99 cm/mol for deuterium), M is the molecular weight (4 g/mol for deuterium),  $V_c$  is the cavity volume ( $1.9 \times 10^{-5} \text{ m}^3$ ) and  $m_p$  is the pellet mass.

The output signal of the reflected wave generated by the frequency shift as the pellet is travelling in the cavity is detected with the system as shown in Fig. 6.

### 2.2.3 Shadowgraph of a pellet

A shadowgraph gives an indication of a pellet size and shape. Photographs of a pellet are taken through a Polaroid camera by backlighting the pellet with a spark lamp (Xenon Corporation: model 437A Nanopulser) as shown in Fig. 7.

### 2.2.4 Ablation

A time evolution of the pellet ablation, e.g. the Balmer  $H_\alpha$  or  $D_\alpha$  line emission is measured in the horizontal direction by using a single photo-diode (Hamamatsu Photonics Corporation: S1226-5BQ) with an interference filter. The ablation profile as a function of the plasma radius can be obtained from this signal and the measured pellet velocity. Another diagnostic apparatus, which is 8 channel photo-diode array with a slit and an interference filter and views the pellet ablation in the vertical direction as shown in Fig. 8, can be obtained a local ablation and ablation profile from the envelope of peak intensity at each channel. Also, the velocity of the pellet travelling in a plasma can be evaluated from this measurement. The rise time of these detectors is about 2.8  $\mu\text{sec}$ . Spatial resolution of this array is about 12 mm on the equatorial plane.

### 2.2.5 Scattering of pellet trajectory

A Al-foil target was installed 1.71 m downstream of the muzzle (Fig. 9). Scattering of trajectory for pellets from the gun barrel is monitored from scattering of holes on the Al-foil.

## 3. Production and Injection of Pellet[10]

### 3.1 Condition of pellet production

Properties of hydrogen and deuterium gas below their critical temperature are shown in Table 1. When the temperature in the pellet carrier and main and barrel housings is less than 5-10°K, the hydrogen or deuterium gas easily freezes, and the solid pellet is produced. In the present pellet injection, the hydrogen gas freezes at about 5.6°K on the surface of the heat exchanger. The flow rate of liquid helium, which indicates the gas flow rate after evaporation of liquid helium, is about 100 l/min. For deuterium pellet production, the flow rate of liq.He is 48-56 l/min. and temperature is about 9.2°K. The flow rate of the fuel gas is 12-13 l/min., and when the pressure near the production region of the pellet reaches at 400-430 Torr, a fuel supply is stopped. Cycle of pellet production and injection is about 3.5 minute.

The injector can be produced the mixed pellet of hydrogen and deuterium[14]. Production of the mixed pellet examined for three mixed gases with different ratio of hydrogen to the mixed gas ( $H_2+D_2$ ). The ratio is about 39.8%, 23.5% and 6.3%, respectively. The temperature to freeze the mixed gas is about 7.2°K, and other condition is nearly the same with that of the hydrogen or deuterium pellet. The component of the mixed pellet is estimated from the component of mixed gas, which is measured by the mass spectrometer after the pellet is evaporated in the drift chamber. The component is different from that of the mixed gas before freezing as shown in Fig. 10. This may attribute to freezing temperature and gas pressure of supplied fuel. The detail control of temperature and pressure may be necessary to produce the mixed pellet with the desired component.

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### 3.2 Result of pellet injection test

#### 3.2.1 Pellet velocity

The injection velocity of the pellet can be changed by setting the pressure of a propellant gas to different value. The gas is helium or hydrogen (the sound velocity is about 1000 m/s for helium and 1300 m/s for hydrogen at room temperature). Figure 11 shows the velocity as a function of propellant gas pressure. The pellet size is  $1.65 \text{ mm}\phi \times 1.65 \text{ mmL}$  in the pellet carrier disk. A solid line is a calculated velocity of a  $\text{H}_2$ -pellet on the basis of the ideal gun model given by Milors and Foster [15]. The propellant gas is helium at temperature of  $288.16^\circ\text{K}$ . The velocity ( $V_p$ ) is given by the following equation.

$$P_0 = \frac{2m_p C_0}{(\gamma+1)A_p} \frac{U_{\max}}{L} \left[ \frac{\gamma-1}{2} + \left(1 - \frac{V_p}{U_{\max}}\right)^{-\frac{(\gamma+1)}{(\gamma-1)}} - \frac{(\gamma+1)}{2} \left(1 - \frac{V_p}{U_{\max}}\right)^{-\frac{2}{(\gamma-1)}} \right] \text{ dyne/cm}^2$$

where  $P_0$  is pressure of the propellant gas,  $C_0$  is the sound velocity of propellant gas ( $= \sqrt{\gamma RT/M}$ ),  $\gamma$  is specific heat of propellant gas,  $R$  is gas constant ( $= 8.314 \times 10^7 \text{ erg}^\circ\text{K}^{-1}\text{mol}^{-1}$ ),  $T$  is temperature ( $^\circ\text{K}$ ) of propellant gas,  $M$  is molecular weight of propellant gas,  $m_p$  pellet mass,  $A_p$  is cross section of a gun barrel,  $L$  is length of a gun barrel,  $U_{\max} = 2C_0/(\gamma-1)$ . Closed circles are experimental results of the  $\text{H}_2$ -pellet accelerated by He gas and crosses are the obtained velocity of the  $\text{D}_2$ -pellet propelled by  $\text{H}_2$  gas. Data for the  $\text{H}_2$ -pellet indicate that velocities are 80 to 95% of the calculated velocity except some data at  $10 \text{ kg/cm}^2$ . In usual injection experiments, the velocity is 714-833 m/s. Reproducibility of the velocity is 80 to 90%. The maximum velocity for the  $\text{D}_2$ -pellet propelled by  $\text{H}_2$  gas is about 970 m/s at  $24 \text{ kg/cm}^2$ .

#### 3.2.2 Pellet mass

The  $\text{D}_2$ -pellet with the size of  $1.65 \text{ mm}\phi \times 1.65 \text{ mmL}$  includes deuterium atoms of about  $2.15 \times 10^{20}$  atoms. Under the good condition of freezing, the mass of the injected pellet is 71 to 90% of the maximum mass, corresponding to  $1.5-1.9 \times 10^{20}$  atoms.

The microwave cavity is set in front of the gun barrel. It is

indicated from output of the detector (Fig. 6) that the mass-reproducibility of the pellet from the muzzle is very good. The data of pellet mass measured by two methods distribute from large to small values along the straight line going through origin in the diagram as shown in Fig. 12. These data indicate that small pellets produced in the carrier disk and injected from the muzzle. Sometimes, small pellets are injected into the drift chamber although the signal of the microwave cavity indicates large pellets. This may be due to the evaporation of the pellet during passing through the guide-tube.

### 3.2.3 Scattering of a pellet trajectory

An angular dispersion in a pellet trajectory in front of the entrance of JFT-2M is about  $0.46^\circ$  (the spread in the pellet trajectory is about 26 mm in diameter at a plasma center), as shown in Fig. 9.

### 3.2.4 Shadowgraph of an in-flight pellet

Figure 13 is a shadowgraph of an in-flight  $D_2$ -pellet with about 769 m/s. This indicates a pellet integrity. Some photographs show that pellets are broken or the edge of the pellet is lost.

### 3.2.5 Ablation of pellet

Figure 14(a) is the radial ablation profile of  $D_2$ -pellet obtained by using the 1 channel photo-diode with an interference filter.  $R_{out}$  is the distance from the major axis of the torus to the cross-point where the outside magnetic separatrix surface crosses the major radius in the equatorial plane. The pellet ablates just inside the magnetic separatrix surface in the auxiliary-heated discharge with single-null X-point. Ablation profiles are different by low confinement mode(L-mode) or high confinement mode(H-mode), divertor- or limiter-discharges, or electron temperature at a plasma boundary. Penetration depth depends on the size of pellet, pellet velocity and plasma parameters. Figure 14(b)-(e) indicates time histories of  $H_\alpha/D_\alpha$  line intensity (local ablation) at  $X=0.0242$ ,  $0.0729$ ,  $0.1212$  and  $0.1725$  m.  $X$  is a distance from a minor torus axis to the point where the midplane crosses a line of sight of the 8 channel photo-diode array. The envelope of the peak



intensity of local ablation at each channel against X indicates an ablation profile, which qualitatively agrees with the profile measured by using the horizontal photo-diode in Fig. 14(a). The pellet velocity in the plasma can be estimated from the plots of X versus times at peak intensity for each channel. This velocity agrees well with that obtained by the time of flight method on the injection line within the experimental uncertainty. The small hydrogen pellets (pellet size in the carrier =  $1 \text{ mm}\phi \times 1 \text{ mmL}$ ) are injected in Ohmic heated plasmas with different electron density from  $1 \times 10^{19}$  to  $3 \times 10^{19} \text{ m}^{-3}$ . Figure 15 shows the position of peak intensity of  $H_{\alpha}$  line as a function of electron density. In the region of low density less than about  $2 \times 10^{19} \text{ m}^{-3}$ , position of peak intensity decreases. This is due to the enhancement of ablation by run-away electrons, these electrons easily produced in the low density region. On the other hand, the position again decreases as the density increases. This tendency is reasonable according to the neutral shielding model of ablation by Milora and Fester[1]. The penetration depth of pellets ( $H_2$ ,  $D_2$ ) is measured in the auxiliary-heated plasmas with the limiter configuration. Figure 16 is the dependence of the penetration depth of pellets on the auxiliary-heated power ( $P_{NB}$  and  $P_{ICRF}$ ). The electron density before pellet injection is  $2.8\text{--}5.5 \times 10^{19} \text{ m}^{-3}$  and pellet velocity is 714-769 m/s. The pellet size injected into discharges is estimated from the total intensity of  $H_{\alpha}$  line, but the size in the carrier disk is  $1.65 \text{ mm}\phi \times 1.65 \text{ mmL}$ . The experimental results indicate that the penetration depth decreases as increasing of the heating power in both NB and ICRF heated discharges, but the depth in ICRF cases is larger than that in NB cases. In these experiments the ICRF heating is electron heating mode, so the number of the fast ions is small compared with that in NB cases. From the above-mentioned results, it is expected that the pellets can penetrate deeper even in higher ICRF power.

Figure 17 is the color photograph and tree-dimensional expression of  $D_2$ -pellet ablation in the NB( $\sim 0.88 \text{ MW}$ ) heated deuterium plasma with single-null divertor configuration, which was taken by the flaming camera. The pellet life time is about 400  $\mu\text{s}$ .

#### 4. Experimental Results of Pellet Injection into Plasmas

Deuterium or hydrogen pellets are injected into an Ohmic heated plasmas and auxiliary-heated plasmas with limiter or divertor configuration. Ohmic heated discharges was sometimes disrupted by a single pellet injection, if the density rise was larger than  $2 \times 10^{19} \text{ m}^{-3}$ .

##### 4.1 Pellet injection in NB heated single-null divertor discharges

The single-null divertor discharges transfer from the lower confinement mode (L-mode) to the high confinement mode (H-mode) during auxiliary heating. The global energy confinement time ( $\tau_E$ ) recovers up to that in Ohmic heating[16]. Pellets were injected into the L-mode or H-mode in order to improve the plasma properties [12]. The global energy confinement time is calculated according to the energy balance equation, that is the definition of confinement time ( $\tau_E$ ) is as follows,

$$\tau_E = W_s / (P_{\text{total}} - dW_s/dt) ,$$

where  $W_s$  is plasma stored energy,  $P_{\text{total}}$  is total input power.

In the first injection experiments, the deuterium pellet was fueled before H-transition (in L-mode). The injection velocity is about 700 m/s. The hydrogen NB of 34 keV was injected into the deuterium plasmas with upper single-null divertor configuration at 650 ms, heating power injected from the port is about 0.77 MW (net heating power by the calculation of NB power deposition code is about 0.67 MW). The magnetic configuration in the series experiments is shown in Fig. 18. The outer magnetic surface, which is closed surface of about 5 mm inner from the separatrix surface, null point and some configuration parameters determined by the magnetic fitting method. Three different types of discharges with pellet fueling are classified under these conditions.

The increment of  $\bar{n}_e$  at the pellet injection time ( $\Delta\bar{n}_e$  measured by the HCN laser interferometer) is about  $1.5 \times 10^{19} \text{ m}^{-3}$  as shown in Fig. 19. Subsequently, the density is increasing linearly with time. The  $D_\alpha$  line intensity near the limiter or the location of the gas puffing decreases after the pellet injection. These time behaviors of  $\bar{n}_e$  and  $D_\alpha$  line indicates that the particle recycling reduced and the particle confinement

was improved. The radiation loss power including the charge exchanged particle loss is increasing linearly up to the time of sudden increase of  $D_\alpha$  line intensity. The global energy confinement time is determined from the magnetic fitting method assuming the constant internal inductance. After pellet injection,  $\tau_E$  is nearly the same value as that of ohmically heated discharges at the same density region, but decreases gradually as increasing the density and the radiation loss power. This degradation may be due to the increase of the radiation loss power from the central region of the plasma. These properties are similar to that in the H-mode achieved in the gas fueled discharges.

The second discharges are the very high confinement mode (pellets are fueled in the L-mode).  $\Delta \bar{n}_e$  is about  $4.5 \times 10^{19} \text{ m}^{-3}$ . The ablation profile of this large pellet was horizontally measured by the single  $H_\alpha/D_\alpha$  detector. A local ablation rate at the point of  $X=0.1725 \text{ m}$ ,  $X=0.1212 \text{ m}$ ,  $X=0.0729 \text{ m}$  and  $X=0.0242 \text{ m}$  can be also estimated from the signals in Fig. 20. These signals show that the pellet is ablating mainly around  $r=0.1 \text{ m}$  ( $R=1.4 \text{ m}$ ) and passing through the center of the plasma column.  $W_S$  is linearly increasing and when  $W_S$  reaches 34 kJ,  $\dot{B}_\theta$  signal always grows and the plasma energy is bursted out (Fig. 21). The frequency of the  $\dot{B}_\theta$  is about 5 kHz, and the  $\dot{B}_\theta$  signal grows at the safety factor ( $q_\psi$ ) of the outer magnetic surface close to 2.5. As this time,  $\langle \beta_t \rangle$  is about 1.6%, and the factor of  $g$  according to the equation of  $\langle \beta_t \rangle = g I_p / (a B_t)$  [17] is about 2.2. This results agree with the range of theoretical limits due to kink and ideal ballooning modes with free boundary. The burst-out of plasma energy is considered to be caused by the  $\beta$ -limit. During increasing of  $W_S$ , about of 70% of net NB heated power is stored in the plasma ( $dW_S/dt$  is shown in Fig. 21). The increment of  $W_S$  is mainly due to the increase of both electron and ion temperatures ( $T_e^{SX}$ ,  $T_i^{CX}$ ), which are obtained by the soft X-ray energy spectra analysis and mass separated neutral particle energy spectra analysis, respectively. Both electrons and ions are cooled down, following the pellet injection, but after about 30-40 ms both electron and ion temperatures are approximately the same and the plasmas heat to about 0.8 keV as shown in Fig. 22. The radiation loss and  $D_\alpha$  line intensity do not increase up to the time of the saturation of  $W_S$ . These facts indicate that impurity accumulation into a plasma core and particle recycling rate is very little in this case compared with that in the H-mode.  $\tau_E$  increases to 60-75 ms and this level is sustained

for about 20 ms. The value exceeds that in Ohmically heated discharges by a factor of 1.4-1.7. However, the duration sustained very high confinement is shorter than  $\tau_E$ , so the discharges is transient. This is due to the growth of MHD oscillation with large amplitude.

In the another discharges with properties of very high confinement ( $\tau_E = 60-75$  ms) by pellet injection with  $\Delta\bar{n}_e = 4 \times 10^{19} \text{ m}^{-3}$  (Fig. 23),  $W_S$  saturates,  $P_R$  increases and  $\tau_E$  degrades to about 35 ms (near the H-mode level) following  $\dot{B}_\theta$  signal growing ( $P_R$  keeps lower level during very high confinement phase). The ablation profile is similar to that in the above-mentioned case (Fig. 24). The characteristics of this discharges are concluded that the very high confinement mode terminates when  $\langle\beta_t\rangle$  taches close to the  $\beta$ -limit softly, and the discharges turns to the H-mode.

Figure 25 shows the global energy confinement time as a function of the line averaged electron density.  $\tau_E$  in the Ohmic heating increases linearly with  $\bar{n}_e$  up to  $\bar{n}_e = 4 \times 10^{19} \text{ m}^{-3}$  and saturates at about 50 ms.  $\tau_E$  in the small pellet fueled discharges is 50-55 ms and degrades to the level of H-mode in the gas-fueled discharges. The very high confinement (60-75 ms) achieved around  $\bar{n}_e$  of  $6 \times 10^{19} \text{ m}^{-3}$  by the large pellet.

In the second injection experiments, pellets were fueled in the H-mode discharges during NB heating. The pellets much ablate near the peripheral region (more outside than in the L-mode injection cases) as shown in Fig. 26. This is due to enhancement of ablation by the fast ions, which are produced through the charge exchange process between NB and plasma ions. The plasma in the peripheral region is strongly cooled. So, the stored energy dropps rapidly following radiation loss increase (Fig. 27).  $W_S$  does not recover to the level without pellet injection. The global energy confinement time deteriorates. The degree of the degradation of  $W_S$  becomes smaller as the injection timing is earlier.

#### 4.2 Pellet injection in the ICRF heated single-null divertor discharges[12]

In ICRF heated discharges with the lower single-null configuration (Fig. 28), the plasmas are transferred to the H-mode as well as in the NB heated discharges. The large deuterium pellet ( $\Delta\bar{n}_e = 3-4 \times 10^{19} \text{ m}^{-3}$ ) was injected into the L-mode divertor discharges with the number-density

ratio of hydrogen to deuterium ( $n_H/n_D$ ) of about 30%. The ICRF launched power  $P_{rf}$  is 0.6-0.8 MW (from 600 to 900 ms).

Figure 29 shows the behaviors of  $\bar{n}_e$ ,  $W_S$ ,  $\dot{B}_\theta$ ,  $\dot{W}_S$ ,  $I^{D\alpha}$  (solid line: in divertor region, dotted line: in main plasma region), and  $P_R$ , which are very similar to those in the discharges heated by NB (see §4.1) except the behavior after about 760 ms. In this type discharge, the plasma stored energy continuously increases, and the global energy confinement time ( $\tau_E$ ) rises up to 55-60 ms, keeping this level for about 20 ms (transient phenomena), and  $P_R$  sharply increases following a subsequent decrease until large MHD oscillation grows. According to the MHD oscillation growing, the radiation loss power again increases and confinement time decreases although the electron density is continuously increasing. This is due to accumulation of an impurity into a plasma center as spectroscopic measurements of iron line emissions (FeX, FeXVIII) indicate. After MHD oscillation growing, impurity looks like to repeat accumulation and exhaust.

Sometimes, discharges characterized by transiently very-high confinement are disrupted at about 40 ms after pellet injection, which comes from rapid and large growth of MHD oscillation and rapid increase of radiation loss power as shown in Fig. 30. Then, maximum  $W_S$  is about 32 kJ, and volume averaged toroidal beta value  $\langle\beta_t\rangle$  is about 1.3%, corresponding to Toroyan factor  $g=1.6$ . Thus, the plasma disruption attributes to a large amount of radiation cooling and is not due to beta limit. Figure 31 shows the ablation profile obtained from the horizontal measurement of  $D_\alpha$  line emission.  $R_{out}$  is a major radius crossed an outside separatrix surface in the midplane and  $R$  is the local position of the plasma from the major axis of the torus. The pellet penetrated deeply in a core plasma, exceeding a minor axis of a torus ( $R=1.31$  m) as drawn by a solid line. Maximum ablation is around  $R_{out}-R=1.2$  m e.g., about one half of the plasma minor radius. The dotted line indicates a profile of the pellet ablation in L-mode during NB heating.

