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編集兼発行 日本原子力研究所 印 刷 いばらき印刷(株) Uncertainties in Evaluated Total Cross-Section Data for 14 Nuclides Contained in JENDL-3.2

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Variances and covariances of total cross sections have been estimated for 14 nuclides contained in JENDL-3.2. Least-squares analyses using the GMA code were performed to obtain them. Information on the uncertainties of those measurements, which the JENDL-3.2 evaluation was based on, was derived from the associated references and fed into the GMA code system. The results obtained from the present analysis are illustrated.

Keywords: JENDL-3.2, Variance, Covariance, Total Cross Section, Least-squares Analysis

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JENDL-3.2に収納されている14核種の全断面積の誤差

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評価済核データライブラリーJENDL-3. 2 に収納されている14核種について、その全断面積の分散及び共分散の推定を行った。推定には、最小自乗法に基づく計算コードGMAを用いた。 JENDL-3. 2 の評価の際に用いられた個々の実験値の誤差情報は関連する文献より求められ、 GMAコードに入力された。今回の計算結果は図で示される。

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1. Introduction

Variances and covariances of evaluated data are required to estimate uncertainties in reactor calculations from the viewpoint of reliability, safety and economics. The recent evaluated data libraries such as ENDF/B-VI¹⁾ and JEF-2²⁾ contain error files to meet the requirement. However, the second version of JENDL-3 (referred to as JENDL-3.2³⁾), which was made available in 1994, did not contain error files although some users strongly requested them. Moreover, the first version of JENDL Dosimetry File⁴⁾ which was based on JENDL-3 took variance and covariance data from IRDF-85⁵⁾. Therefore, it is an urgent task to make our own error files.

One of the difficulties in making error files is due to the fact that there is no established procedure to derive variance-covariance data. In general, data evaluation is performed on the basis of experimental data, more or less, but few experimenters give detailed error information to evaluators. Thus, it is needed to carry out error analyses with limited experimental information.

A least-squares program GMA⁶⁾ was developed at ANL to derive variance and covariance data from a set of experiments with minimal information. With this code, the percentage of the systematic error to the cross-section value is required as input. The correlation matrix for data points in each experiment is calculated from this percentage. The procedure is very simple and appropriate for producing error files without knowing the detailed error information for each experiment. The present work was undertaken to study the applicability of the GMA code to the production of variances and covariances of evaluated data in JENDL-3.2. Estimated were the uncertainties in the total cross sections of fourteen nuclides ⁶Li, ⁷Li, ⁹Be, ¹²C, ¹⁶O, ²³Na, Ti, Cr, Fe, Ni, ²³⁵U, ²³⁸U, ²³⁹Pu and ²⁴⁰Pu, where the nuclide without a mass number stands for a natural element.

Chapter 2 describes the mathematical method used in the GMA code. Chapter 3 deals with information on errors of measurements for each nuclide. The results are graphically given in chapter 4.

2. Method

Variances and covariances of total cross sections for fourteen nuclides were estimated by using the GMA code system. A flow chart of the calculation procedure is shown in Fig. 1. Experimental data are retrieved from a database NESTOR-2⁷⁾, and the data format is converted using a utility program EXPTOGMA. Energy grids are determined by prior data. Experimental values from one data set are extrapolated to neighboring energy grids by using the shape of prior cross sections as shown in Fig. 2, and then the weighted average value is calculated at the energy grid. The cross-section error at this grid consists of a systematic error given by input and a reduced statistical error calculated from contributing data. This processing is performed by another utility DATGMA. The cross sections in JENDL-3.2 were used for the prior cross sections. A set of experimental data, Y, is defined as

$$Y = \begin{pmatrix} Y_1 \\ Y_2 \\ \vdots \\ Y_n \end{pmatrix} \tag{1}$$

,where Y_i stands for each experiment involving one or multiple data points after being averaged at the grids. In GMA, a correlation coefficient $C_{i,jk}$ of the experiment i is calculated by the formula

$$C_{i, jk} = \begin{pmatrix} \Delta \sigma_{i, j}^{sys} \cdot \Delta \sigma_{i, k}^{sys} & (j \neq k) \\ \Delta \sigma_{i, j}^{total} \cdot \Delta \sigma_{i, k}^{total} & (j \neq k) \end{pmatrix}$$

$$1.0 \qquad (j \neq k)$$
(2)

, where $\Delta \sigma^{total}_{i,j}$ and $\Delta \sigma^{sys}_{i,j}$ are total and systematic errors at the j-th grid. The total error is given by combining systematic and statistical errors, i.e.,

$$\Delta \sigma_{i,j}^{total} = \sqrt{\left(\Delta \sigma_{i,j}^{sys}\right)^2 + \left(\Delta \sigma_{i,j}^{sta}\right)^2}$$
 (3)

, where $\Delta \sigma^{sta}_{i,j}$ stands for a statistical error. A covariance matrix element $V_{i,jk}$ is given by

$$V_{i,jk} = \Delta \sigma_{i,j}^{total} \circ \Delta \sigma_{i,k}^{total} \circ C_{i,jk} . \tag{4}$$

The total covariance matrix V should have the form

$$V = \begin{pmatrix} V_1 & & 0 \\ & V_2 & & \\ & & \cdot & \\ & & & \cdot & \\ 0 & & & V_n \end{pmatrix} . \tag{5}$$

A design matrix Φ is introduced as

$$\Phi_{ij} = \begin{pmatrix} 1 & \text{if } i\text{-th datum lies in } j\text{-th grid} \\ 0 & \text{if otherwise} \end{cases}$$
 (6)

The best estimate X is obtained from the following equation:

$$X = (\boldsymbol{\Phi}^T V^{-1} \boldsymbol{\Phi})^{-1} \boldsymbol{\Phi}^T V^{-1} Y \tag{7}$$

, where T denotes the transpose. The covariance matrix M associated with the best estimate is obtained from

$$M = \gamma^2 (\mathbf{\Phi}^T V^{-1} \mathbf{\Phi})^{-1} \tag{8}$$

,where χ^2 is a measure of fitting, and is expressed by

$$\chi^{2} = \frac{(Y - X)^{T} V^{-1} (Y - X)}{d - g} \qquad (9)$$

In eq. (9), the symbols d and g stand for the dimension of V and the number of energy grids, respectively. The χ^2 value is applied to eq. (8) when it is larger than unity.

3. Determination of Systematic Errors for Each Experiment

With the GMA calculation, it is required to identify systematic errors for each

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measurement in order to produce a covariance matrix V_i . The systematic errors were taken from the references associated with the experiment as much as possible. In some cases where no error information is available, guess-values are assigned. Error information for each experiment is given as follows:

1) 6 Li for $E_n \ge 1 \text{ MeV}$

Knitter et al. 8) (1977)

 $E_n = 80 \text{ keV} - 3 \text{ MeV}$

Total error is given in the data.

Systematic error:

no. of ⁷Li atoms 0

0.5 %

sample size dead time

0.25 % 0.34 %

0.65 %

Lamaze et al.9) (1979)

 $E_n = 3 - 50 \text{ MeV}$

Statistical error is given in the data.

No information is available on systematic error.

A systematic error of 1 % is assumed.

Smith et al. 10) (1982)

 $E_n = 500 \text{ keV} - 4.7 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 2 % due to the sample.

2) 7 Li for $E_n \ge 100 \text{ keV}$

Meadows and Whalen $^{11)}$ (1970)

 $E_n = 100 \text{ keV} - 1.5 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 0.82 % due to the sample.

Foster, Jr. and Glasgow¹²⁾ (1971)

 $E_{n} = 2.3 - 15 \,\text{MeV}$

Total error is given in the data.

A systematic error of 0.5 % due to the sample.

Goulding et al. (1972)

 $E_n = 700 \text{ keV} - 30 \text{ MeV}$

Statistical error is given in the data.

No information is available on systematic error.

A systematic error of 1 % is assumed.

Lamaze et al.⁹⁾ (1979)

 $E_n = 3 - 50 \text{ MeV}$

Statistical error is given in the data.

No information is available on systematic error.

A systematic error of 1 % is assumed.

3) ${}^{9}\text{Be for } E_n \ge 830 \text{ keV}$

Schwartz et al. (1971)

 $E_n = 500 \text{ keV} - 20 \text{ MeV}$

Statistical error is given in the data.

No information is available on systematic error.

A systematic error of 1 % is assumed.

Foster, Jr. and Glasgow¹²⁾ (1971) $E_n = 2.3 - 15 \text{ MeV}$

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Total error is given in the data.

A systematic error of 0.4 % due to the sample.

Auchampaugh et al. (1979)

 $E_{p} = 1 - 14 \text{ MeV}$

Statistical error is given in the data.

Systematic error:

sample thickness background normalization

1.0 % 1.0 %

0.7%

dead time

0.5 % 1.7 %

sum

Finlay et al. (1993)

 $E_n = 5 - 600 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 0.5 % due to normalization.

Sugimoto et al. (1989)

 $E_{n} = 1 - 10 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 1.5 % is assumed.

4) 12 C for $E_n \ge 4.8 \text{ MeV}$

Lamaze et al.9) (1979)

 $E_n = 2.5 \,\mathrm{MeV} - 40 \,\mathrm{MeV}$

Statistical error is given in the data.

No information is available on systematic error.

A systematic error of 1 % is assumed.

Auchampaugh et al. 15) (1979)

 $E_n = 1.2 - 13.9 \text{ MeV}$

Statistical error is given in the data.

Systematic error:

sample thickness background

0.7 %

normalization

1.0 % 1.0 %

dead time

sum

0.5 % 1.7 %

Cieriacks et al. (1980)

 $E_{n} = 3 - 32 \text{ MeV}$

Total error is given in the data.

A systematic error of 1 % is assumed.

