

JAERI-Research

96-056



GENERATION OF A WIMS-D/4 MULTIGROUP CONSTANTS LIBRARY BASED
ON THE JENDL-3.2 NUCLEAR DATA AND ITS VALIDATION THROUGH
SOME BENCHMARK EXPERIMENTS ANALYSIS

November 1996

Mafizur RAHMAN* and Hideki TAKANO

日本原子力研究所
Japan Atomic Energy Research Institute

本レポートは、日本原子力研究所が不定期に公刊している研究報告書です。
入手の問合せは、日本原子力研究所研究情報部研究情報課（〒319-11 茨城県那珂郡東海村）あて、お申し越しください。なお、このほかに財団法人原子力弘済会資料センター（〒319-11 茨城県那珂郡東海村日本原子力研究所内）で複写による実費頒布をおこなっております。

This report is issued irregularly.

Inquiries about availability of the reports should be addressed to Research Information Division, Department of Intellectual Resources, Japan Atomic Energy Research Institute, Tokai-mura, Naka-gun, Ibaraki-ken, 319-11, Japan.

© Japan Atomic Energy Research Institute, 1996

編集兼発行 日本原子力研究所
印 刷 いばらき印刷株

Generation of a WIMS-D/4 Multigroup Constants Library based
on the JENDL-3.2 Nuclear Data and its Validation
through Some Benchmark Experiments Analysis

Mafizur RAHMAN* and Hideki TAKANO

Department of Reactor Engineering
Tokai Research Establishment
Japan Atomic Energy Research Institute
Tokai-mura, Naka-gun, Ibaraki-ken

(Received October 4, 1996)

A new 69 group library of multigroup constants for the lattice code WIMS-D/4 has been generated with an improved resonance treatment, processing nuclear data from JENDL-3.2 by NJOY91.108. A parallel ENDF/B-VI based library has also been constructed for intercomparison of results. Benchmark calculations for a number of thermal reactor critical assemblies of both uranium and plutonium fuels have been performed with the code WIMS-D/4.1 with its three different libraries: the original WIMS library (NEA-0329/10) and the new ENDF/B-VI and JENDL-3.2 based libraries. The results calculated with both ENDF and JENDL based libraries show similar tendency and are found in better agreement with the experimental values. Benchmark parameters are further calculated with the comprehensive lattice code SRAC95. The results from SRAC95 and WIMS-D/4.1 (both with JENDL-3.2 based libraries) agrees well to each other as well as to the other previously published values.

* Institute of Nuclear Science and Technology, Atomic Energy Research Establishment,
Ganak Bari, Savar, P.O. Box 3787, Dhaka, Bangladesh

Keywords: WIMS-D/4, Multigroup Constants, NJOY91.108, JENDL-3.2, ENDF/B-VI, WIMS-Library (NEA-0329/10), Nuclear Data, Averaging Spectrum, Potential Cross Section, Goldstein-Cohen Parameter, Resonance Integrals, Resonance Interference, Thermal Reactor, Benchmark Experiments, Integral Parameters, Data Verification, Code Verification, Validation, SRAC95

JENDL-3.2 の WIMS-D/4 多群定数ライブラリー作成及びベンチマーク実験解析

日本原子力研究所東海研究所原子炉工学部

Mafizur RAHMAN*・高野 秀機

(1996年10月4日受理)

JENDL-3.2 核データを用いて、核計算コードWIMS-D/4の多群定数ライブラリーを作成した。このライブラリー作成においては、NJOY91.108プロセスコードを用いて、共鳴領域の取り扱いを改良して行った。また、ENDF/B-VI核データについても同様の計算を行い多群ライブラリーを作成した。これらの多群定数ライブラリーを用いてウラン及びプルトニウム燃料熱中性子炉のベンチマーク計算を実施した。ベンチマーク計算の結果は、JENDL-3.2とENDF/B-VIともによく似た傾向を示し実験値とも概略よい一致を示した。さらに、格子計算コードSRACT95の結果ともよく似た傾向を示した。

* バングラデッシュ原子力研究所

Contents

1. Introduction	1
2. Data Processing Details	1
2. 1 Data Processing System	2
2. 2 General Remarks on Data Processing	2
2. 3 Specific Material Processing Details	5
3. Benchmark Experiments	6
4. Benchmark Calculations	6
4. 1 Calculation with WIMS-D. 4. 1	6
4. 2 Calculation with SRAC95	7
5. Results and Discussions	7
5. 1 Data Verification	7
5. 2 Code Verification	8
6. Conclusion	9
Acknowledgment	9
References	10
Appendix-A: List of Nuclides in the Library with Details of Data Tabulation	22
Appendix-B: NJOY Input Instructions	24
Appendix-C: Input Specifications of WIMS-D/4.1 and SRAC95 for the Different Benchmark Lattice	39