The second discharges with the deuterium pellet injection in the H-mode have the electron density rise of about  $3 \times 10^{19} \text{ m}^{-3}$  (Fig. 32). Radiation loss after pellet fueling is slightly large compared with that in the discharges fueled the pellet in the L-mode.  $W_S$  continuously increases up to about 35 kJ until the radiation begins to increase. These characteristics are significantly different from that for injection

in H-mode during NB heating. Energy confinement time is improved up to about 50 ms around  $\bar{n}_e = 6 \times 10^{19} \text{ m}^{-3}$ . This value exceeds  $\tau_E$  in Ohmic heating as shown in Fig. 33. Electron temperature  $T_e^{\text{SX}}$ , which is obtained from analysis of soft X-ray spectrum measured in the tangential direction, decreases immediately after the injection from approximately 1.35 keV to less than 1.1 keV, immediately following the injection and recovered up to the value before pellet injection (Fig. 34). The pellet ablation profile is shown in Fig. 31 by a broken line. The pellet is ablated slightly more outside than in L-mode cases in ICRF or NB heatings. Most of the pellet is ablated around  $R_{\text{out}} - R = 0.15 \text{ m}$ . Ablation in a scrape-off layer is very little as in other cases. In NB heated discharges with the pellet injection in the H-mode, the pellet is strongly ablated just inside separatrix layer compared with above-mentioned cases.

The global energy confinement time ( $\tau_E$ ) lineally improved with  $\bar{n}_e$  in the range of  $4-6.2 \times 10^{19} \text{ m}^{-3}$  on  $\bar{n}_e - \tau_E$  diagram as shown in Fig. 33. The maximum confinement time in both H-mode and L-mode is about 1.4-1.7 times as large as that in Ohmic heating around  $\bar{n}_e = 6 \times 10^{19} \text{ m}^{-3}$ . On the other hand, the confinement time for the H-mode without pellet in ICRF heated single-null divertor discharges is 30-40 ms for  $4-6 \times 10^{19} \text{ m}^{-3}$  and  $P_{\text{total}} = 0.8-1.1 \text{ MW}$ , which is in comparison with  $\tau_E$  in Ohmic heating. Above-mentioned experimental results reveal that the energy confinement time can be improved by pellet fueling in L-mode and H-mode during ICRF heating on the single-null X-point operation, although the phenomena are transient.

#### 4.3 Pellet injection in the NB+ICRF heated limiter discharges

The deuterium pellet is injected in the D-shaped limiter discharges (deuterium plasmas) with auxiliary heating combined NB and ICRF. The power of NB is about 0.8 MW and the launched power of ICRF is about 1.8 MW. Figure 35 is the magnetic configuration. The plasma current at the flat top is about 381 kA,  $B_t$  is 1.19 T, the ellipticity ( $\kappa$ ) is about 1.55 and  $q_\psi$  is 3.54 at 750 ms. The plasma is contact with the inner fixed limiter and is separated from outer fixed limiter. The deuterium pellet is injected in the saturation phase of  $W_s$  and  $P_R$  during ICRF heating and just before NB start (before about 10 ms). After pellet injection,  $W_s$  linearly increased up to about 80 kJ, this value is maximum

in the JFT-2M discharges as shown in Fig. 36. The key point to obtain this discharge is adjustment of timing of pellet injection. The electron temperature measured by soft X-ray spectrum analyser is recovered up to about 1.5 keV (Fig. 37). Figure 38 shows the ablation profile of the pellet with the injected velocity of about 769 m/s. The pellet reached inner side, exceeding the minor axis of the torus. The maximum deposition of the particle is near the major axis  $R=1.31$  m, corresponding to about one half of the minor radius of the plasma. The electron density is missed in the discharges.

## 5. Summary and Future Plan

The single-pellet injection experiments and the pellet injector developed for JFT-2M were reviewed. These studies and development of the injector are summarized as follows.

### Injection Experiments

- (1) The global energy confinement time can be improved by pellet injection in the L-mode of NB and ICRF heatings and in the H-mode of ICRF heating.
  - $\tau_E = 60-75$  ms for NB heated L-mode plasmas
  - $\tau_E = 55-60$  ms for ICRF heated L-mode plasmas
  - $\tau_E = 50$  ms for ICRF heated H-mode plasmas
- (2) The particle confinement time can be improved by pellet injection, ---- particle recycling does not increase ( $\bar{n}_e$  increases and maintains higher level, and increase of  $H_\alpha/D_\alpha$  line emission is little).
- (3) Electron and ion temperatures go up to initial level or higher level before pellet injection for 30-40 ms, although temperatures suddenly decrease at pellet injection.
- (4) Radiation loss power, including charge exchanged particle loss is not large during maintaining improved confinement, ---- supply of pure fuel is possible and impurity accumulation into a central region of the plasma is suppressed by pellet injection.
- (5) Plasma stored energy can be increased by optimization of injection timing of pellet and auxiliary heating (NB, ICRF).

It was found from these results that the pellet fueling in the

in the JFT-2M discharges as shown in Fig. 36. The key point to obtain this discharge is adjustment of timing of pellet injection. The electron temperature measured by soft X-ray spectrum analyser is recovered up to about 1.5 keV (Fig. 37). Figure 38 shows the ablation profile of the pellet with the injected velocity of about 769 m/s. The pellet reached inner side, exceeding the minor axis of the torus. The maximum deposition of the particle is near the major axis  $R = 1.31$  m, corresponding to about one half of the minor radius of the plasma. The electron density is missed in the discharges.

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It was found from these results that the pellet fueling in the



plasmas was effective to improve plasma properties.

#### Development of Injector

The single pneumatic-gun type injector was developed.

- (1) Pellet velocity is 970 m/s at maximum and 714-833 m/s in usual injection experiments. These velocities correspond to 80-95% of the velocity calculated from the ideal gun model.
- (2) Pellet mass is 71-90% of the designed pellet mass.
- (3) Hydrogen, deuterium and mixed ( $H_2+D_2$ ) gas can be frozen for solid pellet.
- (4) Microwave cavity method to measure the pellet mass was developed and 8 channel  $H_\alpha/D_\alpha$  photo-diode array with fast response was constructed to measure the ablation profile and the pellet velocity in the plasmas.

Lastly, the future plan of the pellet injection experiments and development of the pellet injector are shown in Fig. 39. In JFT-2M, the injection experiments with 4 pellet injector have been started from April in 1987. An optimization of refueling cycle, impurity transport, improvement of  $\tau_E$ , heating efficiency and high density (density limit), and density profile control during auxiliary heating are investigating.

In the next step, the technical development to make multi-pellet and to obtain high velocity more than 2 km/s is very important in fueling experiments. Three steps of development are proposed, e.g. development of high velocity (2-2.5 km/s) multi-pellet injector, of very high speed (2.5-5 km/s) pellet (single or multi) injector and of hypervelocity pellet injector. These injectors will be tested performance and installed the large machine (JT-60, FER) with long pulse and high-power heated plasmas to study refueling, improvement of plasma confinement, impurity control and density profile control. And the tritium pellet injection system should be developed for FER or a reactor in future, in parallel with the above-mentioned steps as shown in Fig. 39.

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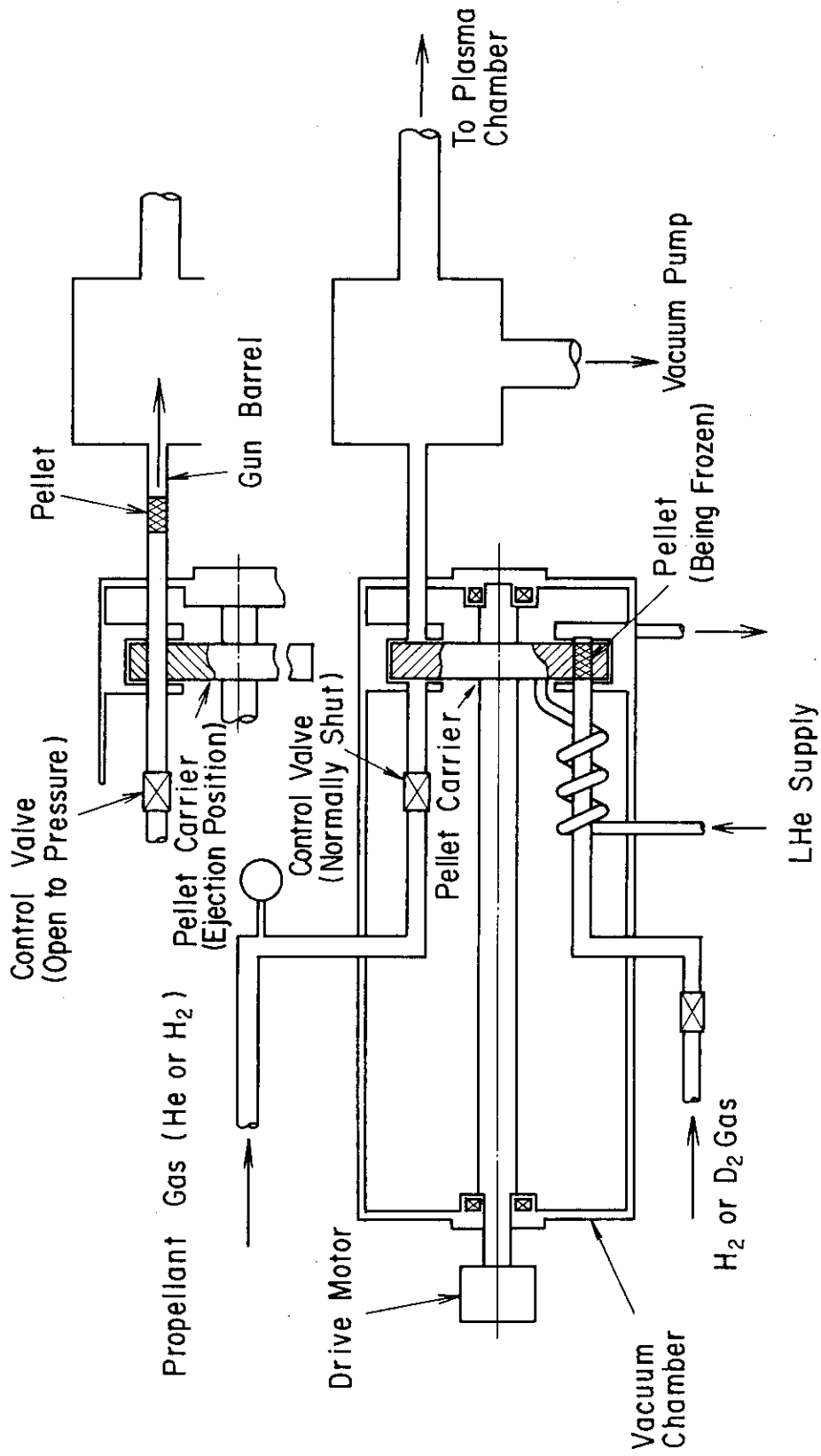
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Table 1 Properties of hydrogen and deuterium gas below critical temperature

	H <sub>2</sub>	D <sub>2</sub>
(1) Mass Density (g/cm <sup>3</sup> ) (Solid at T=4.2°K)	8.93 x 10 <sup>-2</sup>	2.04 x 10 <sup>-1</sup>
(2) Triple Point		
T (°K)	13.8	18.7
P (Torr)	53	169
(3) Sublimation Energy (cal/g) (at 4.2°K)	97.22	71.06



### Pneumatic Pellet Injector

Fig. 1 Principle of pellet injector, H<sub>2</sub> or D<sub>2</sub> gas is frozen by liquid He and the pellet is accelerated by pressurized gas (H<sub>2</sub>, He).

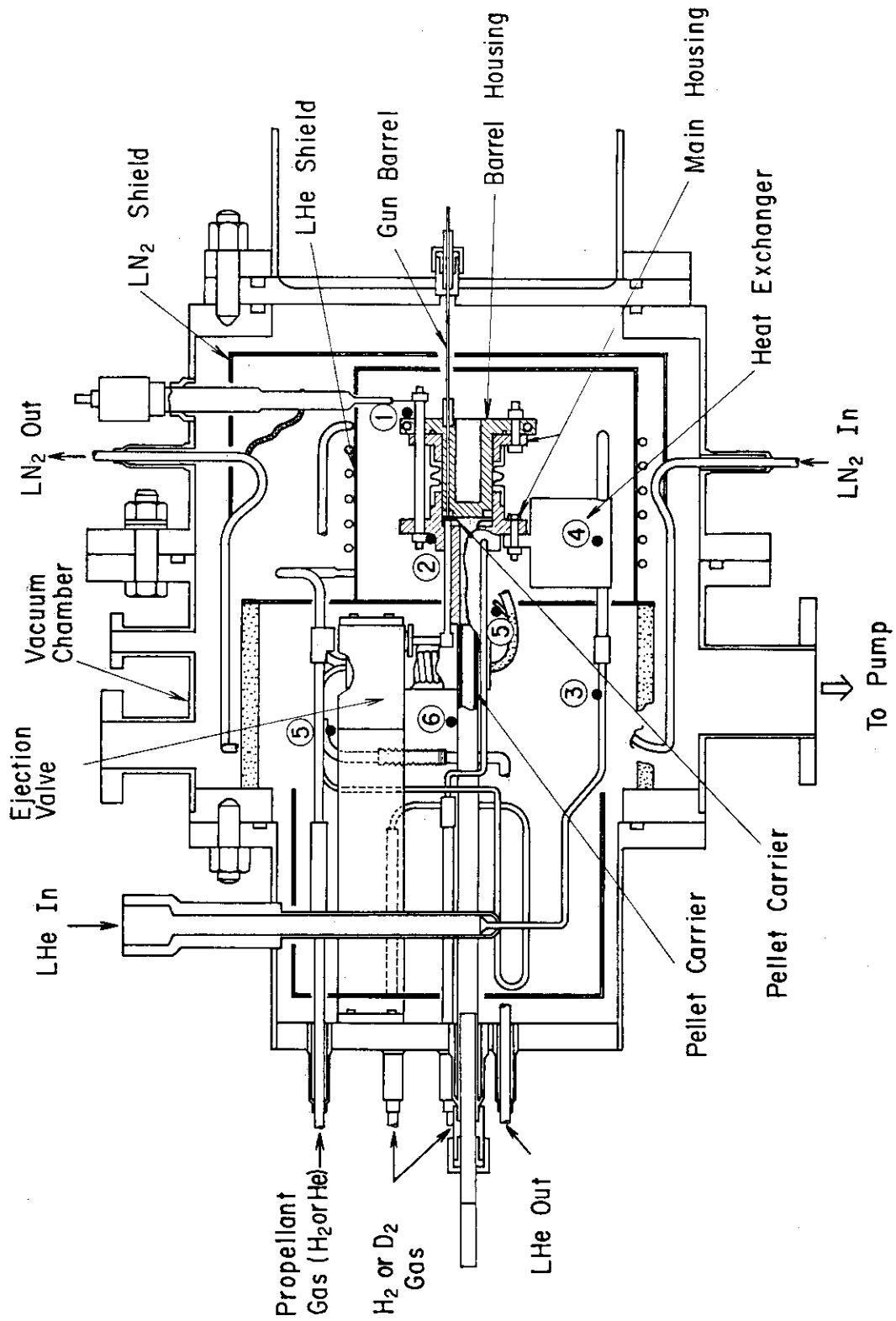
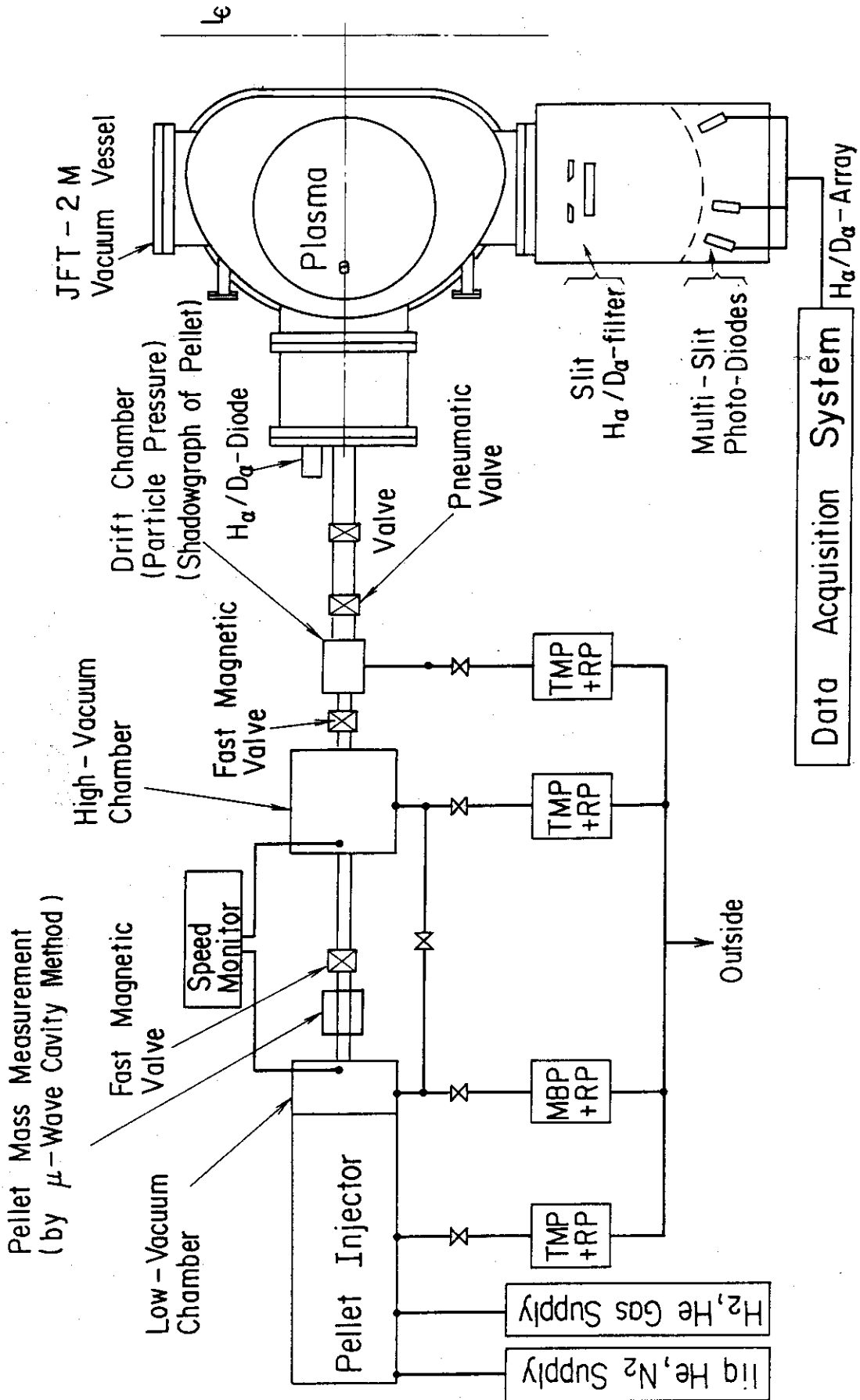