Finlay et al. (1993)

 $E_{\rm n} = 5 - 600 \, \text{MeV}$

Statistical error is given in the data.

A systematic error of $0.5\,\%$ is given by the authors.

5) 16 O for $E_n \ge 3$ MeV

Cierjacks et al. (1980)

 $E_n = 3 - 32 \text{ MeV}$

Total error is given in the data.

A systematic error of 0.5 % is assumed.

Larson¹⁹⁾ (1980)

 $E_{r} = 2 - 71 \,\text{MeV}$

Statistical error is given in the data.

A systematic error of 3 % is deduced.

6) 23 Na for $E_n \ge 350 \text{ keV}$

Cieriacks et al. 20) (1969)

 $E_{r} = 290 \text{ keV} - 32 \text{ MeV}$

Total error is given in the data.

A systematic error of 2 % is deduced.

Langsford et al.²¹⁾ (1965)

 $E_{\rm p} = 180 \; {\rm keV} - 120 \; {\rm MeV}$

Total error is given in the data.

A systematic error of 3 % is assumed.

Stoler et al. (1971)

 $E_n = 650 \text{ keV} - 45 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 1.5 % due to background.

Larson et al.²³⁾ (1976)

 $E_n = 32 \text{ keV} - 37 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 3 % is deduced.

7) Ti for $E_n \ge 100 \text{ keV}$

Foster, Jr. and Glasgow¹²⁾ (1971)

 $E_{n} = 2.3 - 15 \,\text{MeV}$

Total error is given in the data.

A systematic error of 0.5 % due to sample thickness.

Cabe and $Cance^{24}$ (1973)

 $E_n = 150 \text{ keV} - 4 \text{ MeV}$

Total error is given in the data.

A systematic error of 3.5 % is deduced.

Schwartz et al. 25 (1974)

 $E_n = 500 \text{ keV} - 4 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 3 % due to normalization.

Barnard et al.²⁶⁾ (1974)

 $E_n = 100 \text{ keV} - 1.5 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 3 % is deduced.

8) Cr for $E_n \ge 300 \text{ keV}$

Cieriacks et al. 27 (1968)

 $E_{p} = 500 \text{ keV} - 32 \text{ MeV}$

Total error of 3 % is given the data.

A systematic error of 2.8 % is deduced.

Foster, Jr. and Glasgow¹²⁾ (1971) $E_n = 2.3 - 1.5 \text{ MeV}$

Total error is given in the data. A systematic error of 0.5 % is deduced.

Perey et al. 28) (1973)

 $E_n = 180 \text{ keV} - 30 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 4 % is assumed.

Larson¹⁹⁾ (1980)

 $E_n = 2 - 81 \text{ MeV}$

No error information is available.

The error given is assumed to be statistical.

A systematic error of 4 % is assumed.

9) Fe for $E_n \ge 250 \text{ keV}$

Carlson and Cerbone²⁹⁾ (1970)

 $E_n = 460 \text{ keV} - 9 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 4 % is assumed.

Cieriacks et al. 30 (1968)

 $E_n = 500 \text{ keV} - 32 \text{ MeV}$

Total error of 3 % is given in the data.

A systematic error of 2.8 % is assumed.

Perev et al.³¹⁾ (1972)

 $E_n = 190 \text{ keV} - 49 \text{ MeV}$

Statistical error is given in the data.

Systematic error:

impurities in sample 0.14 %

sample thickness

0.5 %

dead time

2.0 %

flux normalization

1.0 %

background

0.1%

sum

2.3 %

Pattenden et al. (1973)

 $E_n = 200 \text{ eV} - 1.1 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 3 % is assumed.

Schwartz et al.²⁵⁾ (1974)

 $E_n = 500 \text{ keV} - 15 \text{ MeV}$

Total error is given in the data.

A systematic error of 1 % is assumed.

10) Ni for $E_n \ge 557 \text{ keV}$

Larson et al. (1983)

 $E_n = 2.2 \text{ keV} - 20 \text{ MeV}$

Total error is given in the data.

A systematic error of 0.61 % is deduced.

Larson¹⁹⁾ (1980)

 $E_{p} = 2 - 80 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 5 % is assumed.

11) 235 U for $E_n \ge 30 \text{ keV}$

Uttley et al. (1966)

 $E_n = 150 \text{ eV} - 950 \text{ keV}$

Total error is given in the data.

A systematic error of 0.5 % is assumed.

Böckhoff et al. (1972)

 $E_n = 5.8 - 270 \text{ keV}$

No error is given in the data.

A systematic error of 3 % and a statistical error of 3 % are assumed.

Schwartz et al. (1974)

 $E_n = 500 \text{ keV} - 15 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 1.1 % is deduced.

Green and Mitchell³⁷⁾ (1973) $E_n = 500 \text{ keV} - 10 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 3 % is assumed.

Foster, Jr. and Glasgow¹²⁾ (1971) $E_n = 2.3 - 15 \text{ MeV}$ Total error is given in the data.

A systematic error of 0.5 % is deduced.

Poenitz et al.³⁸⁾ (1981)

 $E_n = 48 \text{ keV} - 4.8 \text{ MeV}$

Total and statistical errors are given in the data.

A systematic error of 0.6 % is deduced.

Poenitz and Whalen³⁹⁾ (1983)

 $E_n = 1.8 - 20 \text{ MeV}$

Total error is given in the data.

Systematic error:

sample thickness

0.3 % 0.3 %

background

0.5 %

dead time

0.5 %

sum

0.66 %

12) 238 U for $E_n \ge 150 \text{ keV}$

Whalen and Smith⁴⁰⁾ (1971)

 $E_n = 100 \text{ keV} - 1.5 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 3 % is assumed.

Poenitz et al. (1981)

 $E_n = 48 \text{ keV} - 4.8 \text{ MeV}$

Total and statistical errors are given in the data.

A systematic error of 0.4 % is deduced.

Tsubone et al.41) (1984)

 $E_n = 23 - 930 \text{ keV}$

Total error is given in the data.

A systematic error of 2.2 % is deduced.

Foster, Jr. and Glasgow¹²⁾ (1971) $E_n = 2.3 - 15 \text{ MeV}$

Total error is given in the data.

A systematic error of 0.5 % due to sample thickness.

Bratenahl et al. (1958)

 $E_{n} = 7 - 14 \text{ MeV}$

Total error is given in the data.

A systematic error of $0.5\,\%$ is given by the authors.

Peterson et al. (1960)

 $E_n = 17 - 29 \text{ MeV}$

Total error is given in the data.

A systematic error of 0.5% is given by the authors.

13) 239 Pu for $E_n \ge 7 \text{ MeV}$

Schwartz et al. (1974)

 $E_n = 500 \text{ keV} - 15 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 1.1 % is deduced.

Foster, Jr. and Glasgow¹²⁾ (1971) $E_n = 2.3 - 15 \text{ MeV}$

Total error is given in the data.

A systematic error of 0.5 % due to sample thickness.

Nadolny et al. 44) (1973)

 $E_n = 500 \text{ keV} - 31 \text{ MeV}$

Statistical error is given in the data.

A systematic error of 1 % is deduced.

Poenitz et al.³⁹⁾ (1983)

 $E_{n} = 1.8 - 21 \text{ MeV}$

Total error is given in the data.

Systematic error:

sample thickness

0.75 %

background

0.3 %

dead time

0.3 %

sum

0.93 %

14) 240 Pu for En $\geq 40 \text{ keV}$

Smith et al. 45) (1972)

 $E_{_{\rm R}} = 120~{\rm keV} - 1.5~{\rm MeV}$

Total and systematic errors are given in the data.

A systematic error of 3 % is deduced.

Poenitz et al.³⁸⁾ (1981)

 $E_n = 48 \text{ keV} - 4.8 \text{ MeV}$

Total error is given in the data.

A systematic error of 1.2 % is deduced.

Poenitz and Whalen³⁹⁾ (1983)

 $E_n = 1.8 - 21 \text{ MeV}$

Total error is given in the data.

Systematic error:

sample thickness

0.75%

background

 $0.3\ \%$

dead time

0.45 %

sum

0.93 %

4. Results and Discussion

The calculated results are shown in Figs. 3-23, where the best-fit curves (solid lines) are given together with standard deviations (dashed lines). It was easy to give a fit to smooth cross sections of the nuclides such as ^{6,7}Li, ⁹Be, ^{235,238}U and ^{239,240}Pu in the energy region considered. As seen in Fig. 2, original experimental data are moved to a grid in parallel with a prior cross-section curve, and then an average value is calculated at the grid. Thus, fine grids should be set when resonance-shaped cross sections are involved.

Enhancement factors which are defined by the square root of eq. (9) are given in Table 1. The values for Ti, Cr and Fe are fairly large, since the measurements for each nuclide are discrepant with one another in the resonance energy region below several MeV. It seems unreasonable to apply these large values to obtain standard deviations in the high energy region where no resonance structure is seen. In these cases, the enhancement factors were recalculated in the resonance and non-resonance energy regions separately, and they were used to obtain the standard deviations in each region.

Figures 24-37 show the correlation matrices normalized to 1000. A strong correlation

Poenitz et al.³⁹⁾ (1983)

 $E_n = 1.8 - 21 \text{ MeV}$

Total error is given in the data.

Systematic error:

sample thickness

0.75 %

background

0.3 %

dead time

0.3 %

sum

0.93 %

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Figures 24-37 show the correlation matrices normalized to 1000. A strong correlation

is seen in the figures except for heavy nuclides, since one or two measurements with a huge number of data points are emphasized in the calculations.

5. Conclusion

Error analyses were performed for the total cross section of fourteen nuclides contained in JENDL-3.2. Variances and covariances were determined using the GMA code system, and the results were illustrated. The method used in the present work is very simple, and can be applied to produce the error files of JENDL-3.2.