1. Introduction

The WIMS package ¹⁾ originating from the Winfrith Laboratory is widely used for reactor calculations of a wide variety of thermal reactors. It consists of a lattice transport code and the associated data library. The lattice code exists in a version, of which WIMS-D/4 is available on non-commercial terms and LWR-WIMS and WIMS-E are distributed to the users through the ANSWERS service ²⁾ on commercial terms. They all use basically the same multigroup data library ³⁾ although the commercial versions allow some format extensions and their library is reported to include some further data adjustments ⁴⁾ which improve the performance of WIMS.

The libraries supplied with different versions of the WIMS code are based on old and obsolete basic evaluated data originating in the early sixties. Relatively good performance of the library was achieved through a series of empirical adjustments to the multigroup data and particularly to the resonance integrals so as to force better agreement with integral benchmark experiments through comparisons of the calculated integral parameters with a wide range of benchmark experiments. Instead, several deficiencies ^{6,7)} of the non-commercially available WIMS library, distributed by the NEA Data Bank were observed by the users. Also some important nuclides such as Am-241 for typical power and research reactor calculations are not included in library.

With the objective to improve the performance of WIMS for lattice calculations by updating its library based on one of the recently released evaluated data files such as BROND-2, CENDL-2, ENDF/B-VI, JEF-2 and JENDL-3.2, the Nuclear Data Section of IAEA has initiated a program named 'WIMS Library Update Project (WLUP)' through international cooperation in 1991 ⁵⁾. The work completed so far under this project includes: (i) optimization of WIMS inputs to model some selected benchmark experiments as accurately as possible ⁶⁾, (ii) intercomparison of data entered into the WIMS library using different processing systems ⁷⁾ and (iii) upgrading of the WIMSR module of the NJOY data processing system so that it could be used reliably to prepare group constants for the WIMS library ⁸⁾. A parametric study of the effect of different NJOY input options on integral results calculated by WIMS has also been performed and consistent set of NJOY input instructions in processing some important materials from evaluated data files have been established ⁹⁾.

With these developments in hand, we tried in the present work to construct a new WIMS-D/4 library based on JENDL-3.2, the Japanese evaluated nuclear data library. The new library is verified for its applicability to Mixed Oxide (MOX) cores of varying plutonium contents which is as well, valid for burnup calculation of uranium fueled cores.

2. Data processing details

The evaluated data files we processed are:

- i) ENDF/B-VI : ^1H , ^{16}O , ^{27}Al , ^{235}U (ENDF/B-VI.Rev.3), ^{238}U (revision-2) and
- ii) JENDL-3.2 (revision-2): 24 nuclides.

A list of nuclides in the library with details of data tabulation is given in Appendix-A.

1. Introduction

The WIMS package ¹⁾ originating from the Winfrith Laboratory is widely used for reactor calculations of a wide variety of thermal reactors. It consists of a lattice transport code and the associated data library. The lattice code exists in a version, of which WIMS-D/4 is available on non-commercial terms and LWR-WIMS and WIMS-E are distributed to the users through the ANSWERS service ²⁾ on commercial terms. They all use basically the same multigroup data library ³⁾ although the commercial versions allow some format extensions and their library is reported to include some further data adjustments ⁴⁾ which improve the performance of WIMS.

The libraries supplied with different versions of the WIMS code are based on old and obsolete basic evaluated data originating in the early sixties. Relatively good performance of the library was achieved through a series of empirical adjustments to the multigroup data and particularly to the resonance integrals so as to force better agreement with integral benchmark experiments through comparisons of the calculated integral parameters with a wide range of benchmark experiments. Instead, several deficiencies ^{6,7)} of the non-commercially available WIMS library, distributed by the NEA Data Bank were observed by the users. Also some important nuclides such as Am-241 for typical power and research reactor calculations are not included in library.