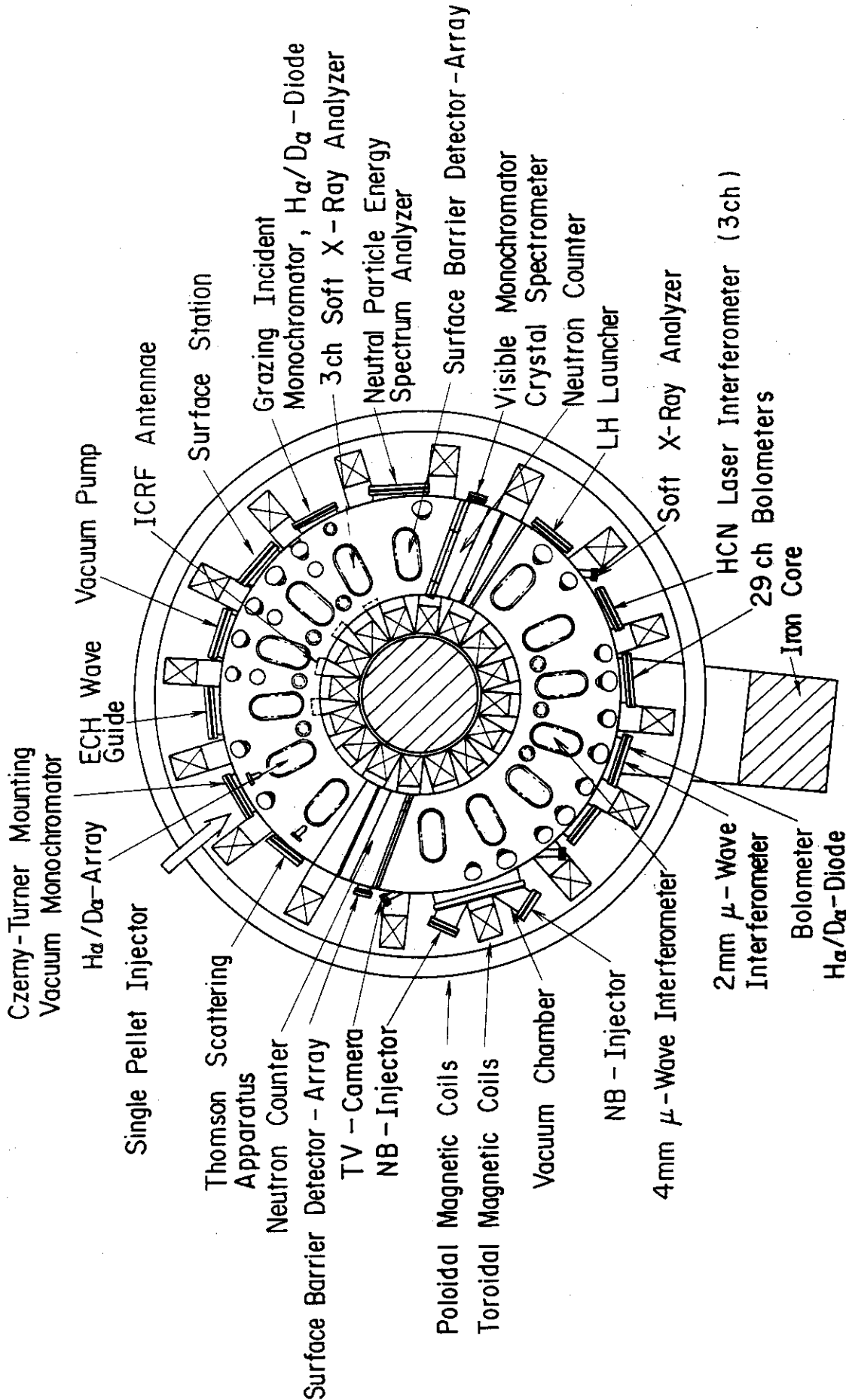
Fig. 2 Pneumatic single-pellet injector for JFT-2M.



### Single Pellet Injector

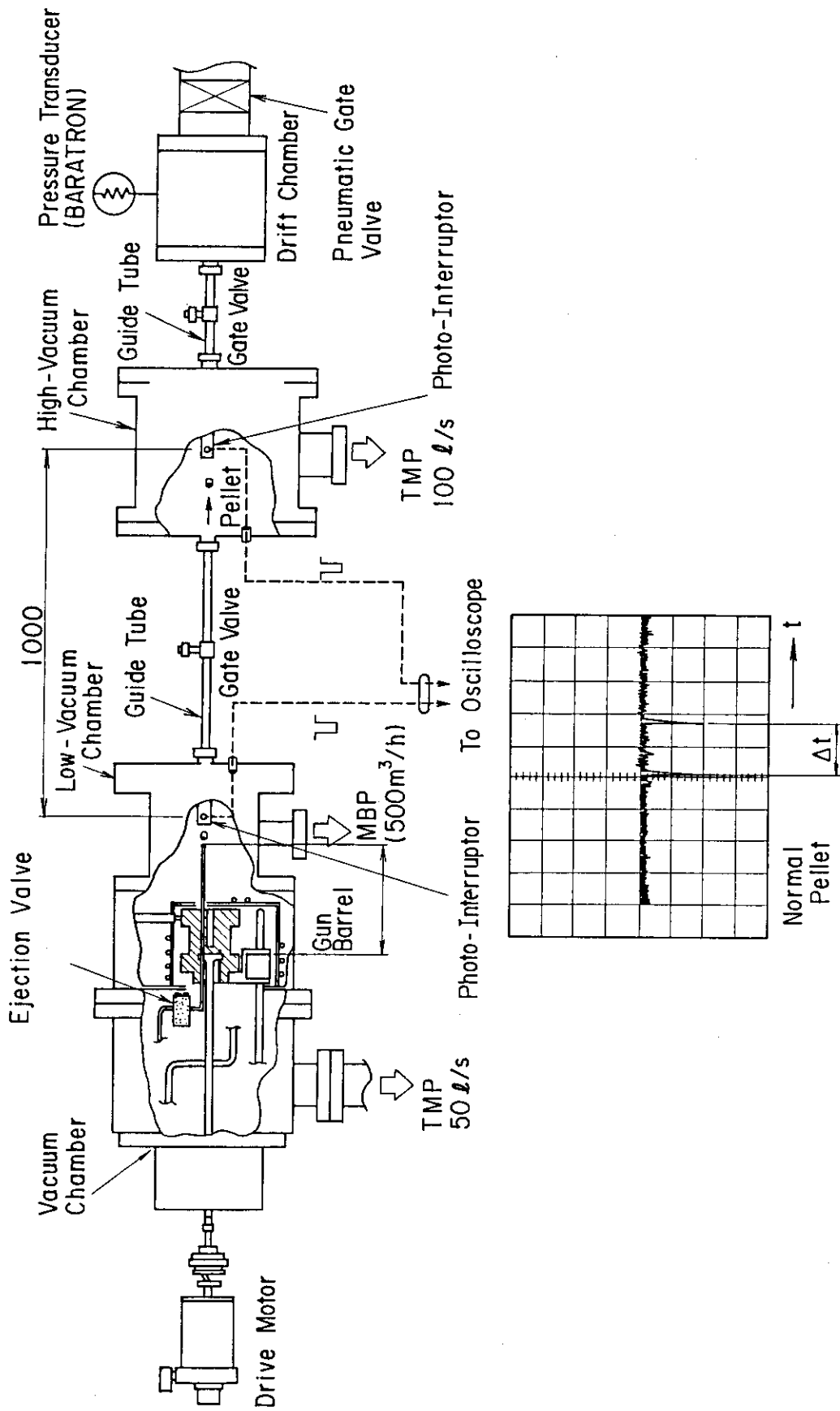
Fig. 3 Schematic diagram of the single-pellet injector and injection line for JFT-2M.





## JFT-2M Tokamak and Diagnostic Apparatuses

Fig. 4 Plane view of JFT-2M, and arrangement of the pellet injector, auxiliary heating system and plasma diagnostic apparatuses.



Signal of Photo-Interruptions on Oscilloscope

Fig. 5 System of pellet velocity measurement.

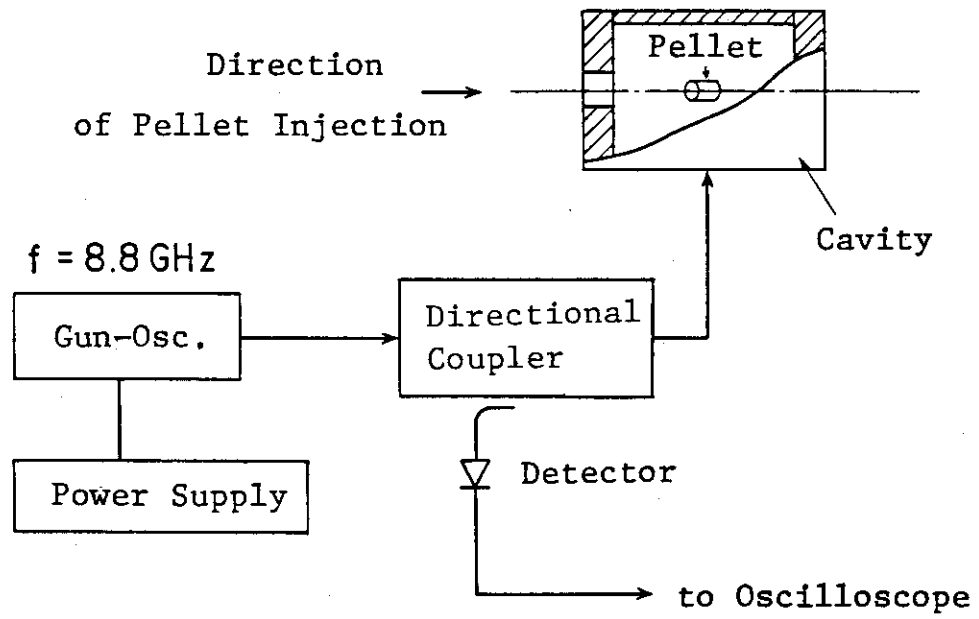


Fig. 6 Block diagram of pellet mass measurement by microwave cavity method.

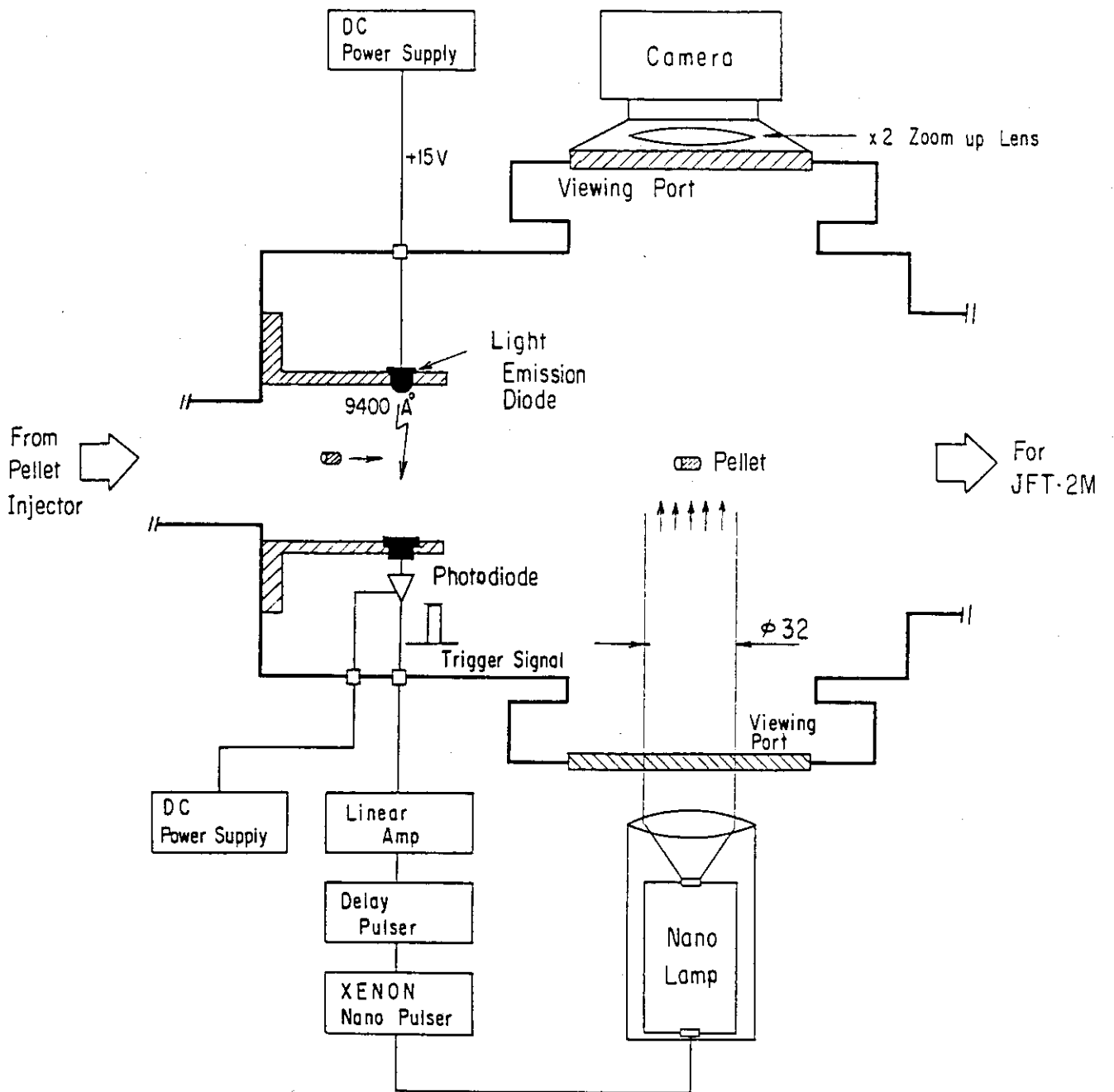


Fig. 7 System of pellet-shadowgraph measurement.

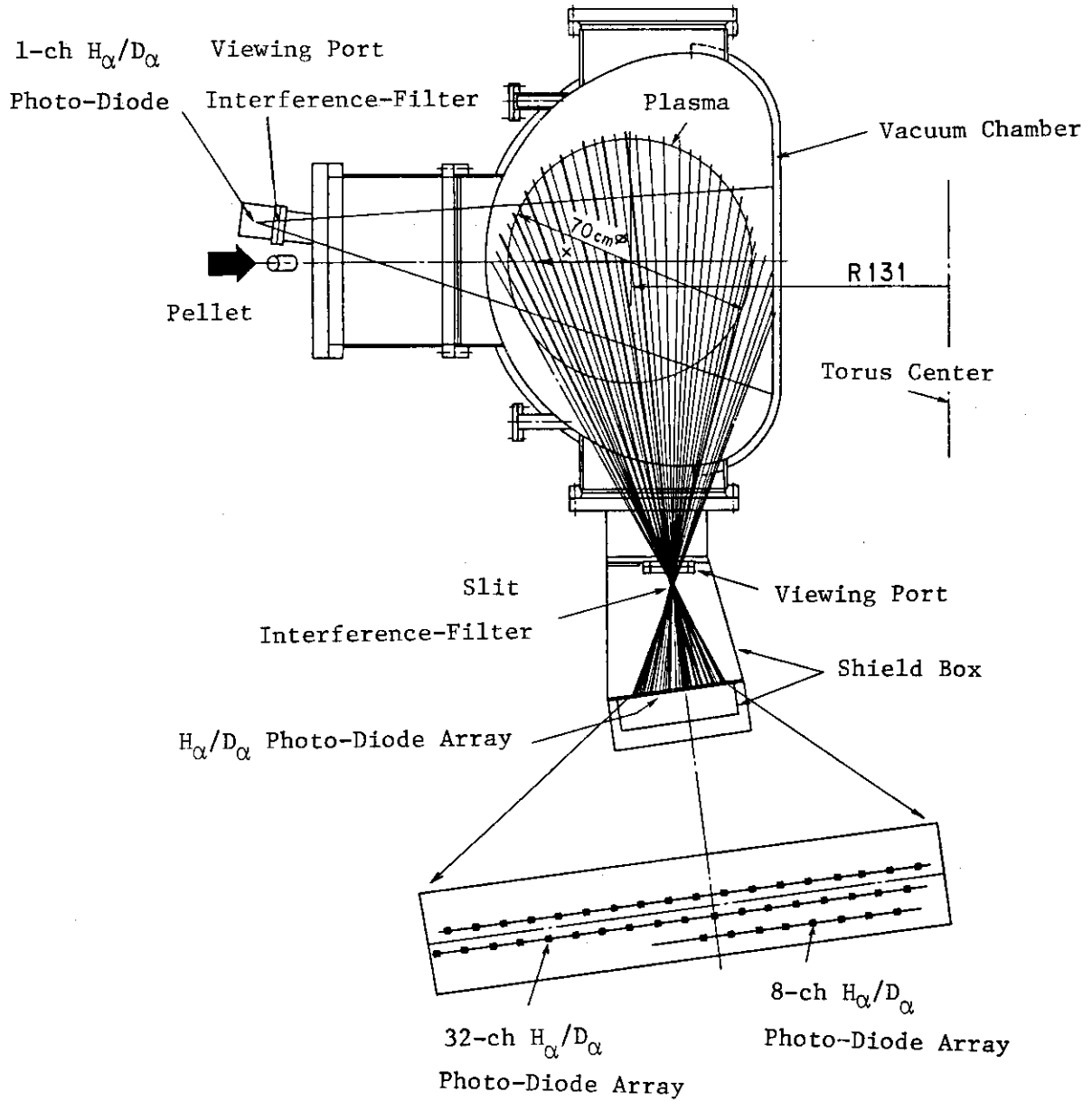


Fig. 8 System of pellet ablation measurement.

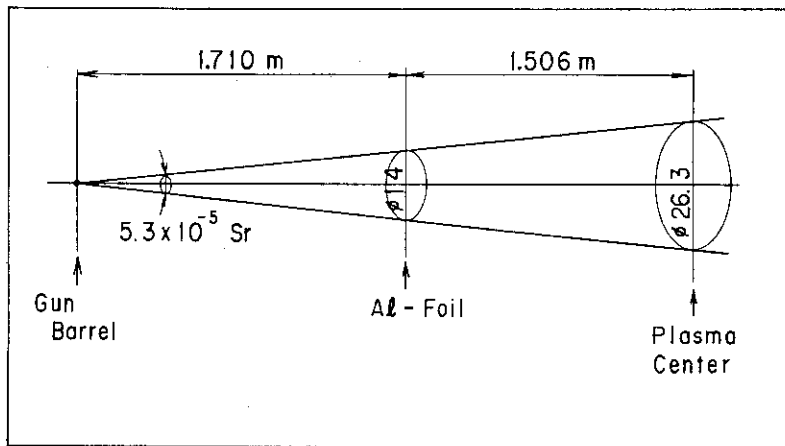
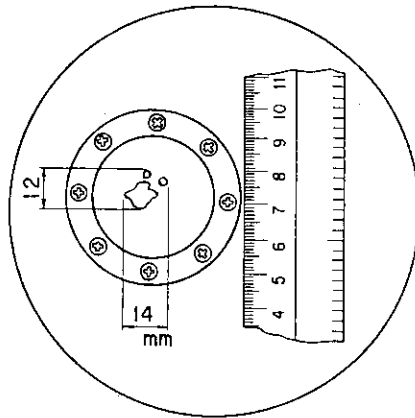


Fig. 9 Measurement of scattering of pellet trajectory.