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References

- 1) Rose P.F.: "ENDF/B-VI Summary Documentation", BNL-NCS-17541, 4th Ed. (1991).
- 2) Lemmel H.D. and Schwere O.: "JEF-2 The Evaluated Nuclear Data Library of the NEA Data Bank", IAEA-NDS-120 Rev.2 (1993).
- 3) Nakagawa T. et al.: Submitted to J. Nucl. Sci. Technol.
- 4) Nakazawa M., Kobayashi K., Iwasaki S., Iguchi T., Sakurai K., Ikeda Y. and Nakagawa T.: "IENDL Dosimetry File", JAERI 1325 (1992).
- 5) Cullen D.E., Kocherov N. and McLaughlin P.M.: "The International Reactor Dosimetry File (IRDF-82)", IAEA-NDS-41/R, rev. 0 (1982). IRDF-85 is a modified version with additional cross section data.
- 6) Poenitz W.P.: "Data Interpretation, Objective Evaluation Procedures and Mathematical Techniques for the Evaluation of Energy-Dependent Ratio, Shape and Cross Section Data", Proc. Conf. Nuclear Data Evaluation Methods and Procedures, BNL-NCS-51363, p.249 (1981).
- 7) Nakagawa T.: "Neutron Data Storage and Retrieval System", JAERI Nuclear Data Center.
- 8) Knitter H.H., Budtz-Jørgensen C., Mailly N. and Vogt R.: "Neutron Total and Elastic Scattering Cross sections of ⁶Li in the Energy Range from 0.1 to 30 MeV", EUR-5726e (1977).
- 9) Lamaze G.P., Kellie J.D. and Schwartz R.B.: Proc. Int. Conf. Nuclear Cross Sections for Technology, Knoxville, 1979, p.48 (1980).
- 10) Smith A.B., Guenther P.T. and Whalen J.F.: Nucl. Phys., <u>A373</u>, 305 (1982).
- 11) Meadows J.W. and Whalen J.F.: Nucl. Sci. Eng., <u>41</u>, 351 (1970).
- 12) Foster, Jr. D.G. and Glasgow D.W.: Phys. Rev., <u>C3</u>, 576 (1971).
- 13) Goulding C.A., Clement J.M. and Stoler P.: Taken from EXFOR (1972).
- 14) Schwartz R.B., Schrack R.A. and Heaton H.T.: Bull. Am. Phys. Soc., <u>16</u>, 495 (1971).
- 15) Auchampaugh G.F., Plattard S. and Hill N.W.: Nucl. Sci. Eng., <u>69</u>, 30 (1979).
- 16) Finlay R.W., Abfalterer W.P., Fink G., Montei E., Adami T., Lisowski P.W., Morgan G.L. and Haight R.C.: Phys. Rev., <u>C47</u>, 237 (1993).

- 17) Sugimoto M., Guenther P., Lynn J., Smith A. and Whalen J.: Nucl. Sci. Eng., 103, 37 (1989).
- 18) Cierjacks S., Hinterberger F., Schmalz G., Erbe D., Rossen P.V. and Leugers B.: Nucl. Instrum. Meth., <u>169</u>, 185 (1980).
- 19) Larson D.C.: "ORELA Measurements to Meet Fusion Energy Neutron Cross Section Needs", Proc. Symp. Neutron Cross-Sections from 10 to 50 MeV", BNL-NCS-51245 p.277 (1980).
- 20) Cierjacks S., Forti P., Kopsch, Kropp L., Nebe J. and Unseld H.: "High Resolution Total Neutron Cross Sections for Na, Cl, K, V, Mn and Co between 0.5 and 30 MeV", KfK-1000 Suppl. 2 (1969).
- 21) Langsford A., Bowen P.H., Cox G.C., Firk F.W.K., McConnell D.B., Rose B. and Saltmarsh M.: Proc. Int. Conf. The Study of Nuclear Structure with Neutrons, Antwerp, 1965, p. 529 (1965).
- 22) Stoler P., Yergin P.F., Clement J.C., Goulding C.G. and Fairchild R.: "The MeV Total Neutron Cross Section Program R.P.I.", Proc. Third Conf. Neutron Cross-Sections and Technology, Knoxville, 1971, p.311, Vol. 1 (1971).
- 23) Larson D.C., Harvey J.A. and Hill N.W.: "Measurement of the Neutron Total Cross Section of Sodium from 32 keV to 37 MeV", ORNL/TM-5614 (1976).
- Cabe J. and Cance M.: "Measurements of the Neutron Total Cross-Sections of Be, ¹¹B, C, Al, Si, S, Ti, V, Ni, ²³⁵U, ²³⁸U, ²³⁹Pu between 100 keV and 6 MeV", CEA-R-4524 (1973).
- 25) Schwartz R.B., Schrack R.A. and Heaton H.T.: "MeV Total Neutron Cross Sections", NBS-MONO-138 (1974).
- Barnard E., de Villiers J., Reitmann D., Moldauer P., Smith A.B. and Whalen J.: Nucl. Phys. 229, 189 (1974).
- 27) Cierjacks S., Forti P., Kopsch, Kropp L., Nebe J. and Unseld H.: "High Resolution Total Neutron Cross Sections for Si, Cr and Ni between 0.5 and 30 MeV", KfK-1000 Suppl. 1 (1968).
- 28) Perey F.G., Love T.A. and Kinney W.E.: Taken from EXFOR (1973).

- 29) Carlson A.D. and Cerbone R.J.: Nucl. Sci. Eng., <u>42</u>, 28 (1970).
- 30) Cierjacks S., Forti P., Kopsch, Kropp L., Nebe J. and Unseld H.: "High Resolution Total Neutron Cross-Sections between 0.5 and 30 MeV", KfK-1000 (1969).
- 31) Perey F.G., Love T.A. and Kinney W.E.: "A Test of Neutron Total Cross-Section Evaluations from 0.2 to 20 MeV for C, O, Al, Si, Ca, Fe and SiO₂", ORNL-4823 (1972).
- Pattenden N.J., Belcher I.T., Blair I.M., Bowen P.H., Cox G.C., Dolley P.E. and Mcmurray W.R.: "A Preliminary Measurement of the Neutron Total Cross-Section of Iron from 24 keV to 1000 keV Using the Harwell Synchrocyclotron", AERE-R-7425 (1973).
- 33) Larson D.C., Larson N.M., Harvey J.A., Hill N.W. and Johnson C.H.: "Application of New Techniques to ORELA Neutron Transmission Measurements and their Uncertainty Analysis - The Case of Natural Nickel from 2 keV to 20 MeV", ORNL/TM-8203 (1983).
- 34) Uttley C.A., Newstead C.M. and Diment K.M.: "Neutron Strength Function Measurements in the Medium and Heavy Nuclei", Proc. First IAEA Conf. Nuclear Data for Reactors, Paris, 1966, p. 165, Vol.1 (1967).
- 35) Böckhoff K.H., Dufrasne A., Rohr G. and Weigmann H.: J. Nucl. Energ., 26, 91 (1972).
- 36) Schwartz R.B., Schrack R.A. and Heaton H.T.: Nucl. Sci. Eng., <u>54</u>, 322 (1974).
- 37) Green L. and Mitchell J.A.: "Total Cross Section Measurements with a ²⁵²Cf Time of Flight Spectrometer", WAPD-TM-1073 (1973).
- 38) Poenitz W.P., Whalen J.F. and Smith A.B.: Nucl. Sci. Eng., <u>78</u>, 333 (1981).
- 39) Poenitz W.P. and Whalen J.F.: "Neutron Total Cross Section Measurements in the Energy Region from 47 keV to 20 MeV", ANL-NDM-80 (1983).
- 40) Whalen J.F. and Smith A.B.: "Uranium Total Cross Section", ANL-7710 (1971).
- 41) Tsubone I., Nakajima Y., Furuta Y. and Kanda Y.: Nucl. Sci. Eng., <u>88</u>, 579 (1984).
- 42) Bratenahl A., Peterson J.M. and Stoering J.P.: Phys. Rev., 110, 927 (1958).
- 43) Peterson J.M., Bratenahl A. and Stoering J.P.: Phys. Rev., <u>120</u>, 521 (1960).
- 44) Nadolny K.A., Green F.L. and Stoler P.: "Total Neutron Cross Section of ²³⁹Pu", USNDC-9, p.170 (1973).

45) Smith A.B., Whalen J.F. and Lambropoulos P.: Nucl. Sci. Eng., 47, 19 (1972).

Table 1 Enhancement factor F^{\bullet}

Nuclide	F	Nuclide	F	Nuclide	F
6 Li	2.33	²³ Na	2.83	^{235}U	2.39
⁷ Li	3.39	Ti	13.29, 13.29 ¹⁾ , 0.66 ²⁾	^{238}U	1.51
⁹ Be	5.20	Cr	17.82, 20.80 ³⁾ , 4.72 ⁴⁾	²³⁹ Pu	1.29
¹² C	4.76	Fe	16.97, 18.94 ⁵⁾ , 3.64 ⁶⁾	240 Pu	1.34
¹⁶ O	6.53	Ni	0.21		

^{*)} Defined by the square root of eq. (9).

¹⁾ Calculated below 5 MeV.

²⁾ Calculated above 5 MeV.

³⁾ Calculated below 4.5 MeV.

⁴⁾ Calculated above 4.5 MeV.

⁵⁾ Calculated below 3 MeV.

⁶⁾ Calculated above 3 MeV.