With the objective to improve the performance of WIMS for lattice calculations by updating its library based on one of the recently released evaluated data files such as BROND-2, CENDL-2, ENDF/B-VI, JEF-2 and JENDL-3.2, the Nuclear Data Section of IAEA has initiated a program named 'WIMS Library Update Project (WLUP)' through international cooperation in 1991 ⁵⁾. The work completed so far under this project includes: (i) optimization of WIMS inputs to model some selected benchmark experiments as accurately as possible ⁶⁾, (ii) intercomparison of data entered into the WIMS library using different processing systems ⁷⁾ and (iii) upgrading of the WIMSR module of the NJOY data processing system so that it could be used reliably to prepare group constants for the WIMS library ⁸⁾. A parametric study of the effect of different NJOY input options on integral results calculated by WIMS has also been performed and consistent set of NJOY input instructions in processing some important materials from evaluated data files have been established ⁹⁾.

With these developments in hand, we tried in the present work to construct a new WIMS-D/4 library based on JENDL-3.2, the Japanese evaluated nuclear data library. The new library is verified for its applicability to Mixed Oxide (MOX) cores of varying plutonium contents which is as well, valid for burnup calculation of uranium fueled cores.

2. Data processing details

The evaluated data files we processed are:

- i) ENDF/B-VI : ^1H , ^{16}O , ^{27}Al , ^{235}U (ENDF/B-VI.Rev.3), ^{238}U (revision-2) and
- ii) JENDL-3.2 (revision-2): 24 nuclides.

A list of nuclides in the library with details of data tabulation is given in Appendix-A.

2.1 Data processing system

The NJOY system¹⁰⁾ has recently become the standard tool for processing basic nuclear data into multigroup libraries for various applications. The principal advantage of NJOY is its most general purpose applicability and comprehensive capability to process data in the recent ENDF-6 format. Version NJOY91.108 is used in our work with the following corrections / changes to the IAEA updates of the WIMSR module of NJOY¹¹⁾:

- i) Modifications required for calculating library group constants with 16-resonance-groups (thermal cutoff at 2.1 eV), in contrast to the usual library with 13-resonance-groups (thermal cutoff at 4 eV), in routines RESINT, RESOUT and WIMOUT
- ii) Modifications required to calculate the resonance contributions to elastic scattering and to include corresponding resonance integrals into the WIMS library for selected isotopes (e.g., ²³⁸U, ²⁴⁰Pu and ²⁴²Pu), in routines RESINT, RESOUT and WIMOUT
- iii) A format change in writing P-1 scattering matrices to meet the requirements of 'WRITER' - a program to write WIMS binary library, in routine WIMOUT
- iv) A variable 'SIGOL' is replaced with 'SGOL', used in converting cross sections to resonance integrals, in routine WIMOUT
- v) A variable 'NZ' is replaced with 'LZ' in a statement that defines the inverse flux variable 'CFLUX' for reference or fully shielded sigma zero, in routine XSECS

2.2 General remarks on data processing

For WIMS-D library generation the following modules of NJOY are invoked in sequence: MODER-RECONR-BROADR-UNRESR-THERMR-GROUPR-WIMSR in FACOM M-780 computer. One important finding is that in RECONR, the default value '7' for the option 'number of significant digits' reconstructs the resonance integrals incorrectly; we used 6, instead.

The selection of some important data processing input parameters defines the range of applicability of a multigroup data library. The demands of the WIMS-D library in particular are rather high, since it has to cover the requirements for a large variety of lattices, differing in the type of moderator, fuel/moderator volume ratio, lattice pitch, operating conditions etc. The library format restrictions on multigroup constants representation are limited to only a few data types, where relatively weak (but non-negligible) lattice type dependence is expected. Example of such cases are the shape of the weighting function in generating the multigroup constants and the assumptions related to the detailed slowing-down treatment in generating the resonance integrals. In this work, the input option selection has been made on basic principles, having in mind a typical light water reactor (LWR). However, the applied principles and the assumptions are valid for other reactor types, too.

2.2.1 Averaging spectrum

For averaging the group constants we used the mid-life PWR flux spectrum, one of the in-built input options of the GROUPR module (IWT = 5; EPRI-CELL LWR) of NJOY. Special treatment was applied in the resonance range and is given for each nuclide separately, where applicable.

2.2.2 Fission spectrum

In the WIMS-D/4 library a single fission spectrum is specified for all fissile nuclides. This does not seem to have an strong effect on the multiplication factors of well thermalized systems, but the reaction rates were found to differ considerably with changes in fission spectrum¹²⁾. In our analysis, for uranium cores ²³⁵U fission spectrum from ENDF/B-V and for plutonium cores ²³⁹Pu fission spectrum from JENDL-3.2 are used.