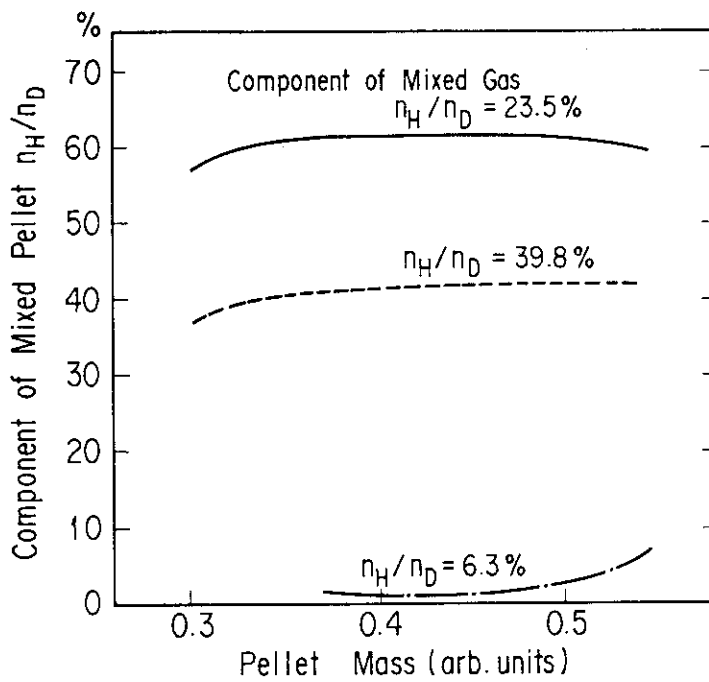


Fig. 10 Component of mixed pellet ( $H_2+D_2$ ),  $n_H/n_D$  versus pellet mass

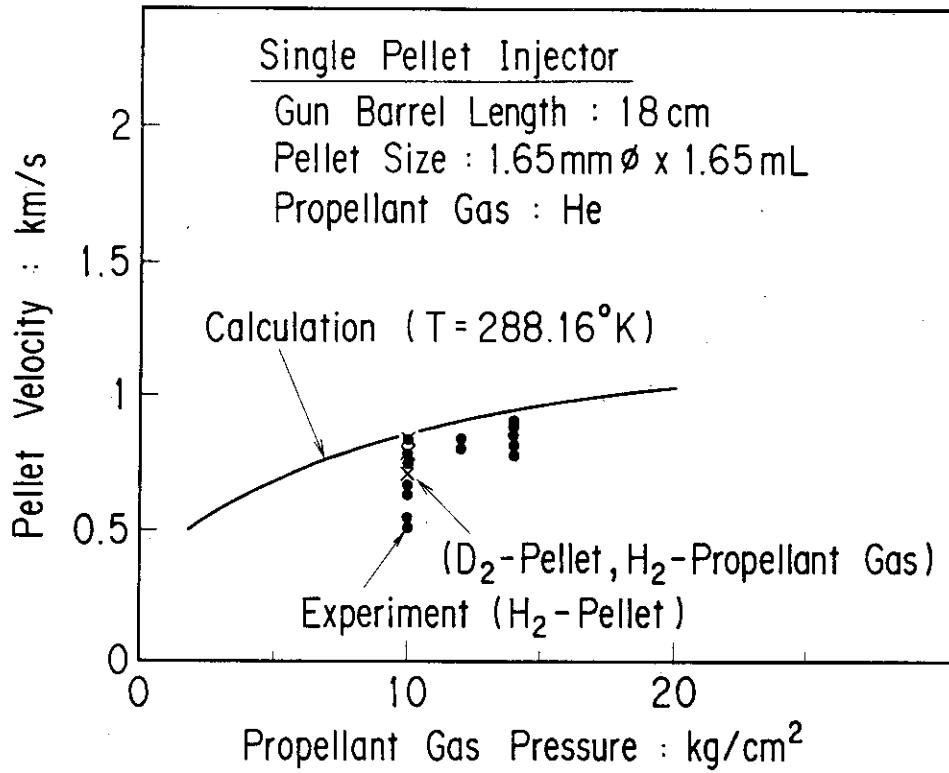


Fig. 11 Dependence of pellet velocity on propellant gas pressure. Closed circles are measured velocity of H<sub>2</sub>-pellet (propellant gas is He) and crosses are measured velocity of D<sub>2</sub>-pellet (propellant gas is H<sub>2</sub>). Solid line is calculated velocity of H<sub>2</sub>-pellet accelerated by He gas.

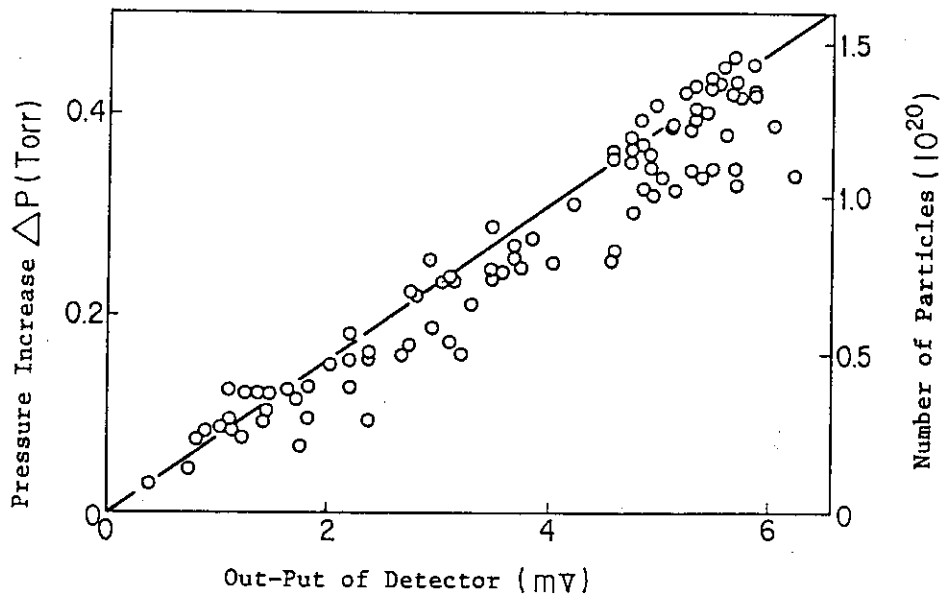


Fig. 12 Pressure rise by evaporation of pellet in the drift chamber versus output of detector in microwave cavity method.

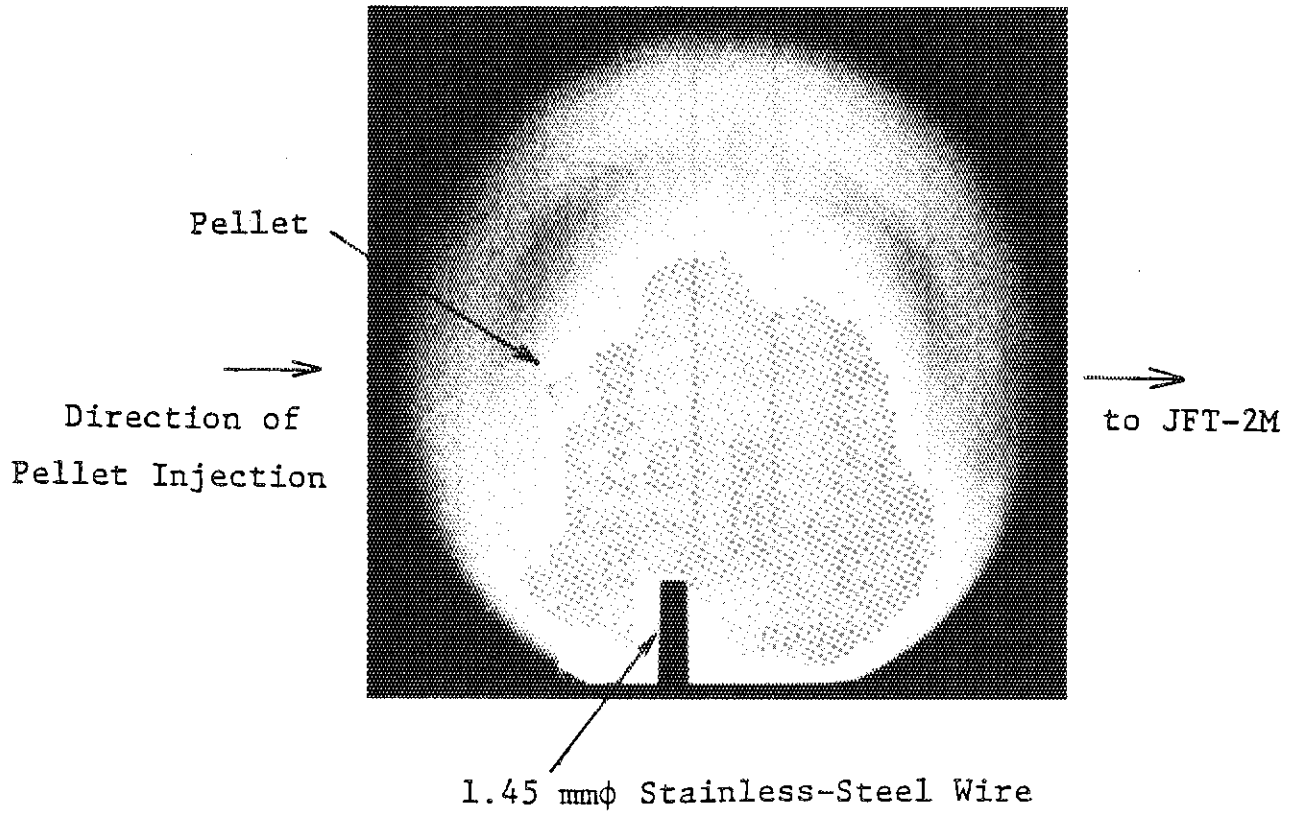


Fig. 13 Shadowgraph of D<sub>2</sub>-pellet, pellet velocity = 769 m/s.



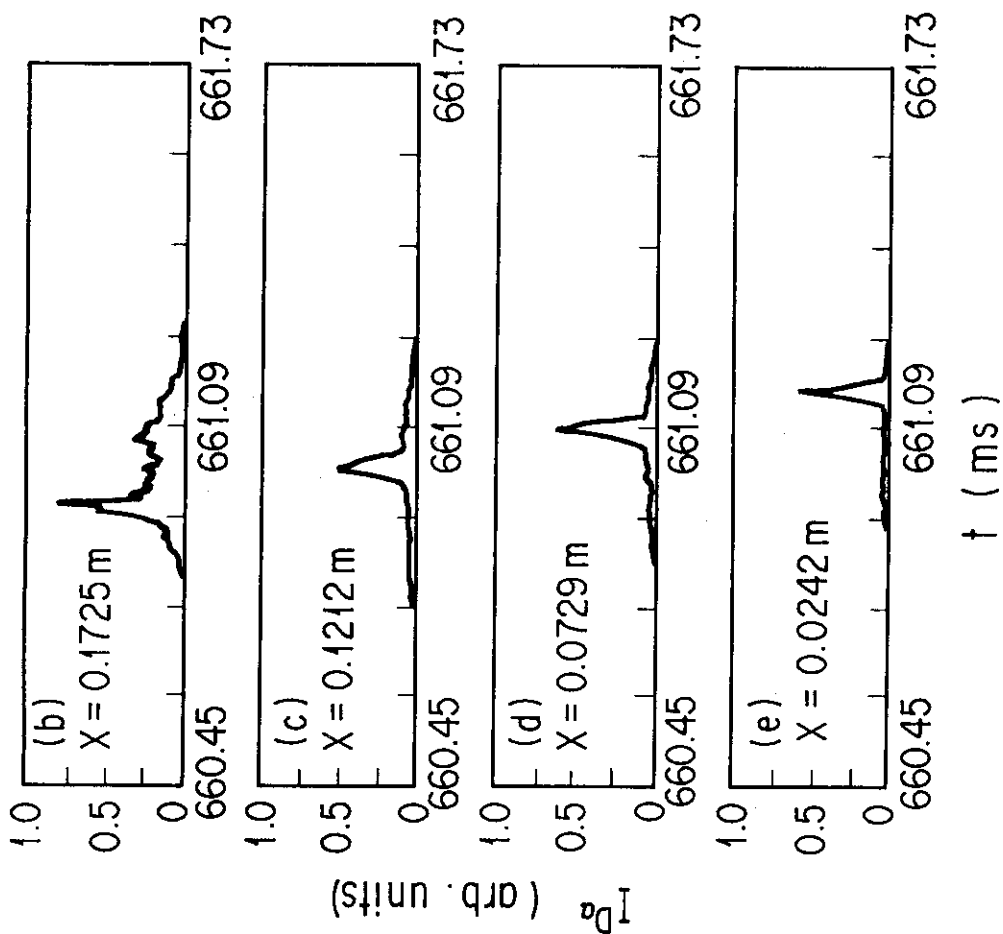


Fig. 14 (a) Ablation profile of D<sub>2</sub>-pellet obtained by using 1 channel photo-diode, (b), (c), (d), (e) time histories of D<sub>α</sub> line intensity at each channel of 8 channel photo-diode array.

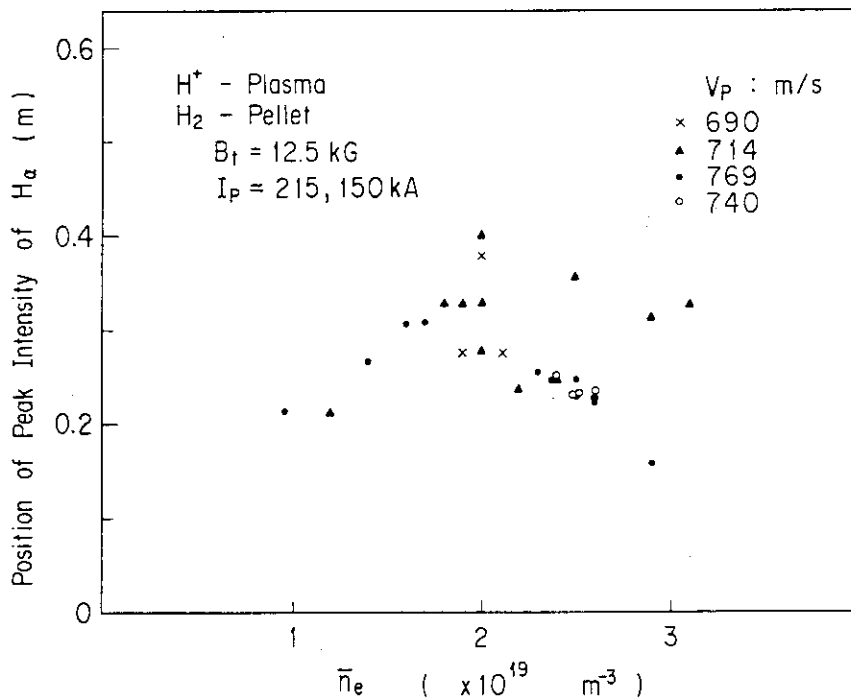


Fig. 15 Position of peak intensity of  $H_\alpha$  line emission (peak deposition of particles) versus line averaged electron density. Ohmic heated discharges, pellet size =  $1 \text{ mm}\phi \times 1 \text{ mmL}$ .

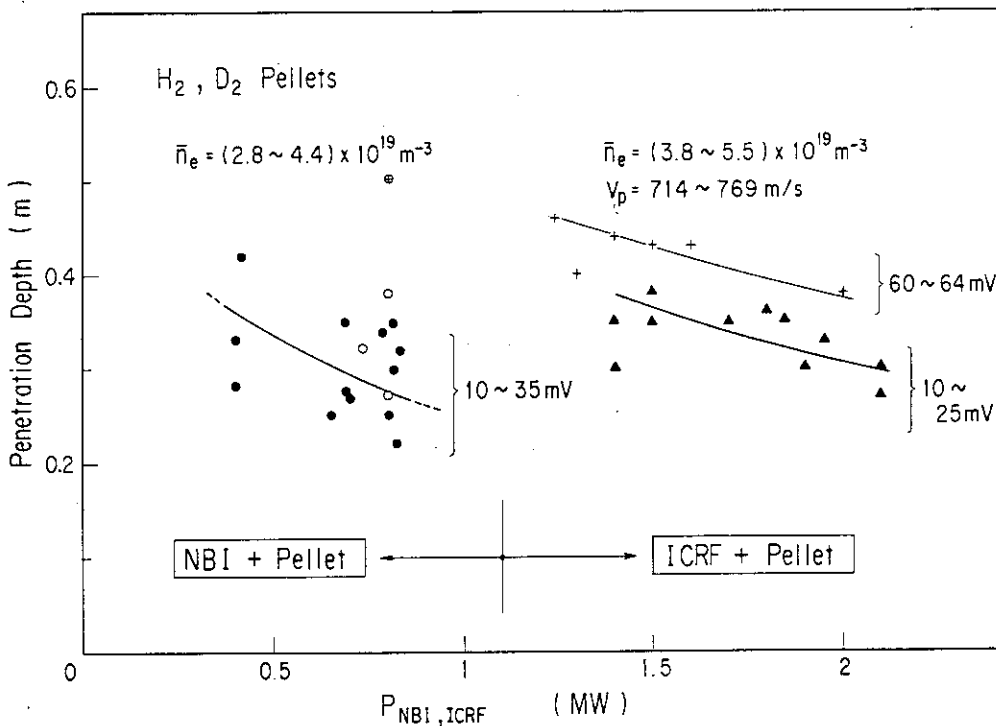


Fig. 16 Penetration depth of pellet ( $H_2$  or  $D_2$ ) in auxiliary-heated plasmas versus power of auxiliary heating, pellet size =  $1.65 \text{ mm}\phi \times 1.65 \text{ mmL}$ .