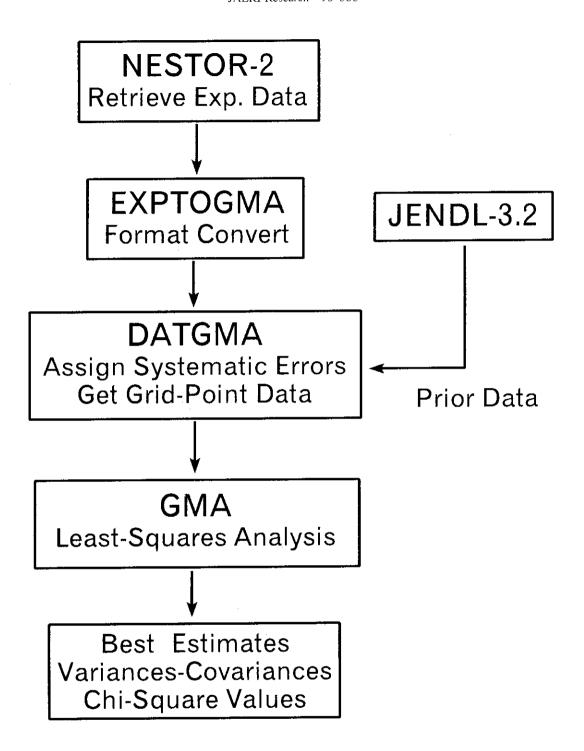


Fig. 1 Flow chart of the GMA code system.

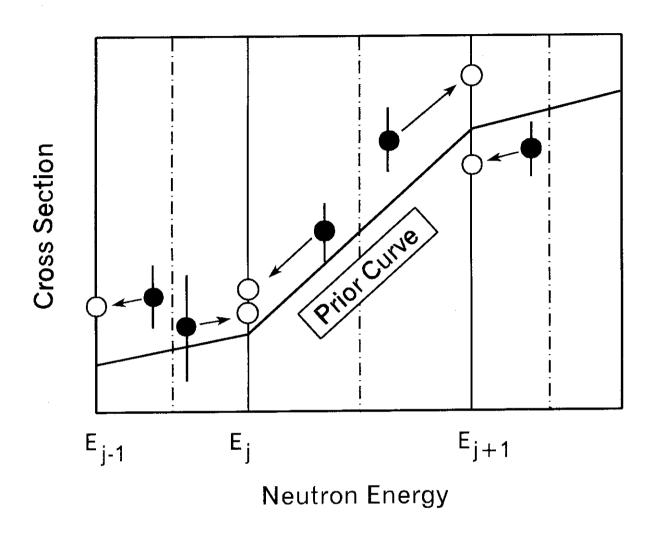


Fig. 2 Procedure to obtain experimental values at a grid energy.
Original data are moved to a grid energy in parallel with a prior curve.

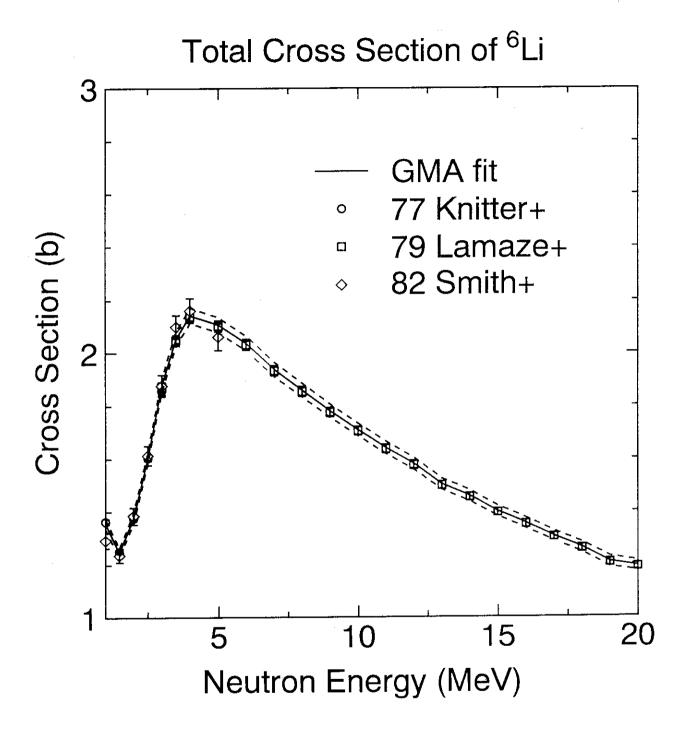


Fig. 3 Total cross section of ⁶Li.

The solid line is the best estimate, and the dashed lines standard deviations.

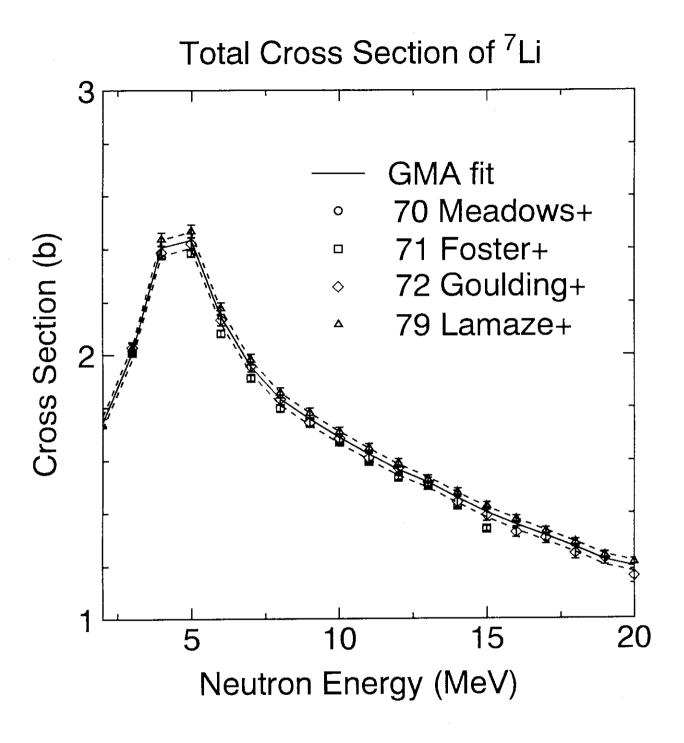


Fig. 4 Total cross section of ⁷Li.

Total Cross Section of ⁹Be

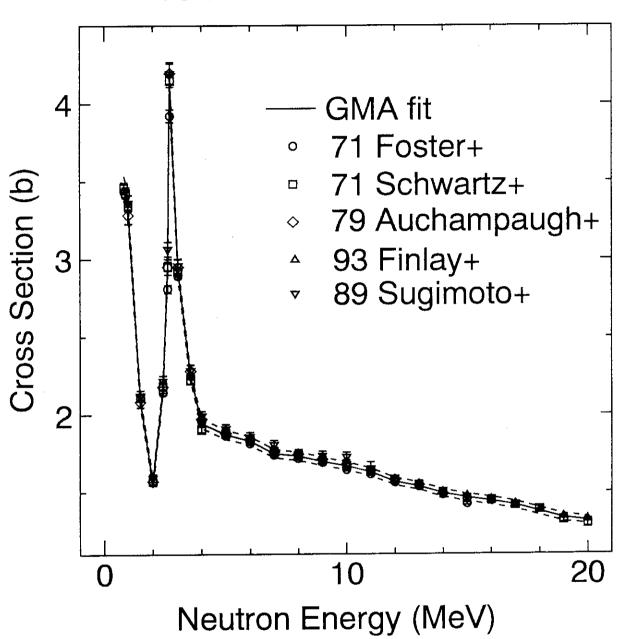


Fig. 5 Total cross section of ⁹Be.

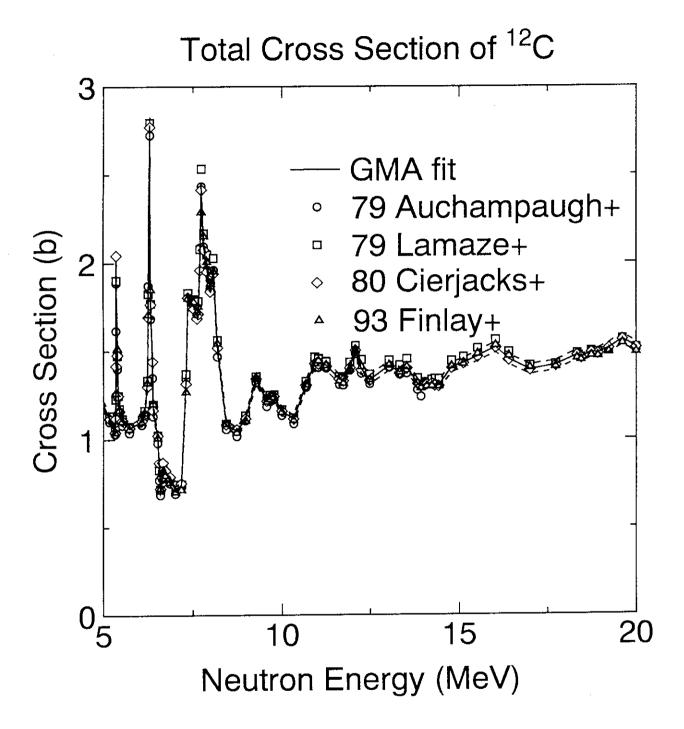


Fig. 6 Total cross section of ¹²C.