2.2.3 Potential cross section

The sensitivity of the integral parameters on the choice of potential cross section is not very high, provided the resonance integrals are derived from the cross sections consistently. We used a constant value (taken from the comment section of the evaluated data file) for resonance materials and corresponding self-shielded scattering cross sections (by setting SIGP=0 on WIMSR input) for non-resonant isotopes.

2.2.4 Slowing down power

The slowing down power is defined as the product of the average lethargy decrement per collision ξ and the scattering cross section σ_s , normalized by the group lethargy widths τ . It is defined by using the self-shielded scattering cross section.

2.2.5 Goldstein-Cohen parameter

The Goldstein-Cohen value λ_g should be consistent with the resonance integral. The λ_g values for ¹H, ¹⁶O, ²³⁵U and ²³⁸U are taken from EPRI-CELL code validation report¹³⁾. This code uses resonance treatment similar to WIMS. The reported values were condensed to the WIMS group structure. For other materials, a constant value is used, taken from the present WIMS-D library.

2.2.6 Transport cross section

The transport cross section $\sigma_{tr}(g)$ in the thermal range is defined in the usual way,

$$\sigma_{tr}(g) = \sigma_a(g) + \sigma_{s0}(g) - \sum_h \sigma_{s1}(g \rightarrow h) \quad (1)$$

where $\sigma_a(g)$ is the absorption cross section, $\sigma_{s0}(g)$ the scattering cross section and $\sigma_{s1}(g \rightarrow h)$ is the first moment of the scattering cross section for transfer from group g to h. Above thermal energies an alternative definition is applied:

$$\sigma_{tr}(g) = \sigma_a(g) + \sigma_{s0}(g) - \sum_h \sigma_{s1}(h \rightarrow g) \frac{J_{(h)}}{J_{(g)}} \quad (2)$$

where $J_{(g)}$ is the neutron current spectrum. The B_1 flux approximation for large systems as calculated by NJOY was applied to define $J_{(g)}$.

2.2.7 Absorption cross section

In the WIMS definition of the absorption cross section σ_a , the values are reduced by the value of the $(n,2n)$ cross section. The neutron balance and the total cross section are preserved by taking extra neutrons into account in the transfer matrix.

2.2.8 Scattering cross section

It contains a corrections to the self-scattering term such that the sum of the absorption cross section and all the elements of the scattering matrix for a particular group add up to the transport cross section in that group (and not the total cross section). The exceptions are the moderators with explicitly given P_1 scattering matrices (i.e., hydrogen, deuterium, oxygen and carbon).

2.2.9 Resonance integrals

In the WIMS library the resonance integrals for absorption and neutron yield per fission (product of the average number of neutrons emitted per fission and the fission cross section) are tabulated as function of the background cross section σ_b . The relation between more commonly applied Bonderenko background cross section σ_0 and the WIMS σ_b is given by

$$\sigma_b = \sigma_0 + \lambda \sigma_p \quad (3)$$

where λ is the Goldstein-Cohen parameter and σ_p is the potential scattering cross section of the absorber.

The self-shielded cross section for some reaction x as calculated by NJOY can be converted to the resonance integral I_x through the relation

$$I_x = \frac{\sigma_x \sigma_b}{\sigma_b + \sigma_a} \quad (4)$$

where subscript 'a' stands for absorption. The above definition is not exact and ignores the resonance scattering, but it is exactly the inverse of the relation used in WIMS to calculate the self-shielded cross sections from the resonance integrals:

$$\sigma_x = \frac{I_x}{1 - \frac{I_a(\sigma_b)}{\sigma_b}} \quad (5)$$

2.2.10 Self-shielding due to elastic resonances

In the WIMS library, self-shielding of the absorption and the fission reaction in resonance materials can be taken into account explicitly by tabulating the resonance integrals. However, self-shielding of the scattering resonances can not be treated in the same way. As an alternative, we considered the intermediate resonance approximation in which the resonance integrals are defined

$$I_x = \int_{\text{g}} \frac{1}{\phi(E)dE} \int_{\text{g}} \sigma_x(E)\phi(E)dE \left\{ \frac{\sigma_0 + \lambda \sigma_p}{\sigma_0 + \lambda \sigma_p + \lambda \sigma_{er}(E) + \sigma_a(E)} \right\} dE \quad (6)$$

where $\phi(E)$ is the smooth neutron spectra (i.e., without perturbation due to resonances, normally by $1/E$ shape) and σ_{er} is the resonance contribution to elastic scattering. Using this definition, the relation between the self-shielded cross sections and the resonance integrals is:

$$\sigma_x(\sigma_b) = \frac{I_x(\sigma_b)}{1 - \frac{I_a(\sigma_b)}{\sigma_b} - \frac{\lambda I_{er}(\sigma_b)}{\sigma_b}} \quad (7)$$