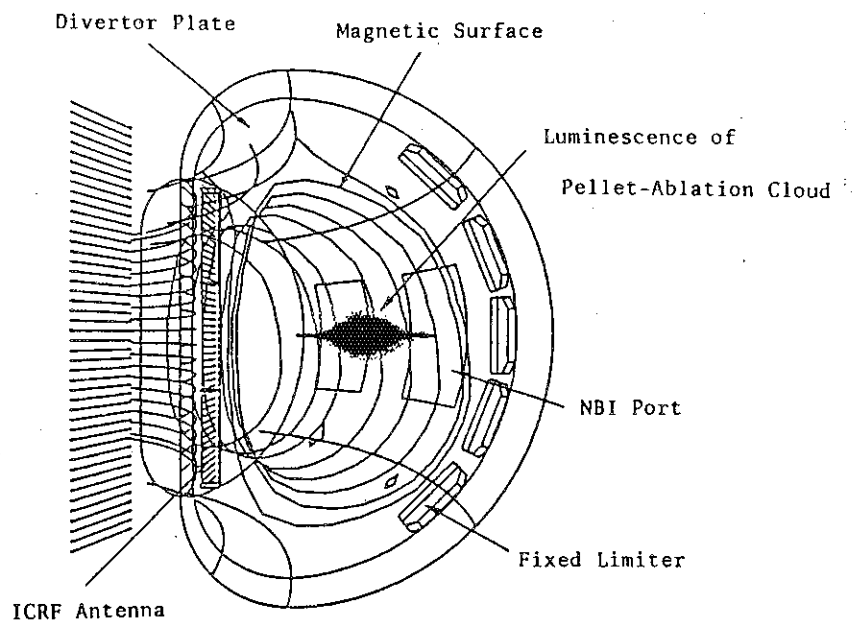
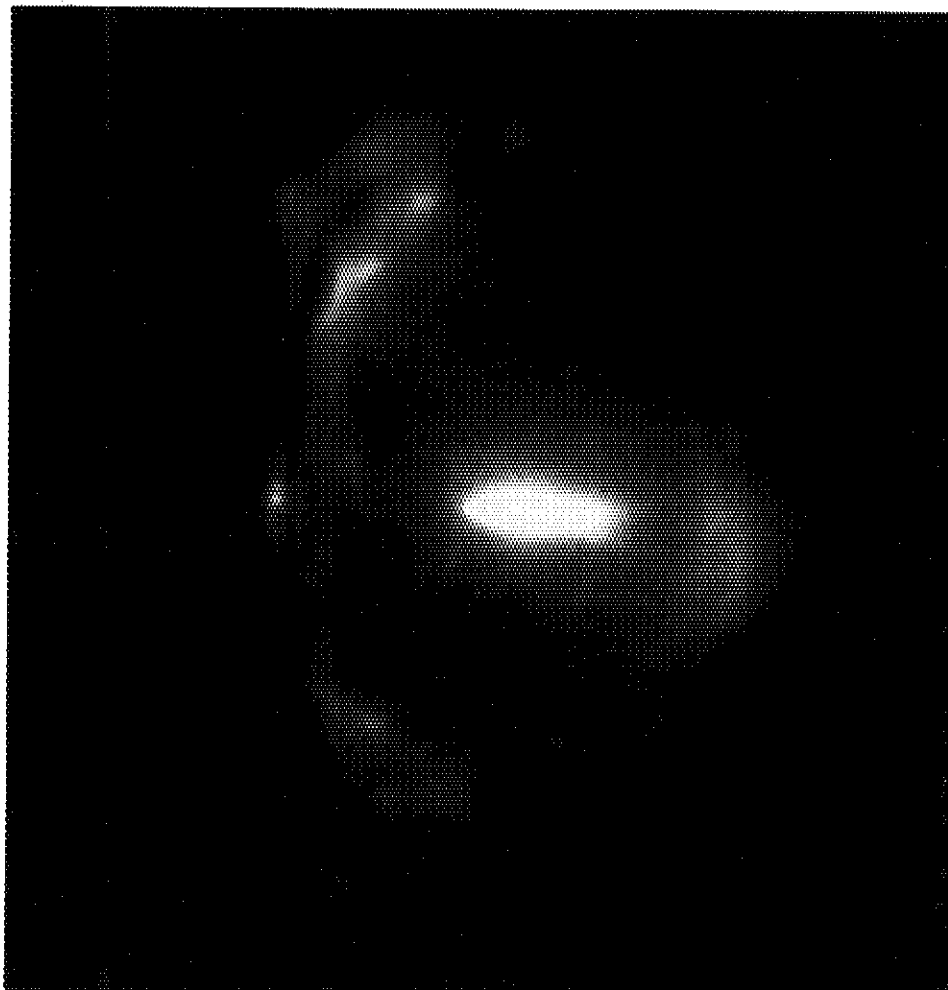


Fig. 17 Photograph of pellet ( $D_2$ ) ablation (emission of  $D_\alpha$  line) in upper single-null divertor discharges with NB (0.88 MW) heating. The pellet is injected from the direction of right hand side.  $B_t = 1.3$  T,  $I_p = 274$  kA,  $V_p = 833$  m/s. The lower picture is three-dimensional expression of pellet ablation by the computer on the upper photograph.

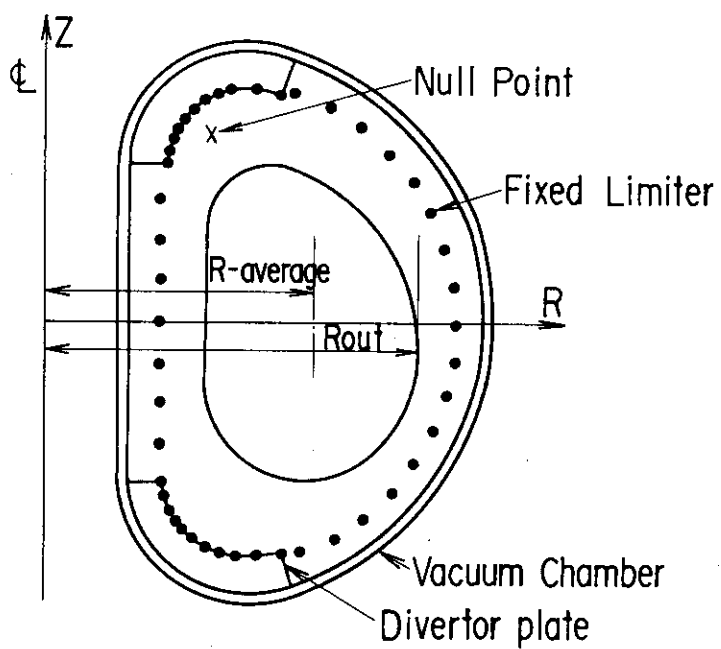


Fig. 18 Magnetic configuration in upper single-null divertor discharges with NB heating.

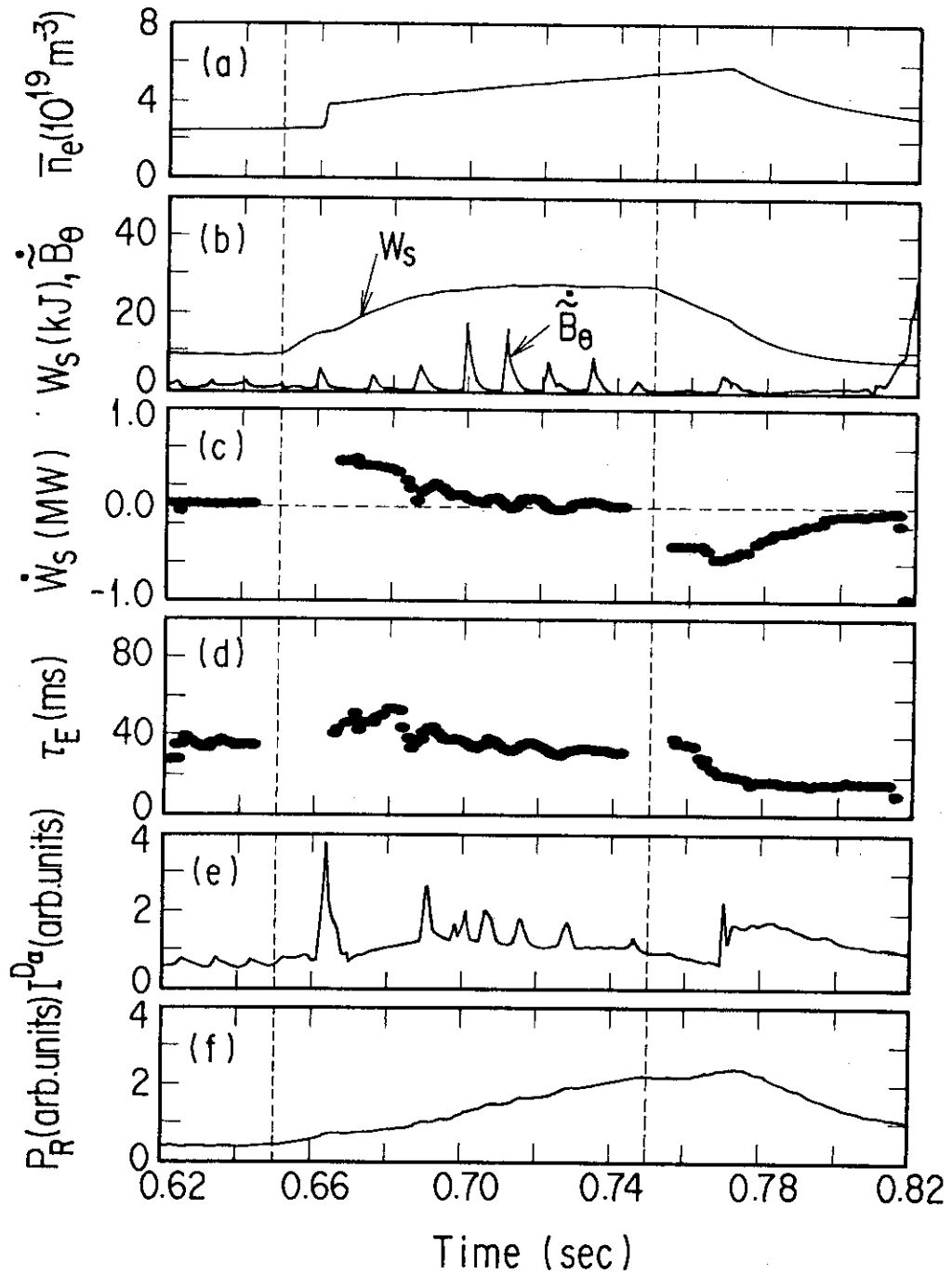


Fig. 19 Time evolutions of line averaged electron density ( $\bar{n}_e$ ), plasma stored energy  $W_s$ , MHD oscillation signal ( $\tilde{B}_\theta$ ), time differential of  $W_s$  ( $\dot{W}_s$ ), global energy confinement time ( $\tau_E$ ), intensity of  $D_\alpha$  line emission ( $I^{D_\alpha}$ ) and radiation loss power including charge exchanged particle loss ( $P_R$ ) in the single-null divertor discharges with neutral beam (NB) heating and small  $D_2$ -pellet injection ( $\Delta\bar{n}_e = 2 \times 10^{19} \text{ m}^{-3}$ ).

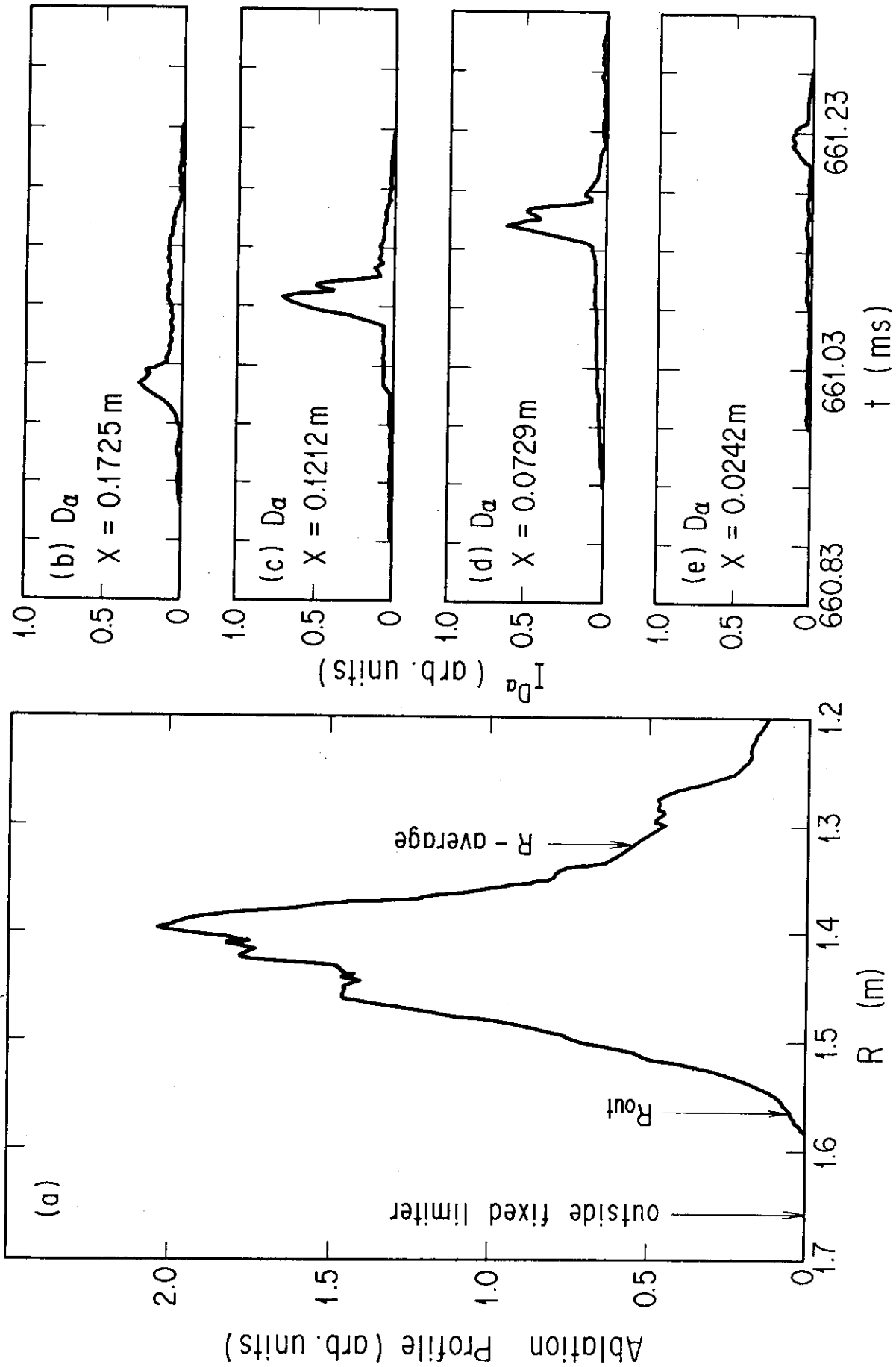


Fig. 20 (a) Ablation profile of D<sub>2</sub>-pellet obtained by using 1 channel photo-diode, (b), (c), (d), (e) time histories of D<sub>α</sub> line intensity at each channel of 8 channel photo-diode array on the discharges in Fig. 19.

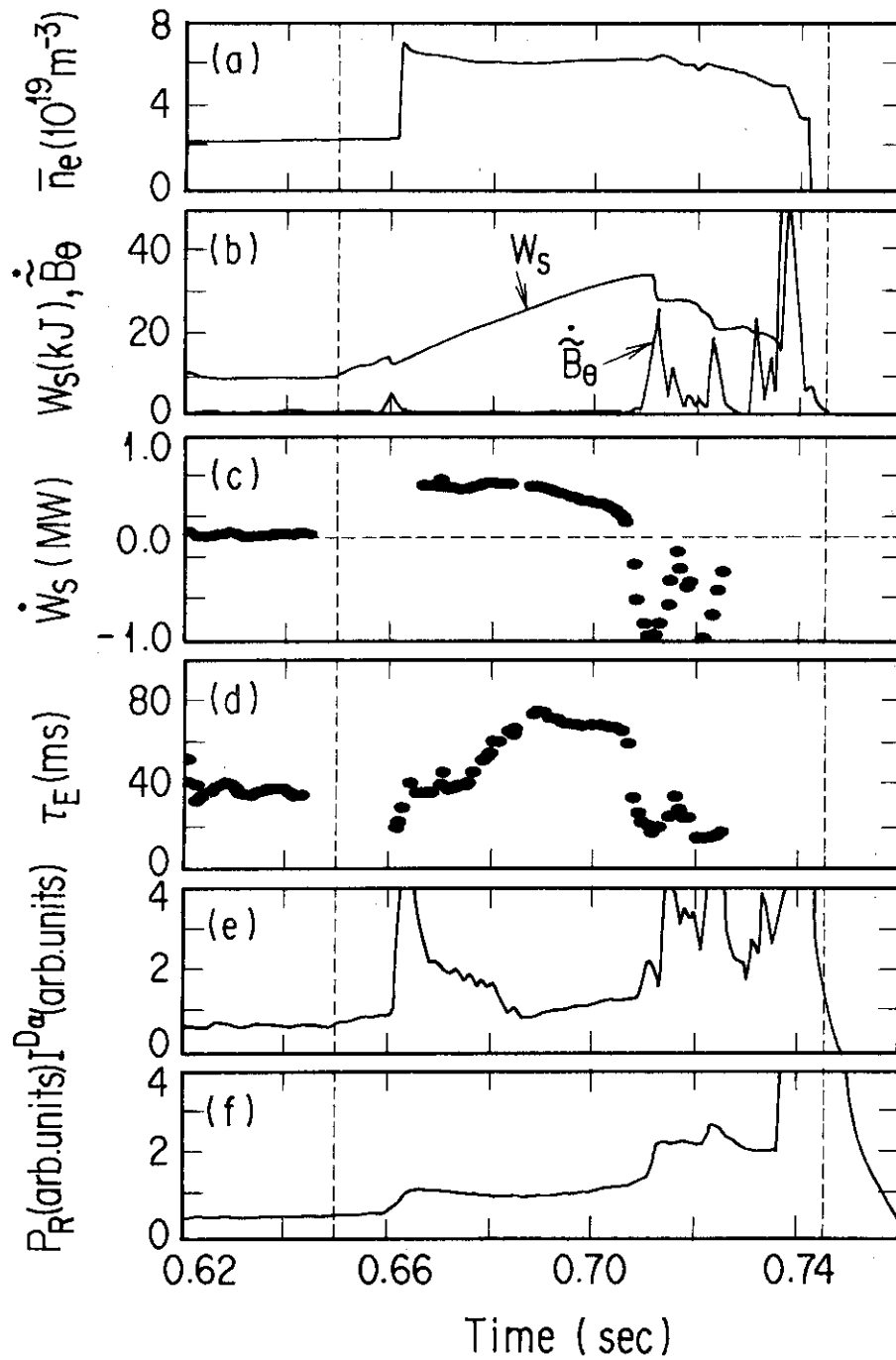


Fig. 21 Time evolutions of plasma parameters in the single-null divertor discharges with NB heating and large D<sub>2</sub>-pellet injection ( $\Delta n_e = 4 \times 10^{19} \text{ m}^{-3}$ ): columns from (a) to (e) are the same as Fig. 19.

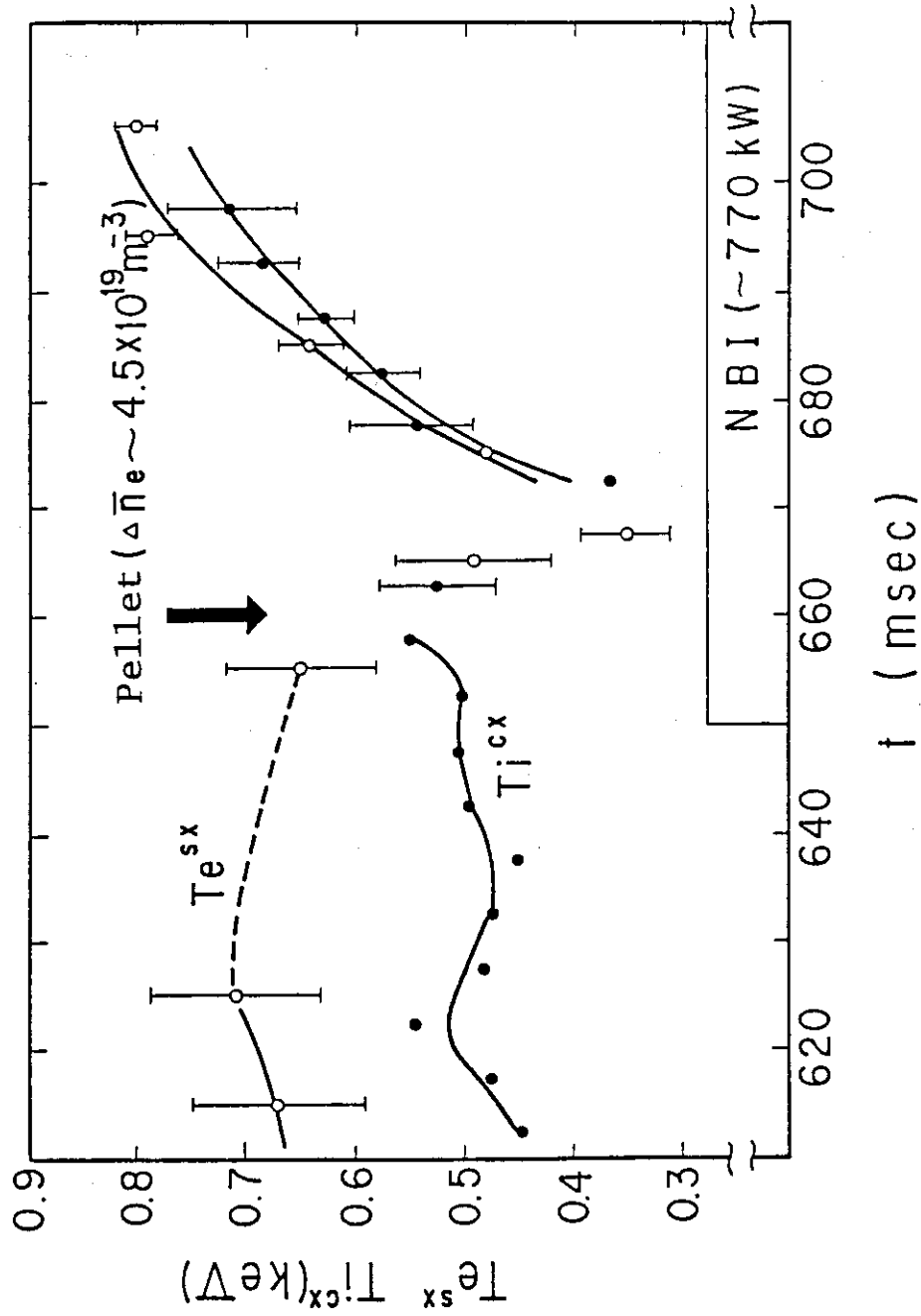


Fig. 22 Time evolution of electron temperature from soft X-ray analysis and ion temperature from charge exchanged neutral particle analysis in the large pellet injected discharges.