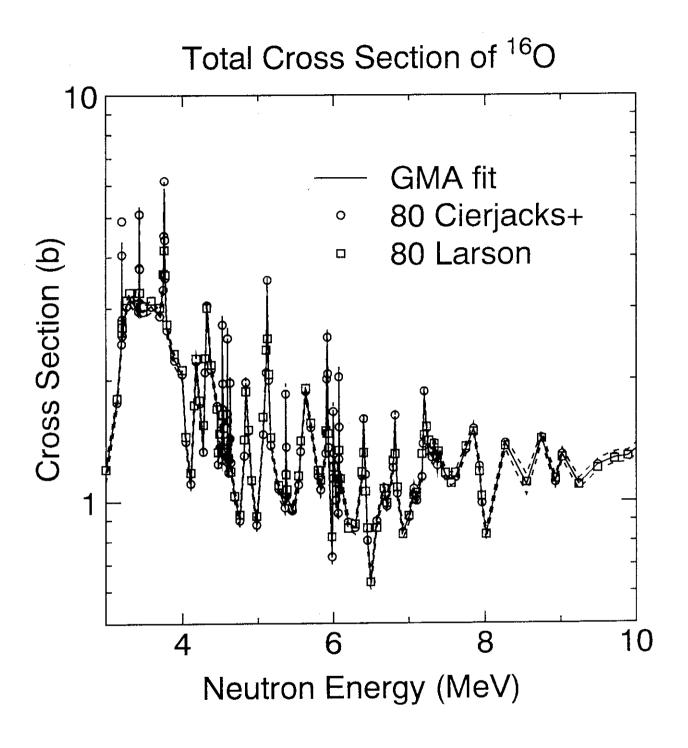


Fig. 7 Total cross section of ¹⁶O below 10 MeV.

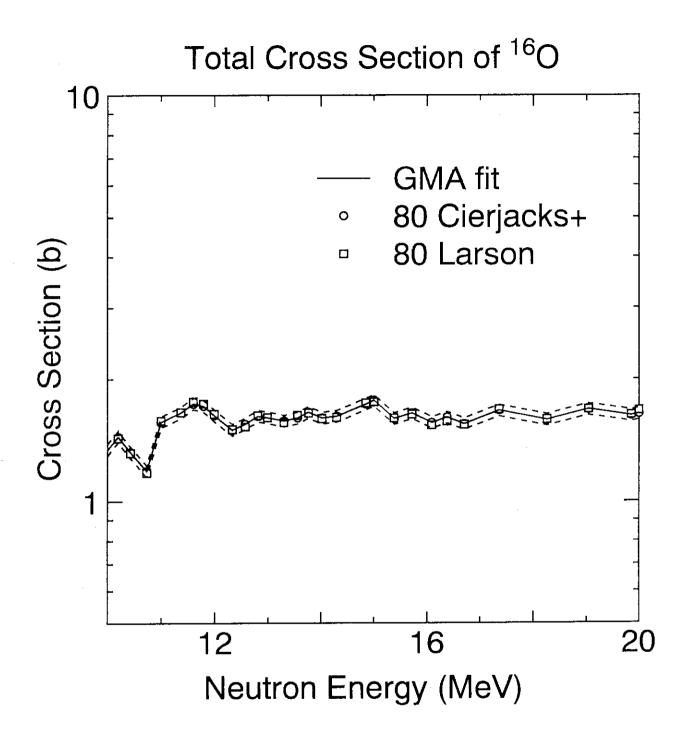


Fig. 8 Total cross section of ¹⁶O above 10 MeV.

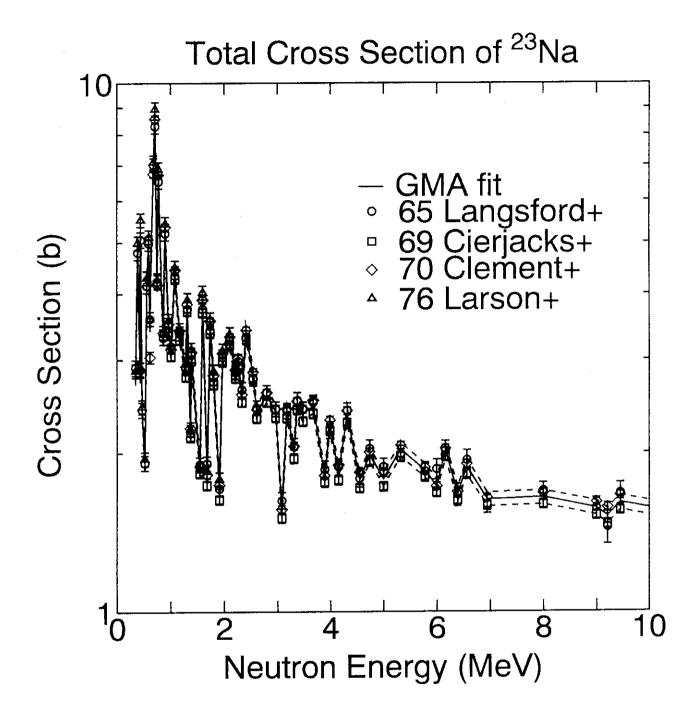


Fig. 9 Total cross section of ²³Na below 10 MeV.

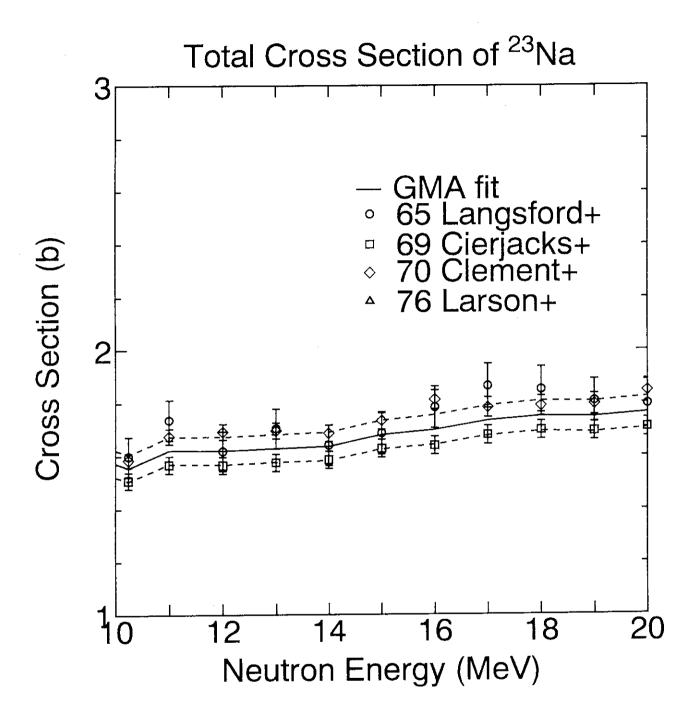


Fig. 10 Total cross section of ²³Na above 10 MeV.

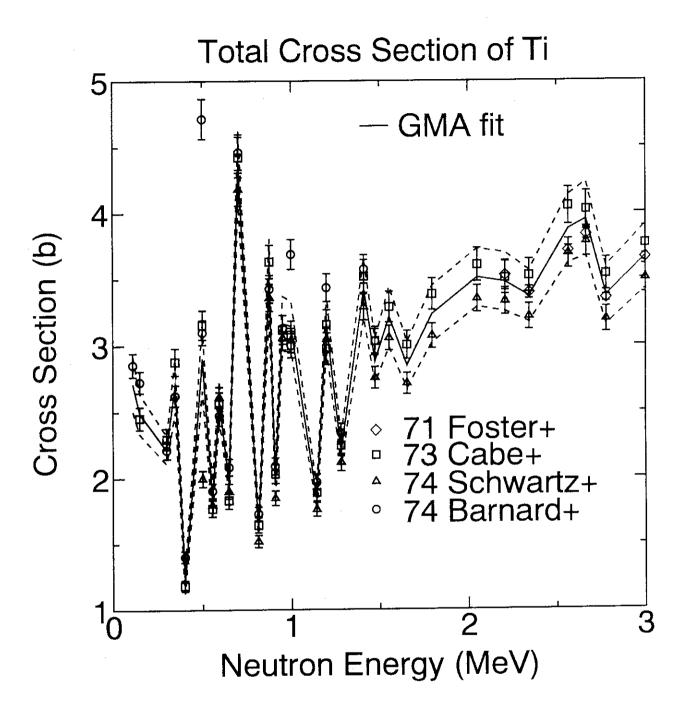


Fig. 11 Total cross section of Ti below 3 MeV.

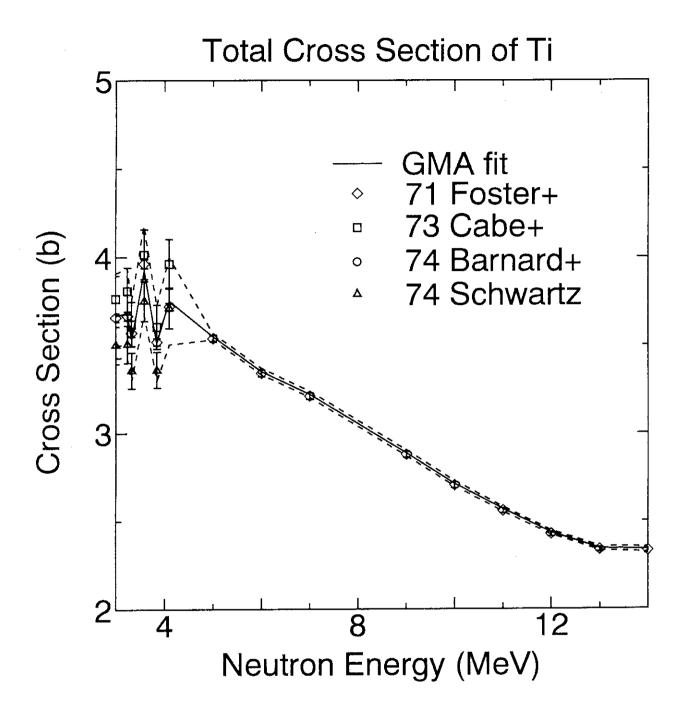


Fig. 12 Total cross section of Ti above 3 MeV.