Definitions given by equations (5) and (7) are equivalent if the resonance contribution to elastic scattering is negligible. But for nuclides with prominent elastic resonances (e.g., ^{238}U , ^{240}Pu and ^{242}Pu), equation (7) will be more appropriate to use (in WIMS) and in that case, its inverse relation

$$I_x = \frac{\sigma_x \sigma_b}{\sigma_a + \sigma_b + \lambda \sigma_{er}} \quad (8)$$

must be used (in WIMSR) in converting the self-shielded cross sections to the corresponding integrals.

2.2.11 Resonance interference treatment in NJOY

In practice, a fuel rod rarely contains only one resonance isotope. As an example, let us consider a mixture of a few percent of ^{239}Pu with ^{238}U as the major component. There will a strong dip in the flux associated with the 6.7 eV ^{238}U resonance that will affect the flux in the region of the 7.8 eV ^{239}Pu resonance (interference effect), and there will also be a dip in the flux corresponding to 7.8 eV resonance (self-shielding effect). This additional complication in the flux shape would be expected to change the group constants for ^{239}Pu since both features lie in the same group for typical group structures. However, the effect of ^{239}Pu on the ^{238}U group constants should be minimal. This argument suggests that the flux calculation be used for ^{238}U as a single resonance material. The resulting flux could then be used to estimate the flux to be used in averaging the ^{239}Pu cross section as follows:

$$\Phi_{239}(E, \sigma_0) = \frac{\Phi_{238}(E, 50)}{\sigma_0 + \frac{\Phi_{239}(E)}{1 + \{\Phi_{238}(E) / 50\}}} \quad (9)$$

where the ^{238}U flux is the characteristic of a background of 50 barns/atom, which is representative of many thermal reactor systems. This formula assumes that the effect of ^{239}Pu on the scattering source for the mixture is small, but it retains the absorption effects. The self-shielding of ^{239}Pu is treated in the narrow resonance approximation only. The GOUPR flux calculator includes an option to write out a file containing the calculated flux and cross section needed for this formula (e.g., for ^{238}U) and another option to skip the flux calculation and use the formula above to obtain the weighting flux (e.g., for ^{239}Pu).

2.3 Specific material processing details

Specific material processing details are furnished as NJOY inputs in Appendix -B.

3. Benchmark Experiments

The validation of the present work is performed by analyzing the following seventeen thermal reactor benchmark experiments:

- TRX-1, TRX-2 (1.3% enriched metal fueled, H_2O moderated critical lattices), BAPL-1, BAPL-2, BAPL-3 (1.3% enriched UO_2 fueled, H_2O moderated critical lattices) recommended by Cross Section Evaluation Working Group¹⁴⁾
- Tank Type Critical Assembly (TCA) lattices¹⁵⁾ :
 - i) 1.50U, 1.83U, 2.48U and 3.00U (2.6 w% enriched UO_2 fueled, H_2O moderated) and
 - ii) 2.42PU, 2.98PU, 4.24PU and 5.55PU (3.0 w% enriched Pu- UO_2 (MOX) fueled, H_2O moderated)
- NEACRP HCLWR lattices¹⁶⁾ :
 - (i) V6E8 ($V_m/V_f = .6$, Pu fis.=8%), (ii) V1E7 ($V_m/V_f = 1.1$, Pu fis.=7%),
- OECD/NEA lattices for Plutonium Recycling in PWRs¹⁷⁾ :
 - (i) Benchmark A - poor-isotopic-quality plutonium : $V_m/V_f = 1.9284$, Pu content= 12.5 w/o (6.0 w/o fissile), isotopic vector (^{238}Pu : ^{239}Pu : ^{240}Pu : ^{241}Pu : $^{242}Pu = 4:36:28:12:20$),
 - (ii) Benchmark B - better plutonium vector: $V_m/V_f = 1.9284$, Pu content= 4.0 w/o(2.8 w/o fissile), isotopic vector (^{238}Pu : ^{239}Pu : ^{240}Pu : ^{241}Pu : $^{242}Pu = 1.8:59:23:12.2:4.0$).