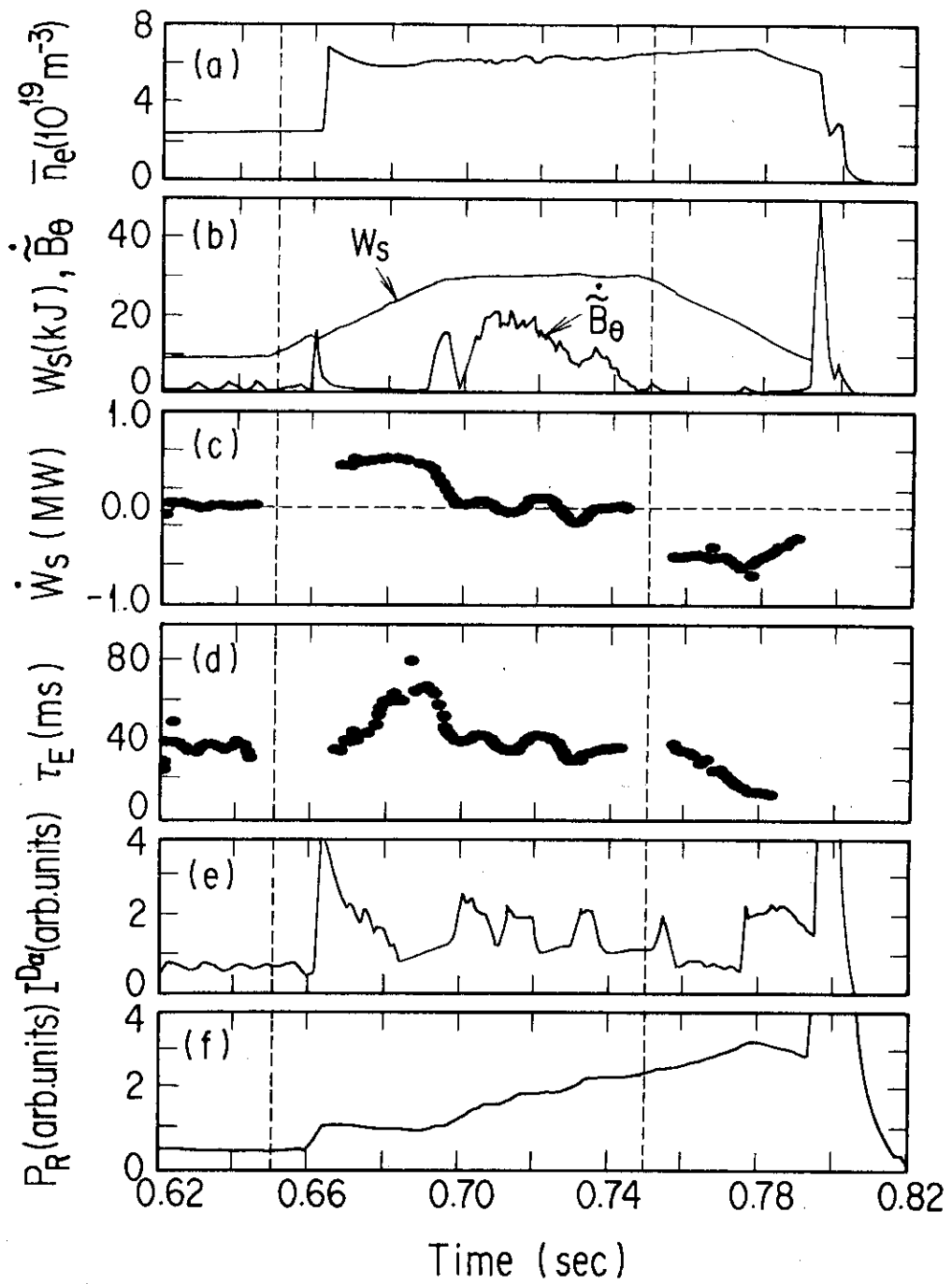


Fig. 23 Time evolutions of plasma parameters in the single-null divertor discharges with NB heating and large D<sub>2</sub>-pellet injection ( $\Delta n_e = 4 \times 10^{19} \text{ m}^{-3}$ ): columns from (a) to (e) are the same as Figs. 19 and 21.

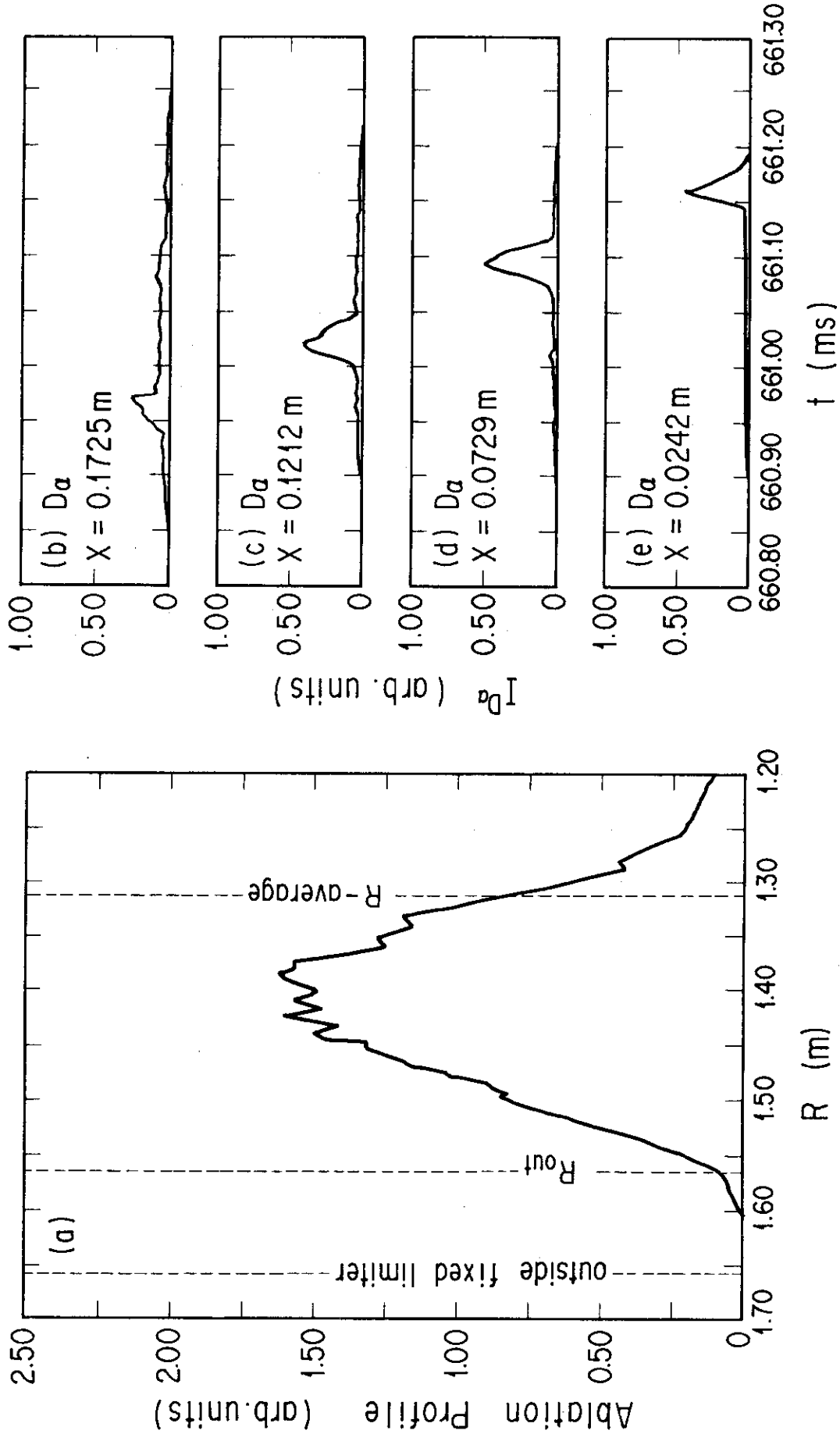


Fig. 24 (a) Ablation profile of D<sub>2</sub>-pellet obtained by using 1 channel photo-diode, (b), (c), (d), (e) time evolutions of D<sub>α</sub> line intensity at each channel of 8 channel photo-diode on the discharges in Fig. 23.

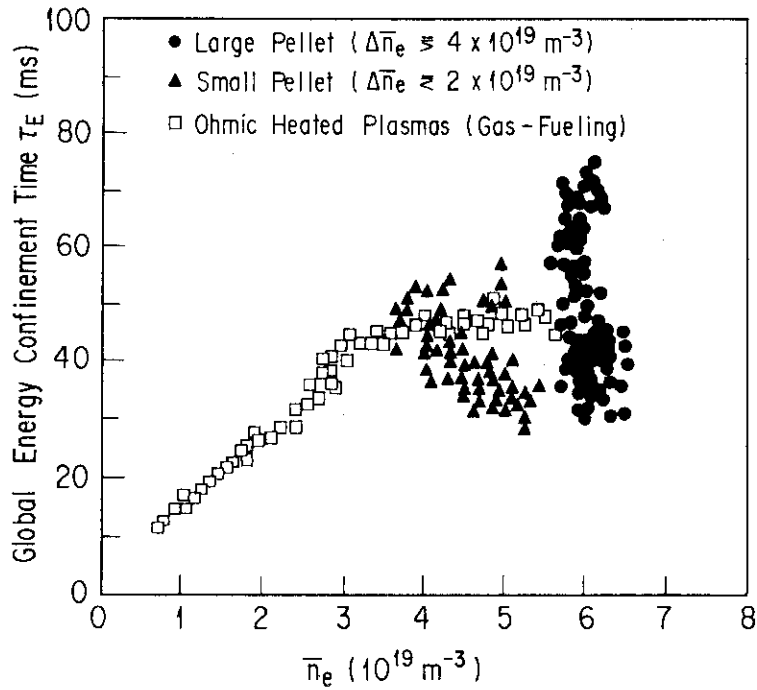


Fig. 25 Global energy confinement time versus line averaged electron density in the single-null divertor discharges with NB heating (pellet is injected into L-mode).

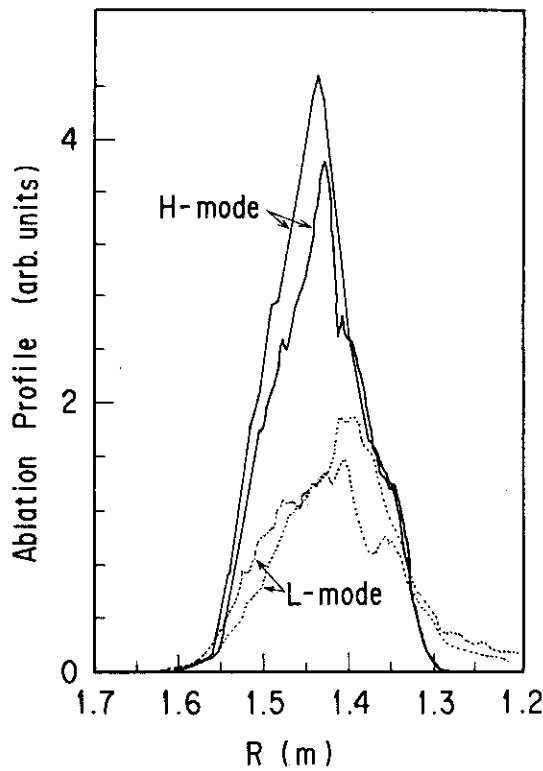


Fig. 26 Ablation profiles of  $D_2$ -pellet. Solid and dotted lines are profiles in pellet injection into H-mode and L-mode of the NB heated single-null divertor discharges.

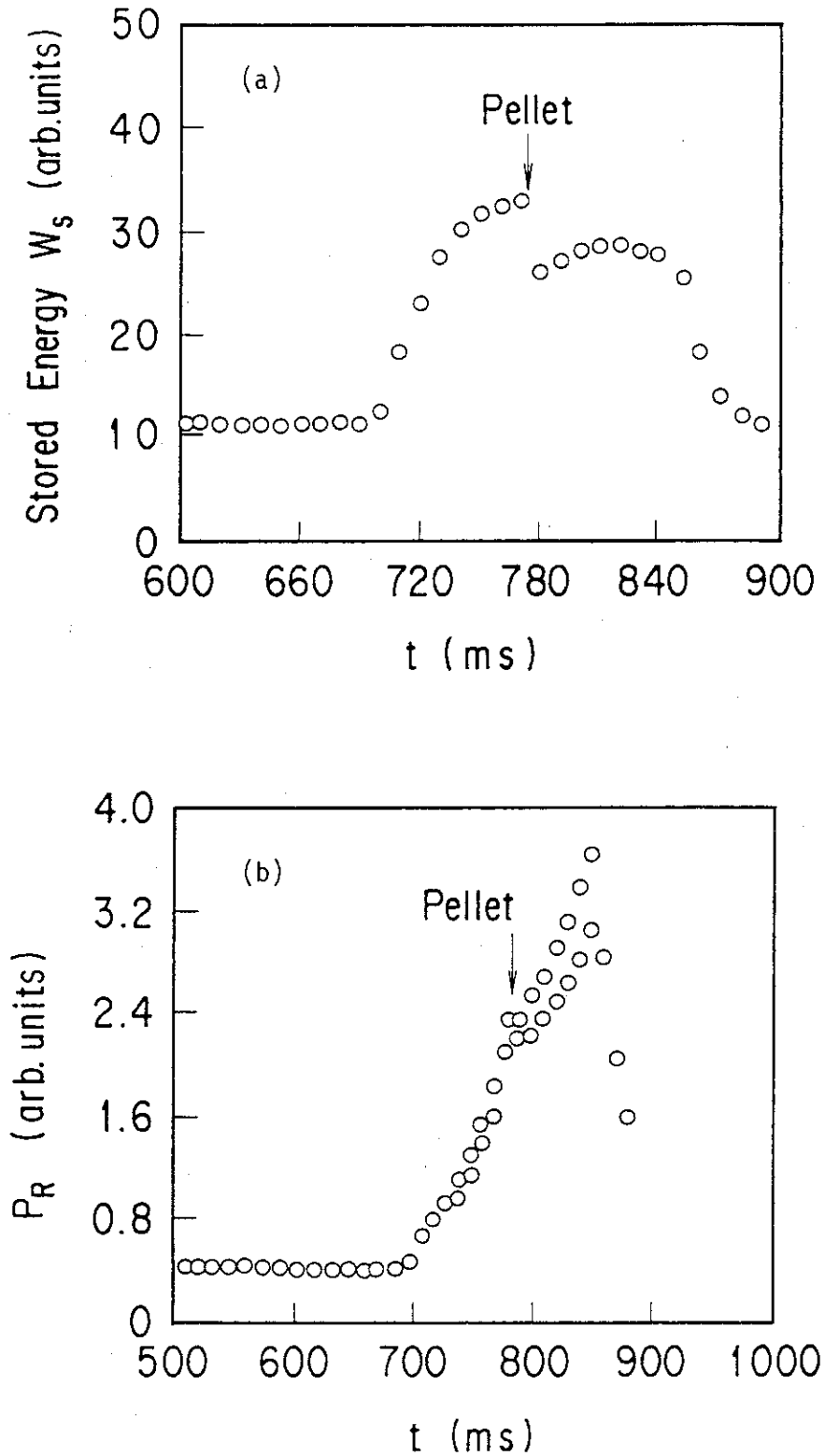


Fig. 27 (a) Time history of plasma stored energy ( $W_s$ ), (b) time history of radiation loss power including charge exchanged particle loss ( $P_R$ ) in pellet injection into H-mode of NB heated single-null divertor discharges.

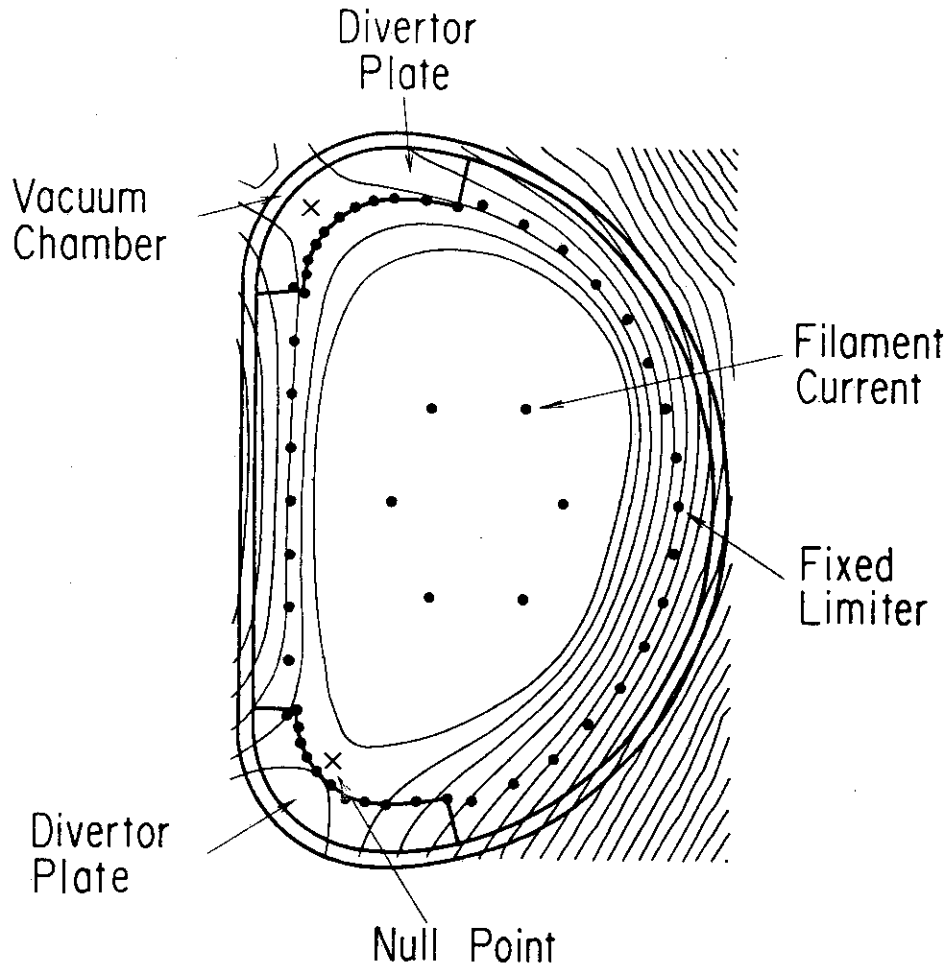


Fig. 28 Magnetic configuration in  $D_2$ -pellet injection into ICRF heated single-null divertor discharges.