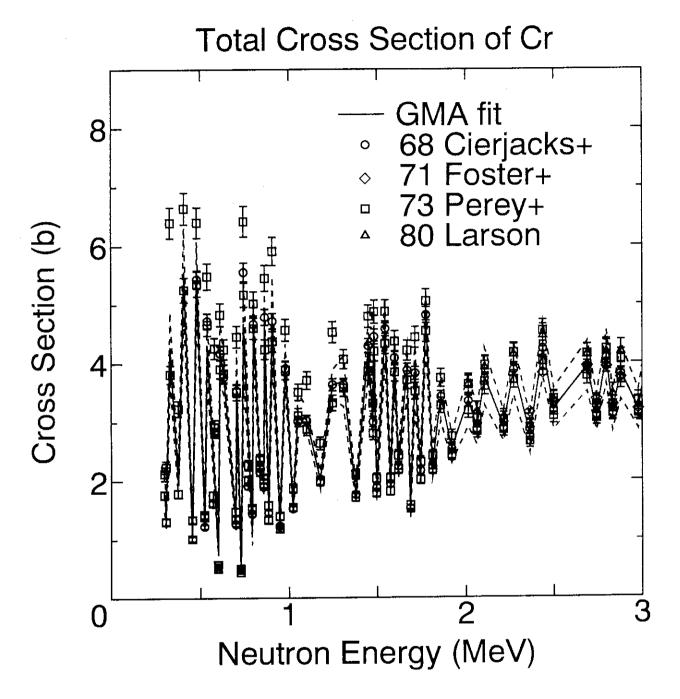


Fig. 13 Total cross section of Cr below 3 MeV.

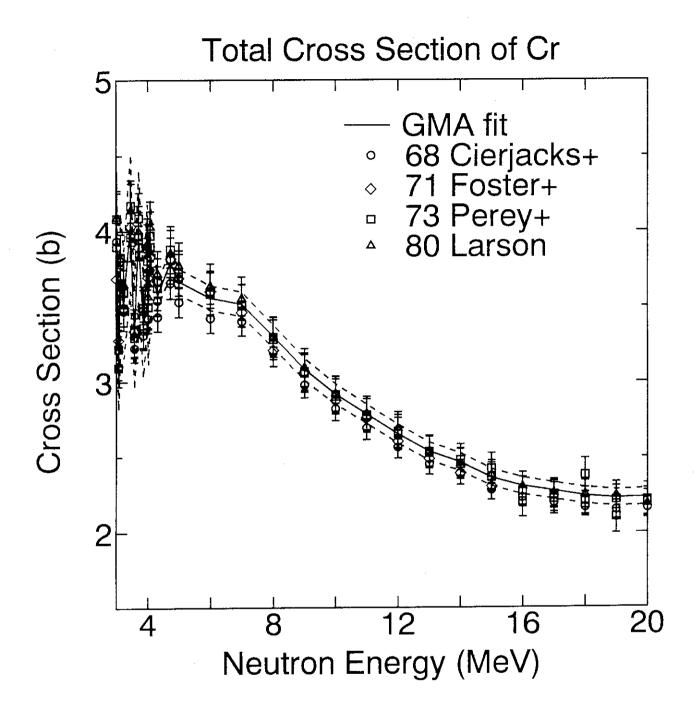


Fig. 14 Total cross section of Cr above 3 MeV.

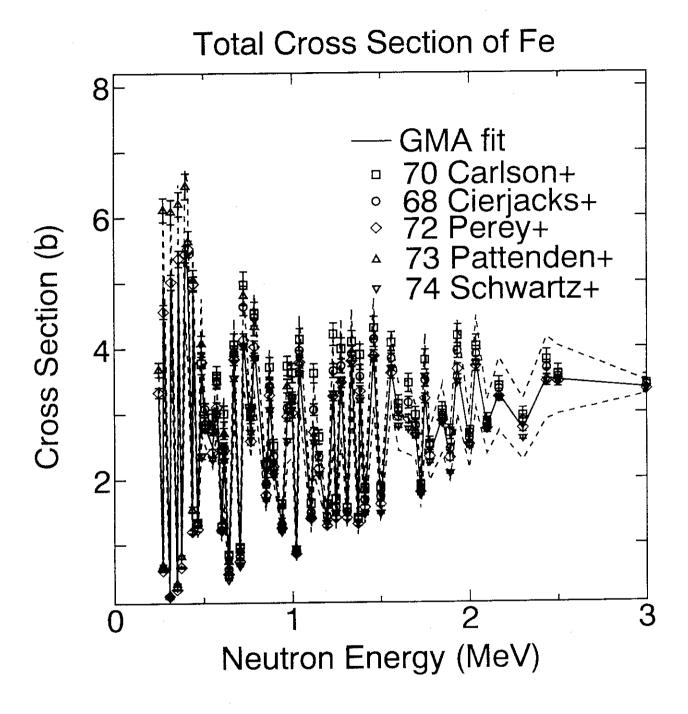


Fig. 15 Total cross section of Fe below 3 MeV.

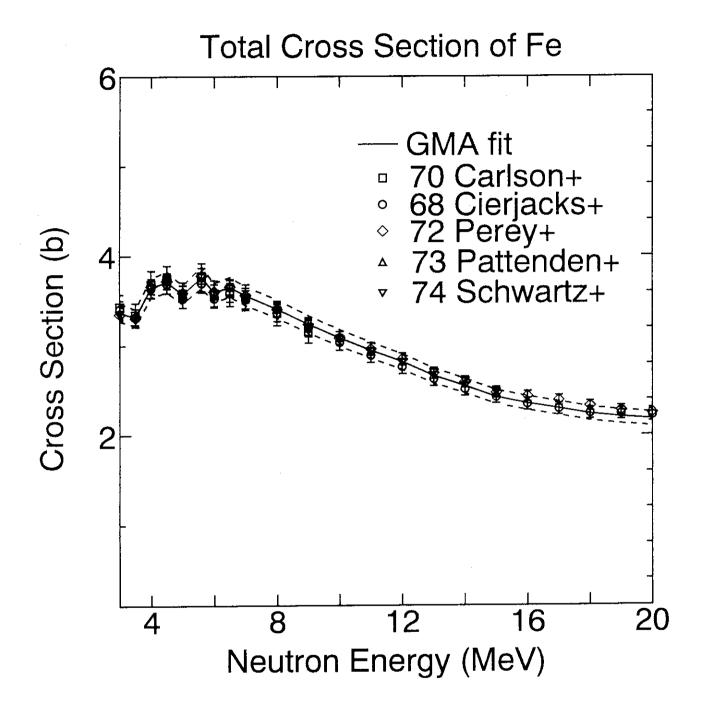


Fig. 16 Total cross section of Fe above 3 MeV.

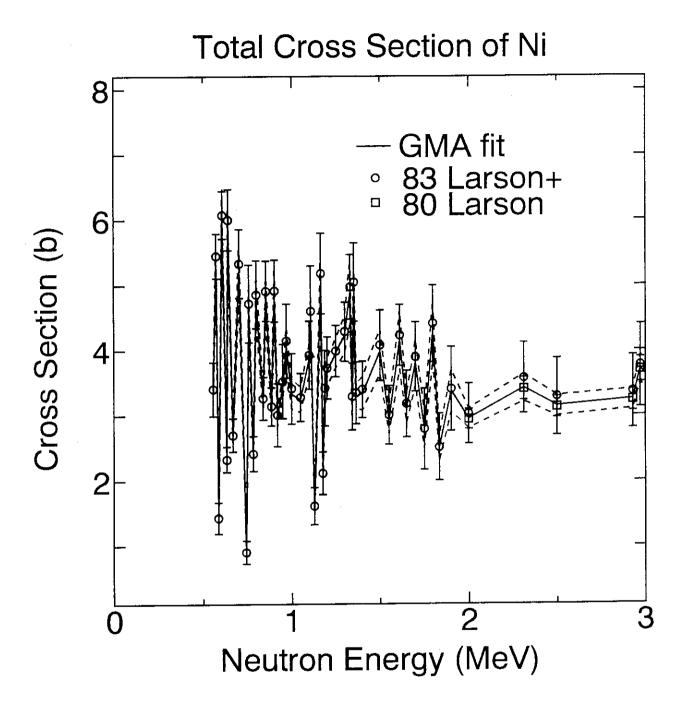


Fig. 17 Total cross section of Ni below 3 MeV.

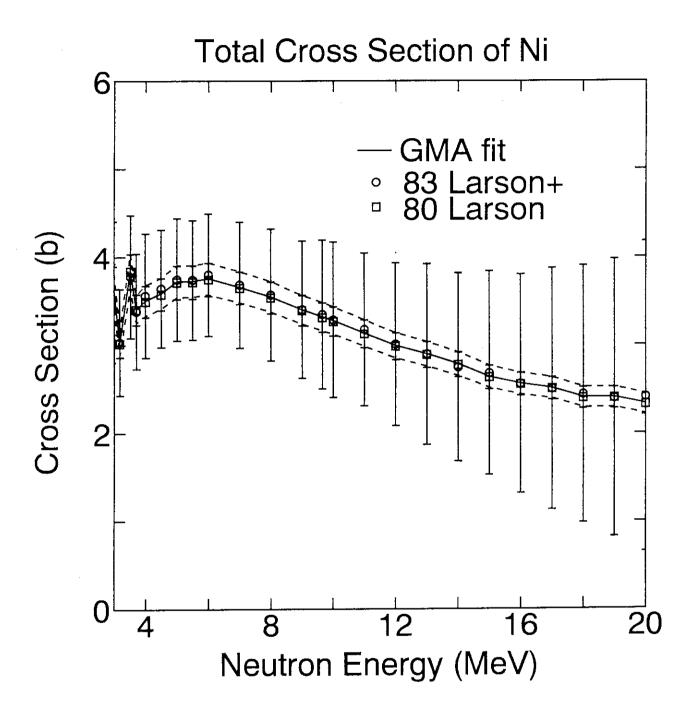


Fig. 18 Total cross section of Ni above 3 MeV.