4. Benchmark calculations

Benchmark calculations for the above mentioned lattices are done with the lattice codes WIMS-D/4.1¹⁸⁾ and SRAC95¹⁹⁾ of which the input specifications are given in Appendix-C. The integral parameters calculated are:

- k_{inf} ----infinite multiplication factor
- k_{eff} ----effective multiplication factor
- ρ^{28} ----the ratio of epithermal to thermal capture reaction rates in ^{238}U ,
- δ^{25} ---- the ratio of epithermal to thermal fission reaction rates in ^{235}U ,
- δ^{28} ----the ratio of total fission reaction rates in ^{238}U and ^{235}U ,
- C^* ----the ratio of capture reaction rates in ^{238}U to fission reaction rates in ^{235}U .

The ratios refer to reaction rates correspond to a thermal cutoff energy of 0.625 eV.

4.1 Calculation with WIMS-D/4.1

Following modifications are introduced into the code WIMS-D/4.1

- To calculate with the 16-resonance-groups library, the dimension of all variables related to the number of resonance groups are changed from 13 to 16 in routines: CHN12, CHN13, CHN21, DASQUE, DATAG, DATAR, REACT, RESALT and RESINT
- To take into account of self-shielding due to elastic resonances in ^{238}U , ^{240}Pu and ^{242}Pu in routine RESALT and RESINT

3. Benchmark Experiments

The validation of the present work is performed by analyzing the following seventeen thermal reactor benchmark experiments:

- TRX-1, TRX-2 (1.3% enriched metal fueled, H_2O moderated critical lattices), BAPL-1, BAPL-2, BAPL-3 (1.3% enriched UO_2 fueled, H_2O moderated critical lattices) recommended by Cross Section Evaluation Working Group¹⁴⁾
- Tank Type Critical Assembly (TCA) lattices¹⁵⁾ :
 - i) 1.50U, 1.83U, 2.48U and 3.00U (2.6 w% enriched UO_2 fueled, H_2O moderated) and
 - ii) 2.42PU, 2.98PU, 4.24PU and 5.55PU (3.0 w% enriched Pu- UO_2 (MOX) fueled, H_2O moderated)
- NEACRP HCLWR lattices¹⁶⁾ :
 - (i) V6E8 ($V_m/V_f = .6$, Pu fis.=8%), (ii) V1E7 ($V_m/V_f = 1.1$, Pu fis.=7%),
- OECD/NEA lattices for Plutonium Recycling in PWRs¹⁷⁾ :
 - (i) Benchmark A - poor-isotopic-quality plutonium : $V_m/V_f = 1.9284$, Pu content= 12.5 w/o (6.0 w/o fissile), isotopic vector (^{238}Pu : ^{239}Pu : ^{240}Pu : ^{241}Pu : $^{242}Pu = 4:36:28:12:20$),
 - (ii) Benchmark B - better plutonium vector: $V_m/V_f = 1.9284$, Pu content= 4.0 w/o(2.8 w/o fissile), isotopic vector (^{238}Pu : ^{239}Pu : ^{240}Pu : ^{241}Pu : $^{242}Pu = 1.8:59:23:12.2:4.0$).

4. Benchmark calculations

Benchmark calculations for the above mentioned lattices are done with the lattice codes WIMS-D/4.1¹⁸⁾ and SRAC95¹⁹⁾ of which the input specifications are given in Appendix-C. The integral parameters calculated are:

k_{inf} ----infinite multiplication factor
 k_{eff} ----effective multiplication factor
 ρ^{28} ----the ratio of epithermal to thermal capture reaction rates in ^{238}U ,
 δ^{25} ---- the ratio of epithermal to thermal fission reaction rates in ^{235}U ,
 δ^{28} ----the ratio of total fission reaction rates in ^{238}U and ^{235}U ,
 C^* ----the ratio of capture reaction rates in ^{238}U to fission reaction rates in ^{235}U .

The ratios refer to reaction rates correspond to a thermal cutoff energy of 0.625 eV.

4.1 Calculation with WIMS-D/4.1

Following modifications are introduced into the code WIMS-D/4.1

- To calculate with the 16-resonance-groups library, the dimension of all variables related to the number of resonance groups are changed from 13 to 16 in routines: CHN12, CHN13, CHN21, DASQUE, DATAG, DATAR, REACT, RESALT and RESINT
- To take into account of self-shielding due to elastic resonances in ^{238}U , ^{240}Pu and ^{242}Pu in routine RESALT and RESINT

- To skip WIMS treatment of resonance interference of ^{238}U on ^{239}Pu , ^{240}Pu and ^{242}Pu in routine RESALT

Calculations are done for equivalent cylindrical lattices with:

- i) the original library (NEA0329/10),
- ii) ENDF/B-VI based new 13-resonance-groups library (thermal cutoff at 4 eV),
- iii) JENDL-3.2 based new 13-resonance-groups library (thermal cutoff at 4 eV),
- iv) JENDL-3.2 based new 16-resonance-groups library (thermal cutoff at 2.1 eV),
- v) JENDL-3.2 based new 16-resonance-groups library (thermal cutoff at 2.1 eV) with elastic resonance integrals for ^{238}U , ^{240}Pu and ^{242}Pu ,
- vi) JENDL-3.2 based new 16-resonance-groups library (thermal cutoff at 2.1 eV) with NJOY treatment of resonance interference of ^{238}U on ^{239}Pu , ^{240}Pu and ^{242}Pu .

The TRX and BAPL lattices were modeled with optimized inputs which were suggested in the final report of the WIMS Library Update Project Stage-I by Ravnik et. al.²⁰⁾. The inputs were the result of a detailed parametric study of the WIMS input options and were optimized for accuracy.

4.2 Calculation with SRAC95

Calculations are done in real lattice geometries, using the PEACO routine (ultra-fine group calculation by the collision probability method) and with JENDL-3.2 based library.

5. Results and discussions

5.1 Data verification

WIMS results for the TRX and BAPL lattices with the original WIMS library (NEA0329/10) and the new ENDF/B-VI and JENDL-3.2 based libraries are presented in Table 1. The C/E value for the five lattices from the three different libraries are plotted in Figs. 1- 5. All the calculated parameters for the five lattices using the two new libraries show similar tendency with negligible differences in some points. Almost all lie within the uncertainty limit of the measurements. These limits are slightly exceeded in ρ^{28} of TRX-1 and δ^{28} of BAPL-2, 3. The parameters lie outside twice-the-uncertainty interval are k_{eff} of TRX-2, BAPL-1, BAPL-2,3 (JENDL only) and ρ^{28} of BAPL-2.

Comparing the results from the original WIMS library and the new library with the measurements we observe that the parameters are predicted equally well. The major improvement in the new library is in predicting ρ^{28} and C^* which may have favorable consequences for burnup calculations due to the increased conversion ratio, particularly for low enriched systems. The predicted k_{eff} is apparently worse with the new library, whereas, the difference from the measurements and the spread of the results with the original WIMS library are considerably larger. No adjustments are made to the multigroup data set of the new library.

- To skip WIMS treatment of resonance interference of ^{238}U on ^{239}Pu , ^{240}Pu and ^{242}Pu in routine RESALT

Calculations are done for equivalent cylindrical lattices with:

- i) the original library (NEA0329/10),
- ii) ENDF/B-VI based new 13-resonance-groups library (thermal cutoff at 4 eV),
- iii) JENDL-3.2 based new 13-resonance-groups library (thermal cutoff at 4 eV),
- iv) JENDL-3.2 based new 16-resonance-groups library (thermal cutoff at 2.1 eV),
- v) JENDL-3.2 based new 16-resonance-groups library (thermal cutoff at 2.1 eV) with elastic resonance integrals for ^{238}U , ^{240}Pu and ^{242}Pu ,
- vi) JENDL-3.2 based new 16-resonance-groups library (thermal cutoff at 2.1 eV) with NJOY treatment of resonance interference of ^{238}U on ^{239}Pu , ^{240}Pu and ^{242}Pu .

The TRX and BAPL lattices were modeled with optimized inputs which were suggested in the final report of the WIMS Library Update Project Stage-I by Ravnik et. al.²⁰⁾. The inputs were the result of a detailed parametric study of the WIMS input options and were optimized for accuracy.

4.2 Calculation with SRAC95

Calculations are done in real lattice geometries, using the PEACO routine (ultra-fine group calculation by the collision probability method) and with JENDL-3.2 based library.

5. Results and discussions

5.1 Data verification

WIMS results for the TRX and BAPL lattices with the original WIMS library (NEA0329/10) and the new ENDF/B-VI and JENDL-3.2 based libraries are presented in Table 1. The C/E value for the five lattices from the three different libraries are plotted in Figs. 1- 5. All the calculated parameters for the five lattices using the two new libraries show similar tendency with negligible differences in some points. Almost all lie within the uncertainty limit of the measurements. These limits are slightly exceeded in ρ^{28} of TRX-1 and δ^{28} of BAPL-2, 3. The parameters lie outside twice-the-uncertainty interval are k_{eff} of TRX-2, BAPL-1, BAPL-2,3 (JENDL only) and ρ^{28} of BAPL-2.