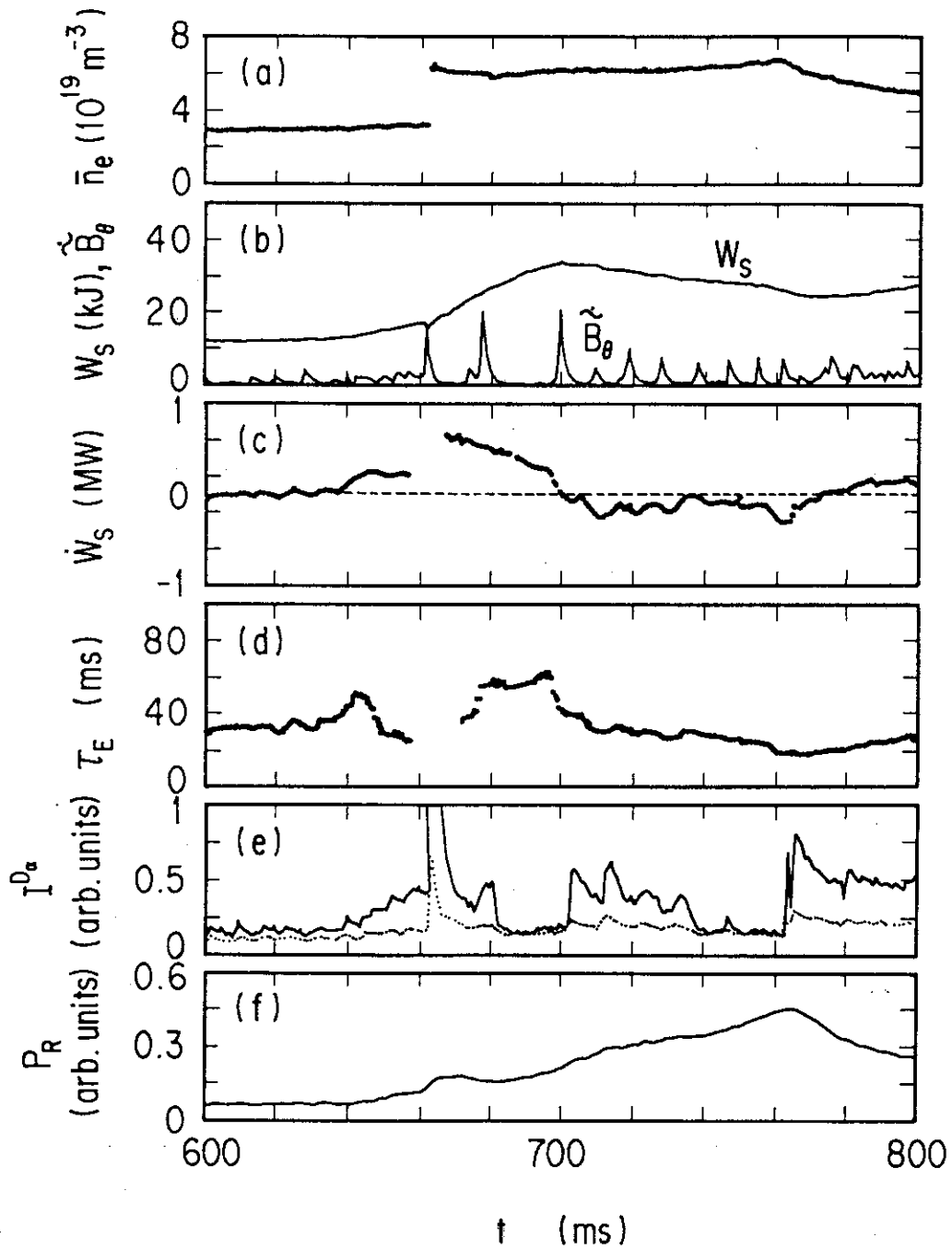


Fig. 29 Time evolutions of line averaged electron density ( $\bar{n}_e$ ), plasma stored energy ( $W_s$ ), Mirnov oscillation signal ( $\tilde{B}_\theta$ ), time differential of  $W_s$  ( $\dot{W}_s$ ), global energy confinement time ( $\tau_E$ ), intensities of  $D_\alpha$  line emission in the divertor (solid line) and main plasma (dotted line) regions ( $I^{D_\alpha}$ ), radiation loss power including charge exchanged particle loss ( $P_R$ ) in the ICRF heated single-null divertor discharges. The pellet is injected into the L-mode at 660 ms and ICRF heating is from 600 to 900 ms.

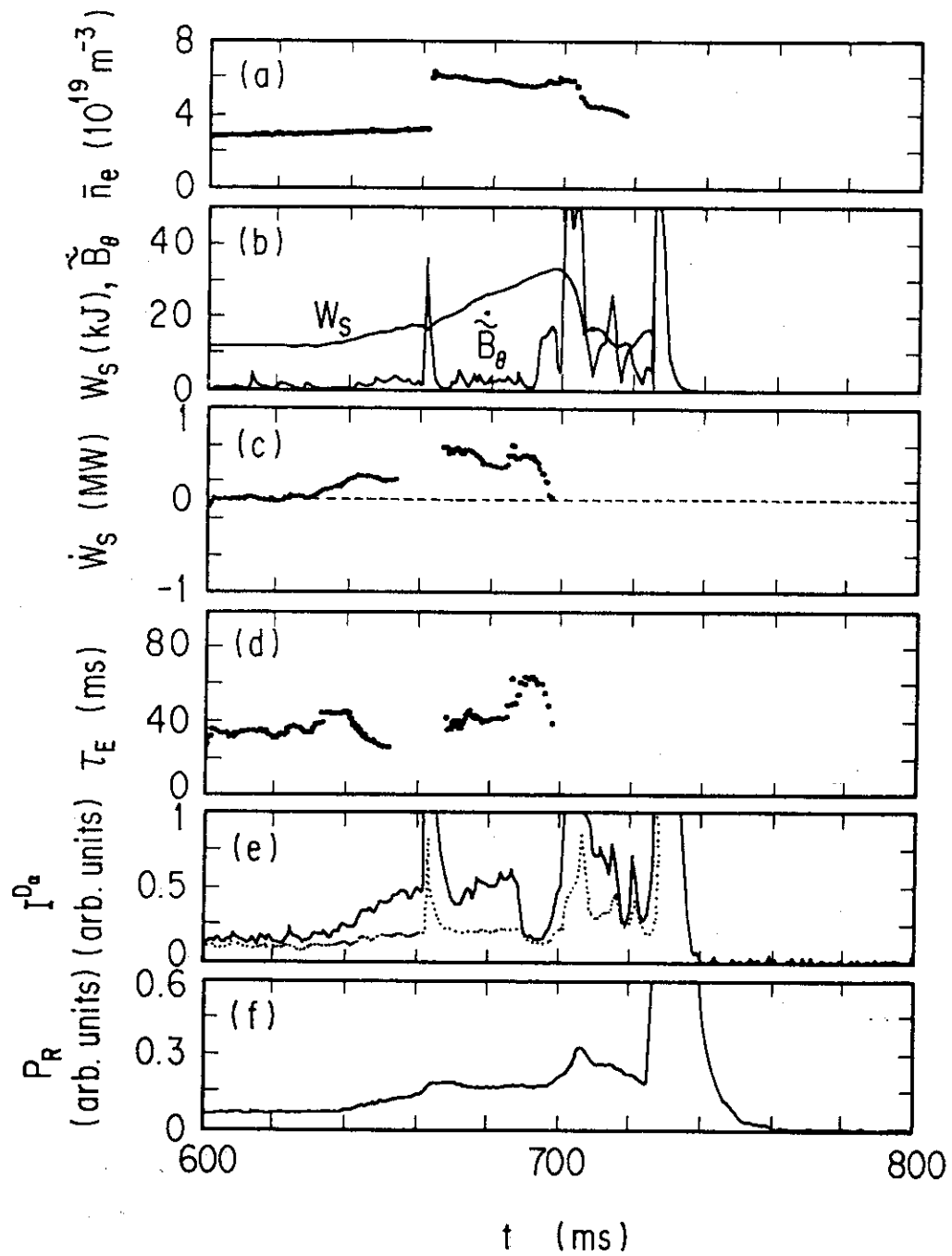


Fig. 30 Time evolutions of plasma parameters in ICRF heated single-null divertor discharges : columns are the same as Fig. 29. The pellet is injected into the L-mode at 660 ms.

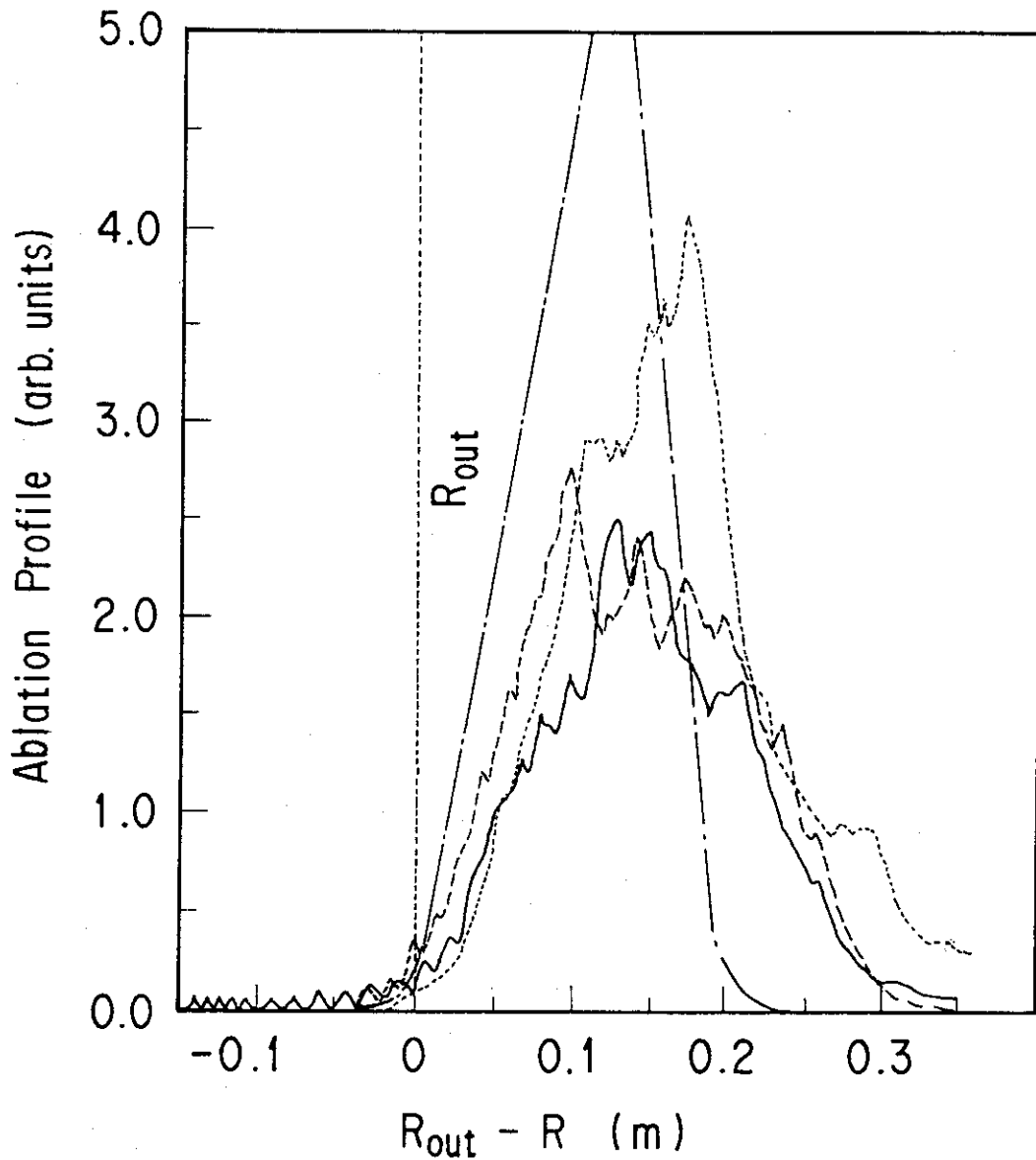


Fig. 31 Ablation profiles of D<sub>2</sub>-pellet. Solid and broken lines are profiles in pellet injection into L-mode and H-mode of ICRF heated single-null divertor discharges, respectively. Dotted and dash-dotted lines are profiles in pellet injection into L-mode and H-mode of NB heated single-null divertor discharges.



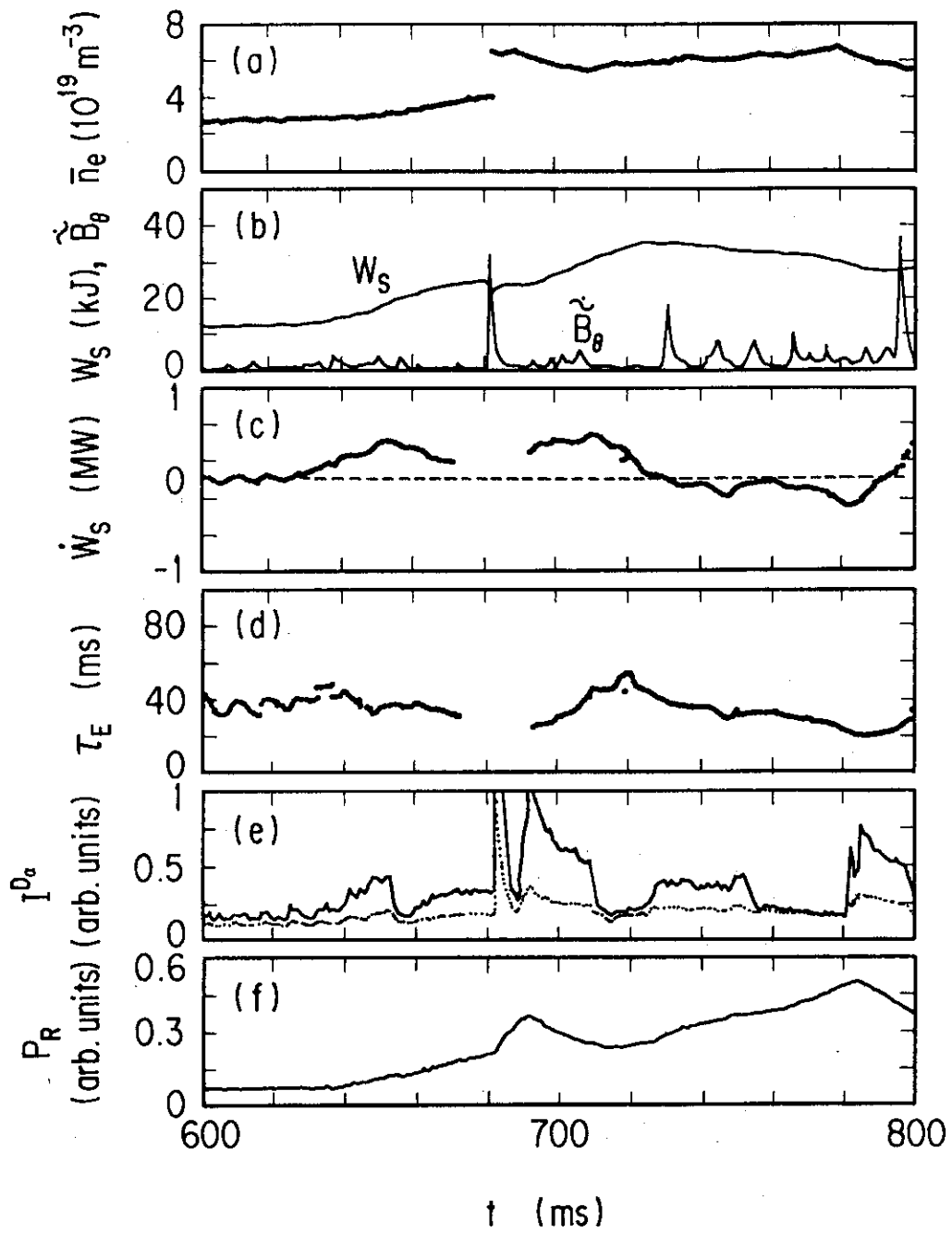


Fig. 32 Time evolutions of plasma parameters in D<sub>2</sub>-pellet injection into the H-mode of ICRF heated single-null divertor discharges. Columns are the same as Figs. 29 and 30.

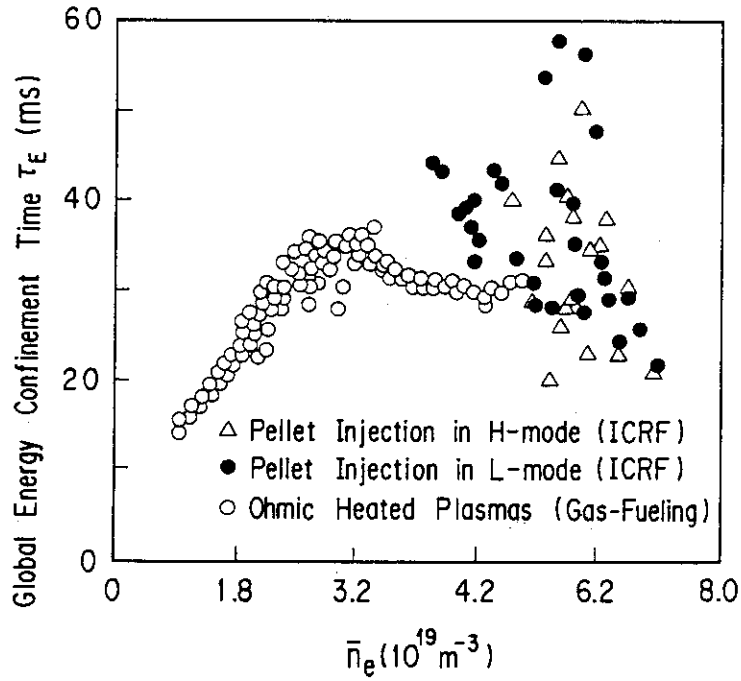


Fig. 33 Global energy confinement time versus line averaged electron density in ICRF heated single-null divertor discharges.