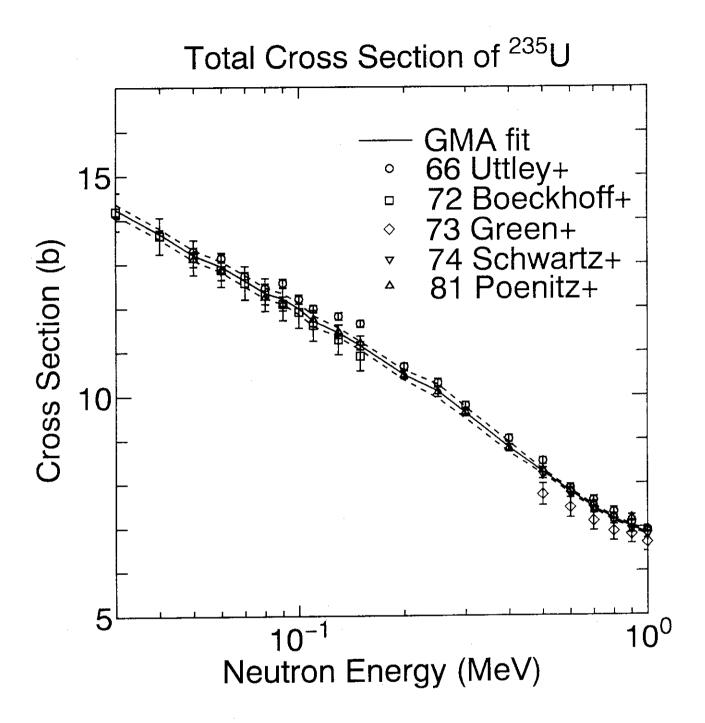


Fig. 19 Total cross section of ²³⁵U below 1 MeV.

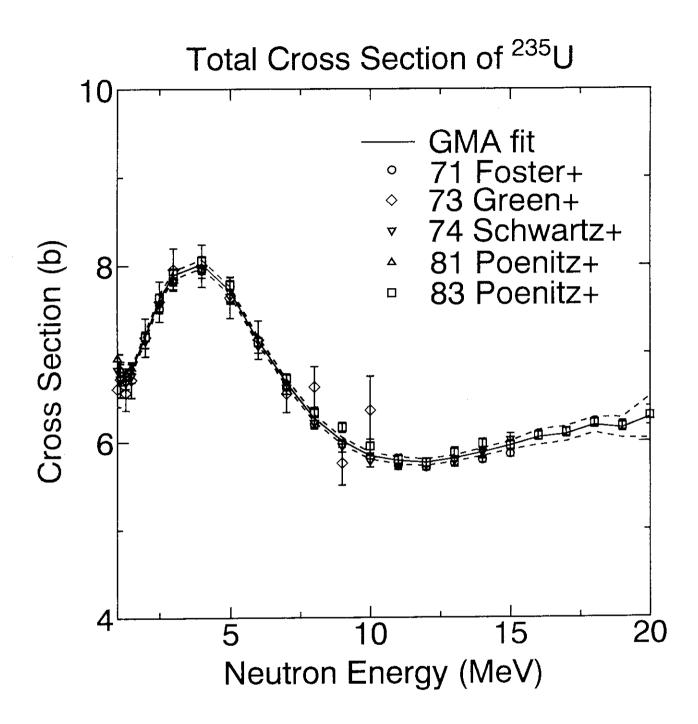


Fig. 20 Total cross section of ²³⁵U above 1 MeV.

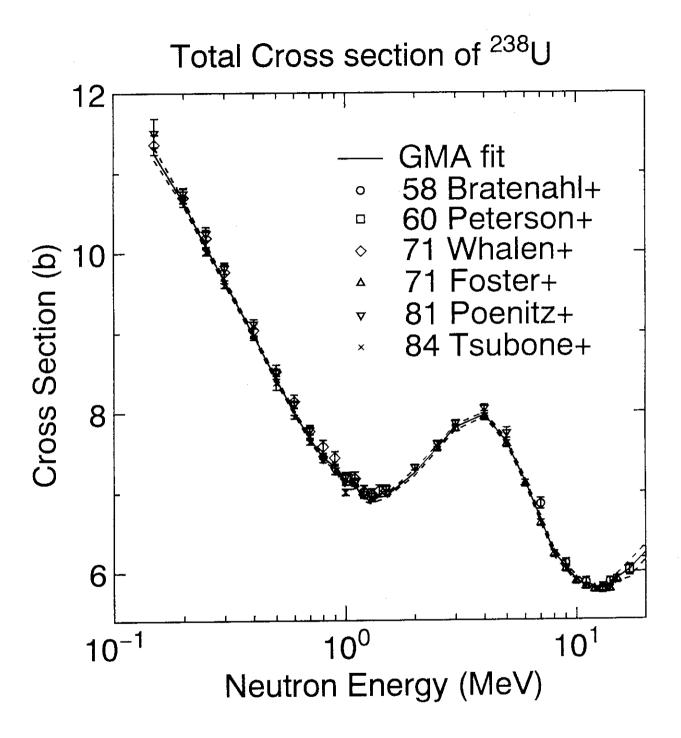


Fig. 21 Total cross section of ²³⁸U.

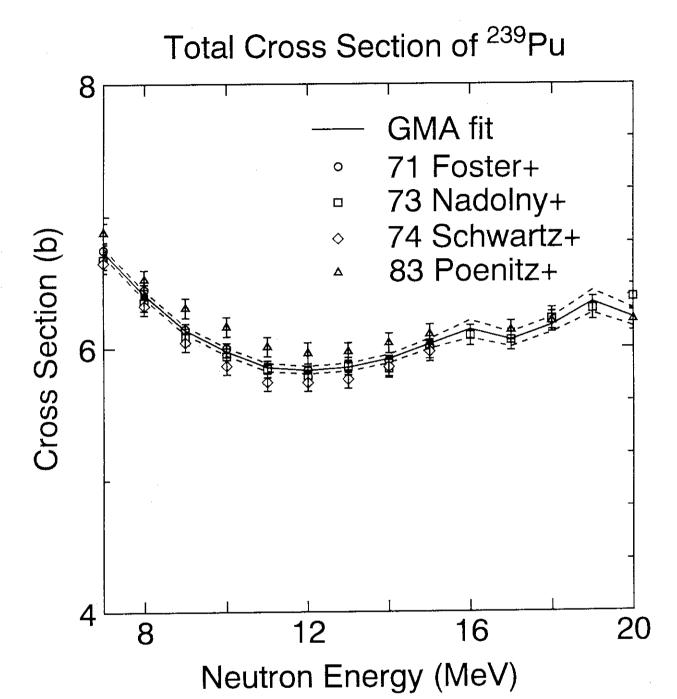


Fig. 22 Total cross section of ²³⁹Pu.

Total Cross Section of ²⁴⁰Pu

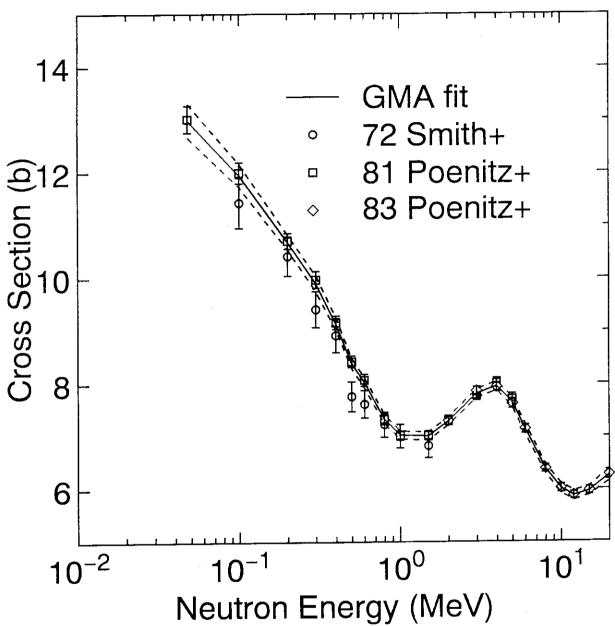


Fig. 23 Total cross section of ²⁴⁰Pu.

Li-6 Correlation Matrix

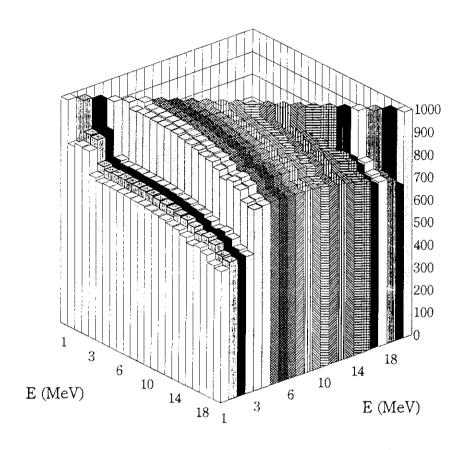


Fig. 24 Correlation matrix for ⁶Li.

Li-7 Correlation Matrix

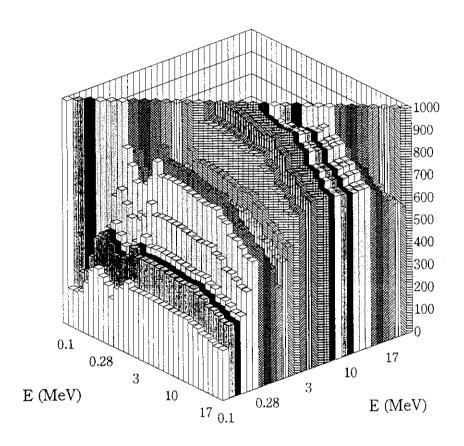


Fig. 25 Correlation matrix for ⁷Li.