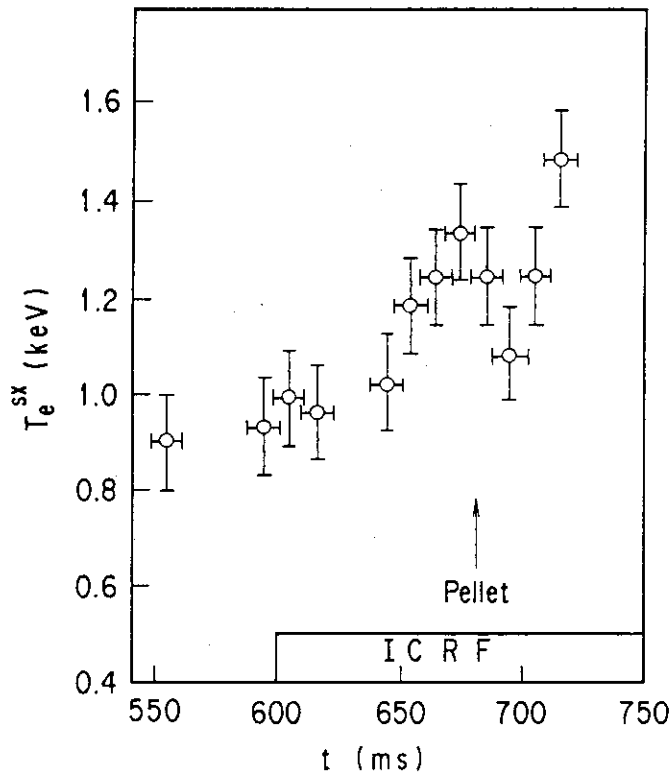


Fig. 34 Time evolution of electron temperature from soft X-ray analysis in  $D_2$ -pellet injection into the H-mode of ICRF heated single-null divertor discharges in Fig. 32.

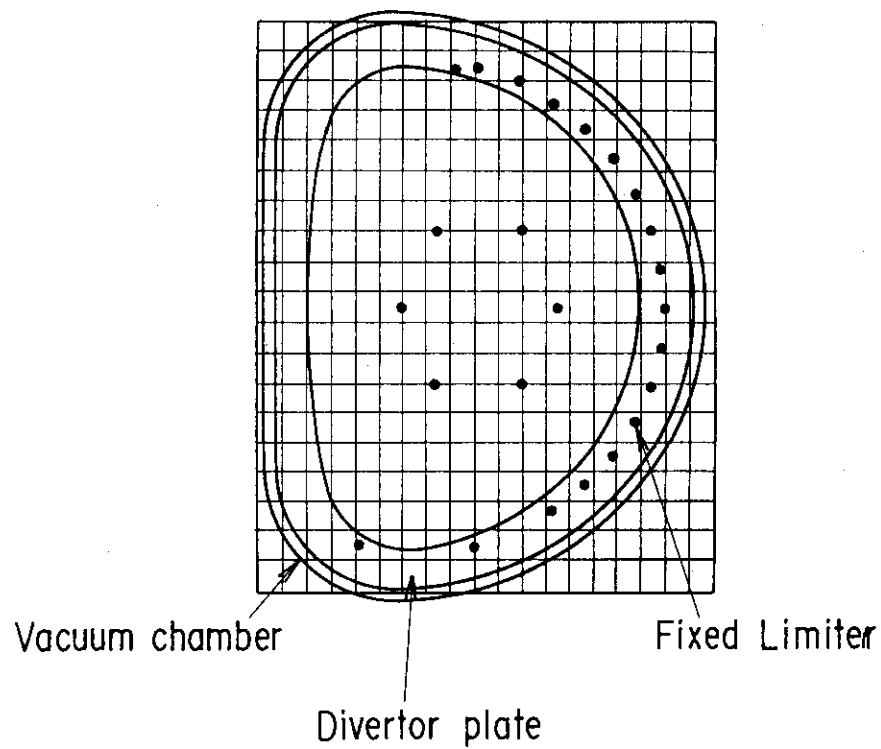


Fig. 35 Magnetic configuration in NB + ICRF heated limiter discharges with  $D_2$ -pellet injection.

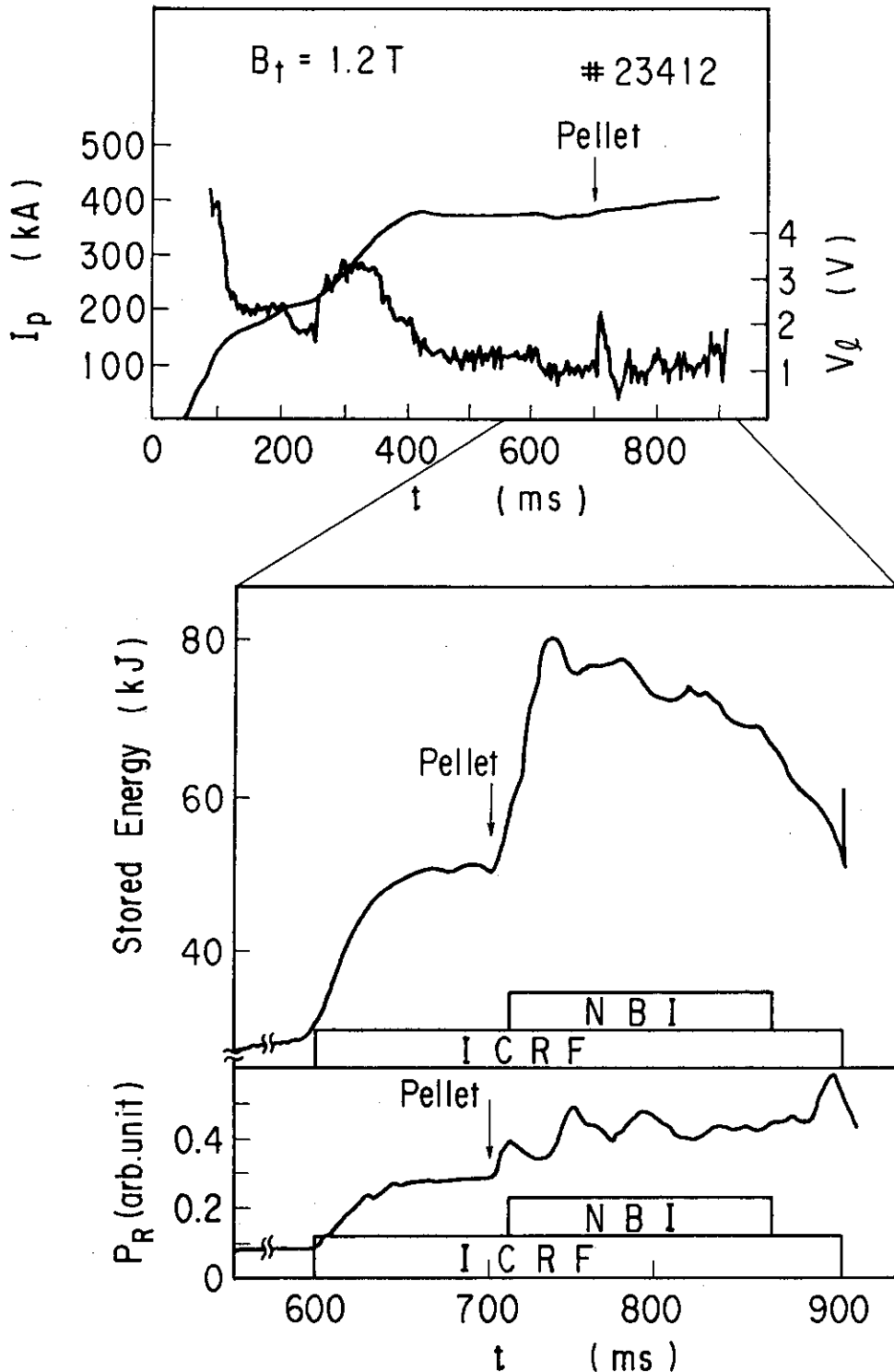


Fig. 36 Time evolution of plasma current ( $I_p$ ), loop voltage ( $V_l$ ), plasma stored energy ( $W_s$ ) and radiation loss power including charge exchanged particle loss ( $P_R$ ) in  $D_2$ -pellet injection into NB+ICRF heated limiter discharges.

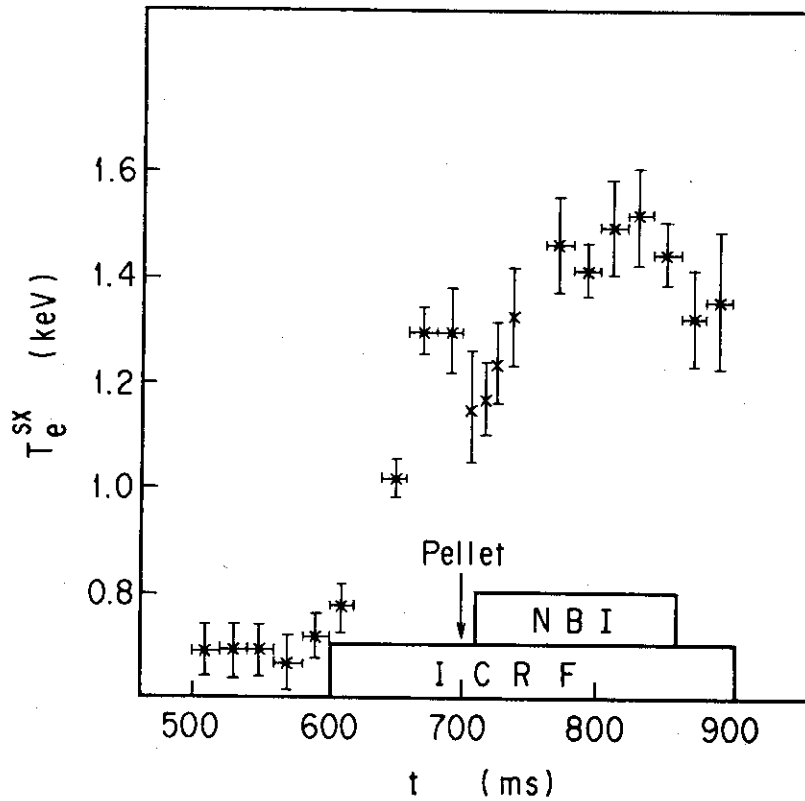


Fig. 37 Time evolutions of electron temperature from soft X-ray analysis on the discharges in Fig. 36.

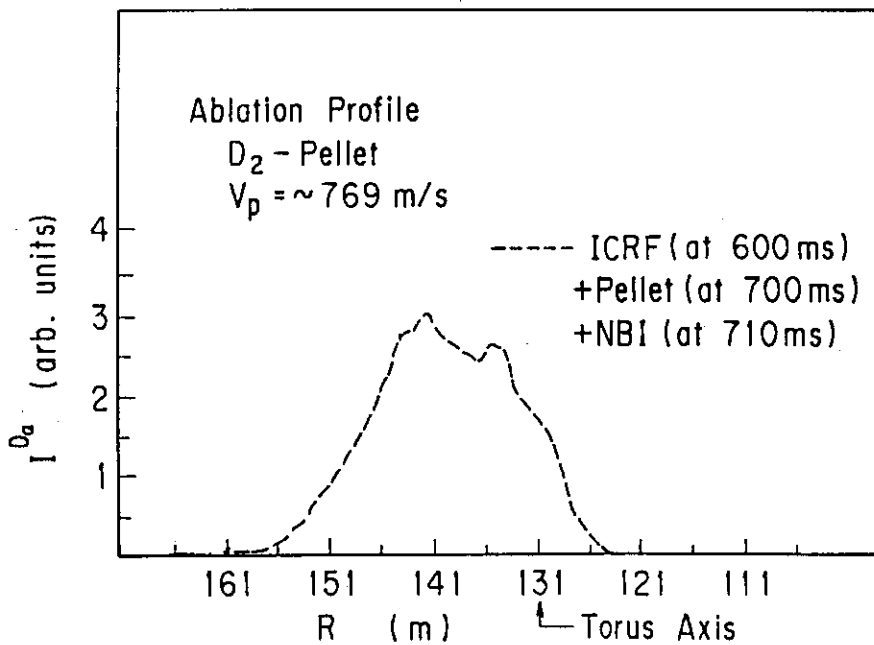


Fig. 38 Ablation profile of D<sub>2</sub>-pellet on the discharges in Fig. 36.

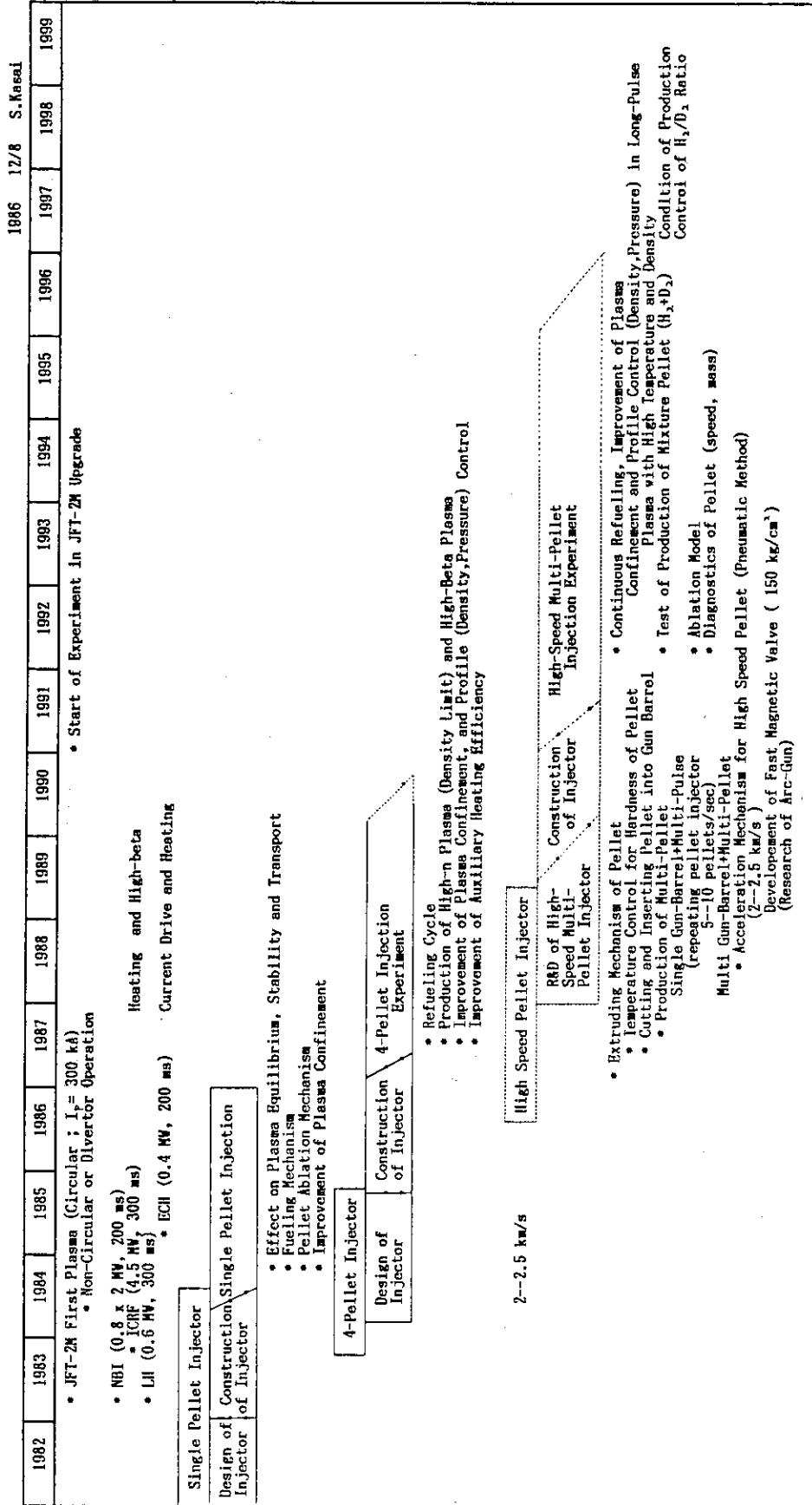


Fig. 39 Time schedule of development of the pellet injector and pellet injection experiment in Nuclear Fusion Research.

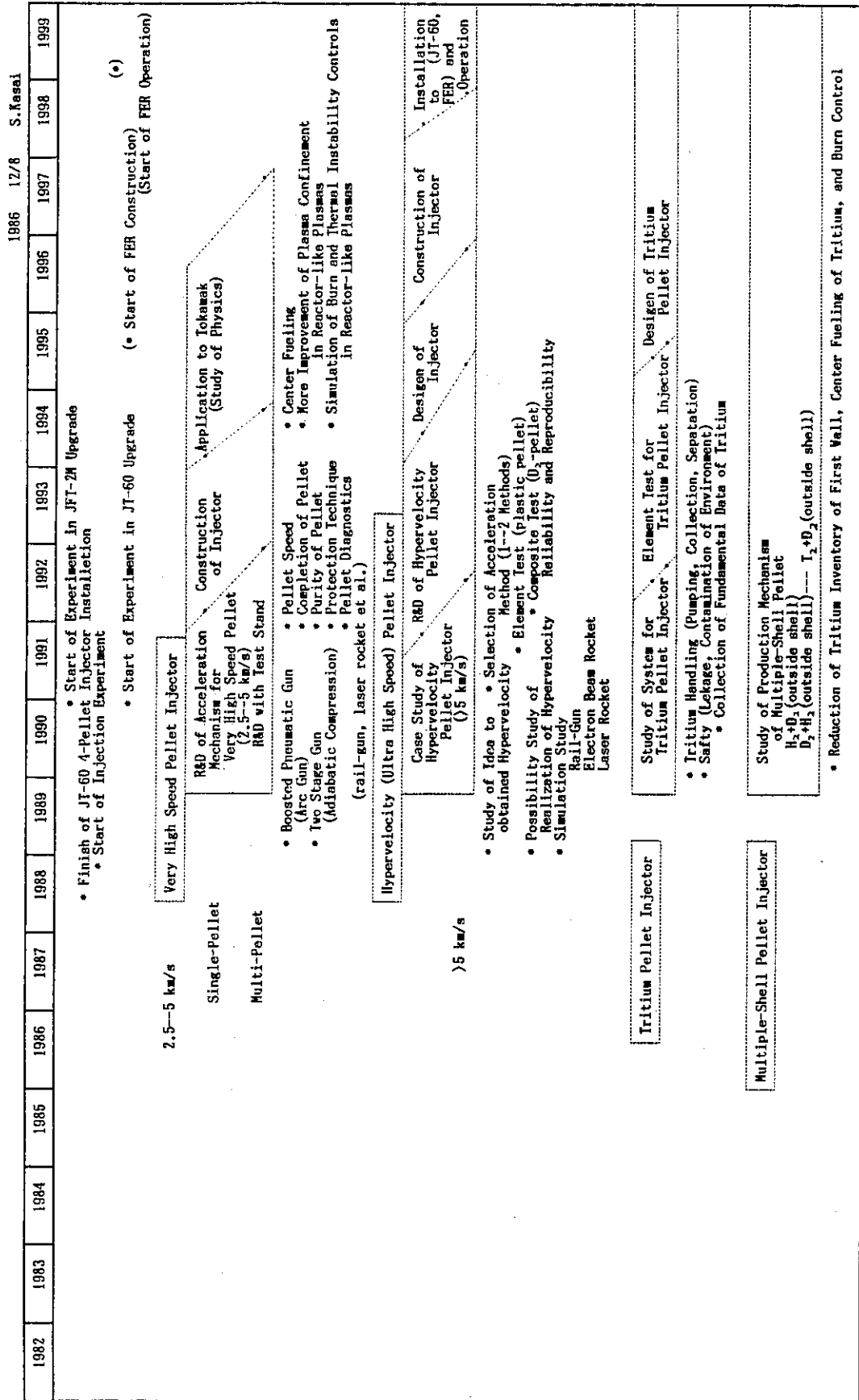


Fig. 39 Time schedule of development of the pellet injector and pellet injection experiment in Nuclear Fusion Research.