Be-9 Correlation Matrix

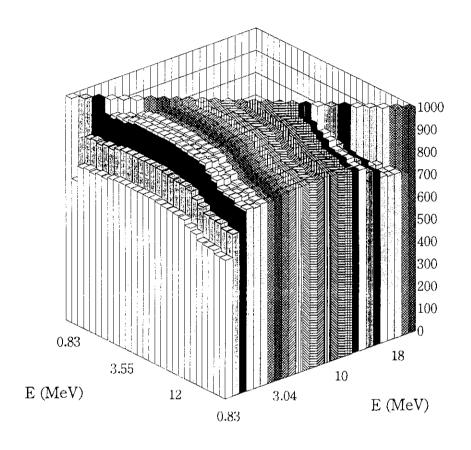


Fig. 26 Correlation matrix for ⁹Be.

C-12 Correlation Matrix

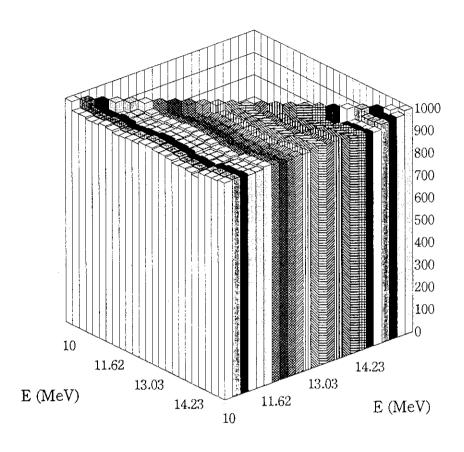


Fig. 27 Partial correlation matrix for ¹²C above 10 MeV.

O-16 Correlation Matrix

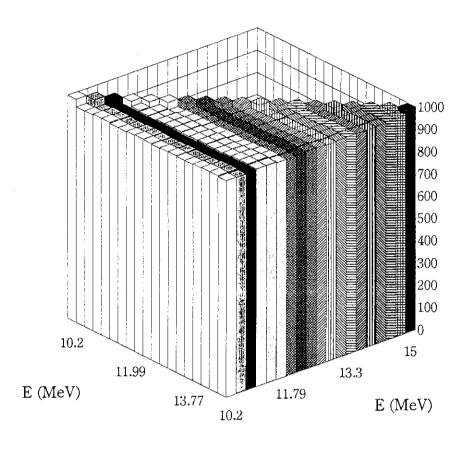


Fig. 28 Partial correlation matrix for ¹⁶O above 10 MeV.

Na-23 Correlation Matrix

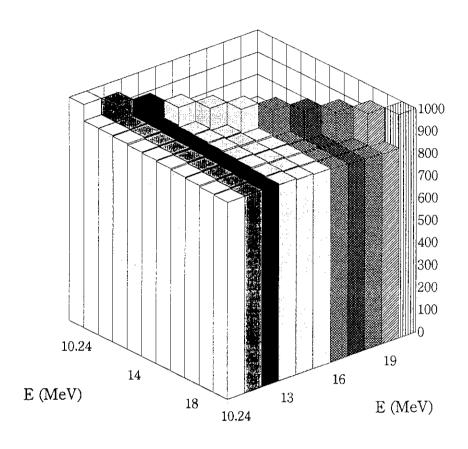


Fig. 29 Partial correlation matrix for ²³Na above 10 MeV.

Ti Correlation Matrix

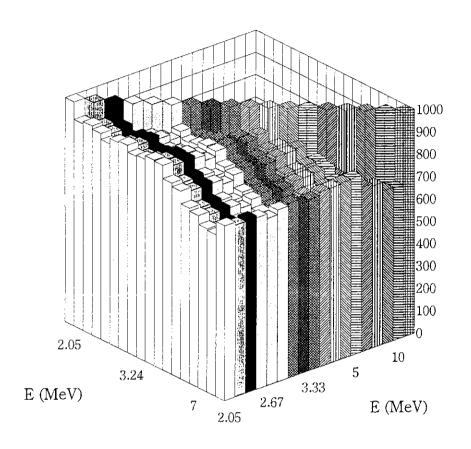


Fig. 30 Partial correlation matrix for Ti between 2 and 10 MeV.

Cr Correlation Matrix

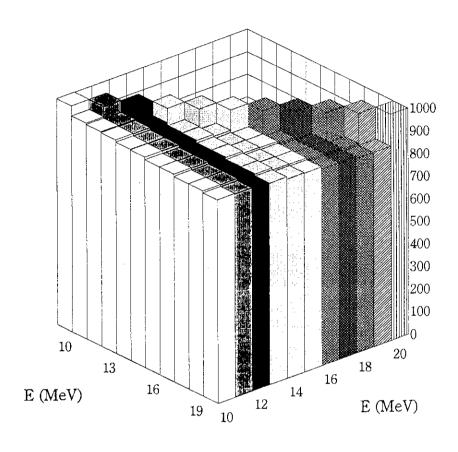


Fig. 31 Partial correlation matrix for Cr above 10 MeV.

Fe Correlation Matrix

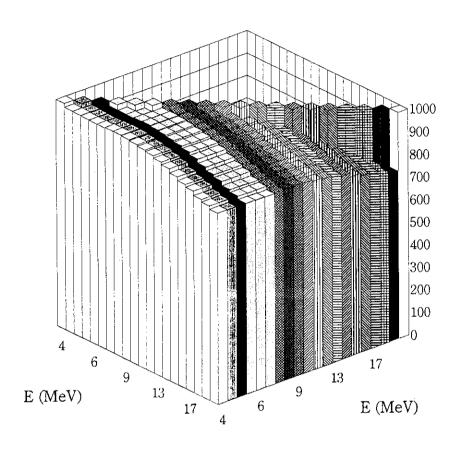


Fig. 32 Partial correlation matrix for Fe above 4 MeV.

Ni Correlation Matrix

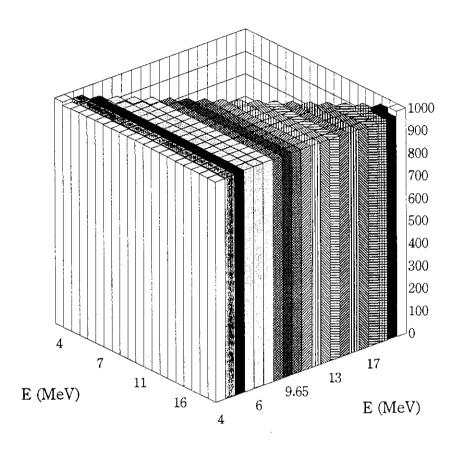


Fig. 33 Partial correlation matrix for Ni above 4 MeV.

U-235 Correlation Matrix

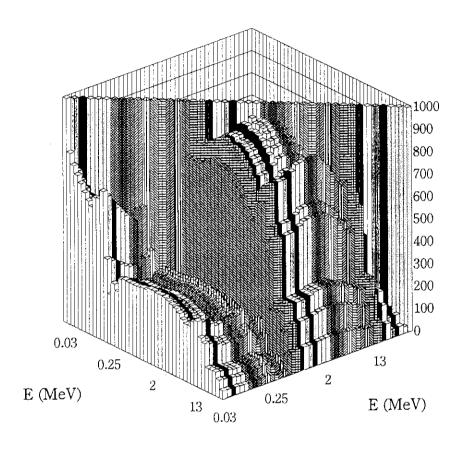


Fig. 34 Correlation matrix for ²³⁵U.

U-238 Correlation Matrix

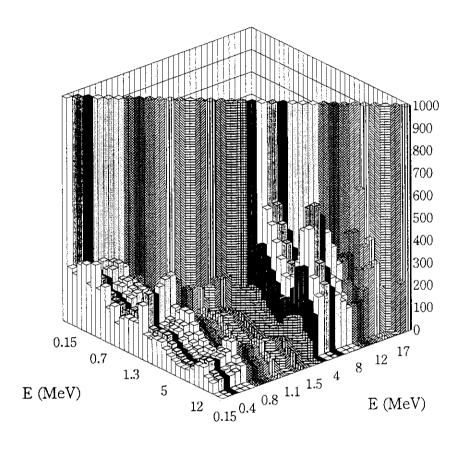


Fig. 35 Correlation matrix for ²³⁸U.

Pu-239 Correlation Matrix

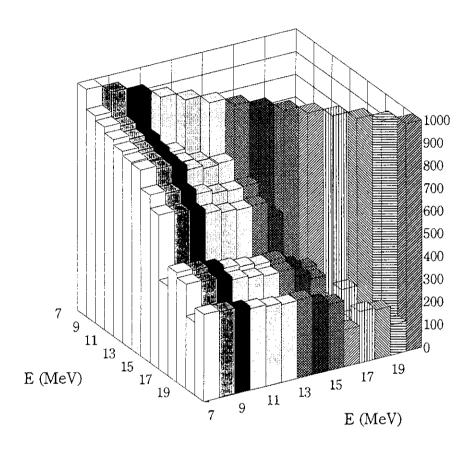


Fig. 36 Correlation matrix for ²³⁹Pu.

Pu-240 Correlation Matrix

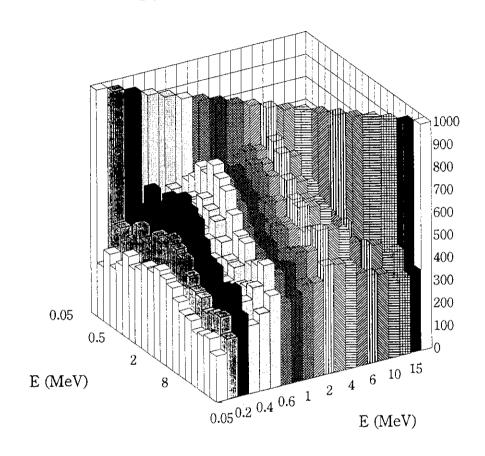


Fig. 37 Correlation matrix for ²⁴⁰Pu.