Comparing the results from the original WIMS library and the new library with the measurements we observe that the parameters are predicted equally well. The major improvement in the new library is in predicting ρ^{28} and C^* which may have favorable consequences for burnup calculations due to the increased conversion ratio, particularly for low enriched systems. The predicted k_{eff} is apparently worse with the new library, whereas, the difference from the measurements and the spread of the results with the original WIMS library are considerably larger. No adjustments are made to the multigroup data set of the new library.

5.2 Code verification

Integral parameters for TRX and BAPL lattices calculated by SRAC95 and WIMS-D/4.1 (both with JENDL-3.2 based library) are given in Table 2 and their C/E values are plotted in Figs. 6-10. They agree well to each other excepting an overestimation in ρ^{28} of TRX-1 and TRX-2 by WIMS-D/4.1 and for BAPL-2 by SRAC95.

For TCA UO₂ lattices, k_{inf} and k_{eff} are calculated by MVP²¹⁾, SRAC95 and WIMS-D/4.1 (thrice: with original NEA 0329/10, and ENDF-B/VI and JENDL based libraries) and they are presented in Table 3. For TCA PuO₂-UO₂ (MOX) lattices, k_{inf} and k_{eff} as calculated by MVP, SRAC95 and WIMS-D/4.1 are given in Table 4. The k_{inf} and k_{eff} values are plotted as function of cell pitch in Figs. 11 and 12 for TCA UO₂ lattices and in Figs. 13 and 14 for TCA PuO₂-UO₂ lattices. They all agree reasonably to each other.

For the benchmark of NEA high conversion LWR and Plutonium Recycling, only k_{inf} is calculated and presented in Table 5. They all are underestimated by WIMS-D/4.1 (using usual 13-resonance-groups new library) compared to those from SRAC95 with a maximum of 4% for the poor-Pu-vector lattice with high content of ²⁴²Pu. These underestimation, as we thought, is due to the unshielded absorption resonance of ²⁴²Pu at 2.67 eV. We reprocessed all the materials in the new library with 16 resonance groups, taking the thermal cutoff at 2.1 eV (group number 30 of WIMS group-structure) instead of the usual one at 4.eV (group number 27). As we can see in the Table 5 and in Fig. 15, WIMS-D/4.1 results are improved to some extent with this 16-resonance-groups library. We then recalculated the parameters for TRX, BAPL, TCA UO₂ and TCA PuO₂-UO₂ lattices with WIMS-D/4.1 using the 16-resonance-groups library which are given along with percent differences from 13-resonance-groups calculations, in Tables 6, 7 and 8 respectively. The differences are positive for k_{inf} , k_{eff} and negative for other parameters. They are negligible and caused mainly due to the lesser thermal upscattering in 16-resonance-group library.

The parameters of all the seventeen lattices are further calculated with WIMS-D/4.1 (with 16-resonance-groups library), taking into account of the self-shielding due to elastic resonances of ²³⁸U, ²⁴⁰Pu and ²⁴²Pu and the effects are presented as the percent differences relative to no elastic shielding, in Tables 9 and 10. These are negligible except for δ^{25} (TRX, BAPL), and for k_{inf} of two NEA HCLWR lattices in which resonance absorption become dominant due to intermediate neutron spectrum.

Moreover, we calculated the parameters for MOX lattices of TCA and NEA to see the effects of resonance interference (of ²³⁸U on ²³⁹Pu, ²⁴⁰Pu and ²⁴²Pu) treatment in NJOY, relative to WIMS-D/4.1 interference treatment. The results as shown in Table 11. The effects are always positive for ²³⁹Pu and negative for ²⁴⁰Pu and ²⁴²Pu, with a very strong net positive contribution for NEA lattices with higher plutonium concentration. More detail study should be done for this mutual interference effect.

Finally, the parameters for the MOX lattices of NEA from WIMS-D.4.1 (including all the effects as stated above, excepting NJOY interference treatment) and SRAC95 along with some other results from previous calculations are furnished in Tables 12 and 13.

6. Conclusion

The JENDL-3.2 based 69-group WIMS-D library was prepared using NJOY91.108 data processing system. A number of thermal reactor benchmark lattices have been analyzed. The results show no significant differences with those from a similarly processed ENDF/B-VI based library. Almost all the parameters lie within or not too far from the uncertainty interval of the measurements which would further be improved by energy mesh refinement in the fast energy range. Compared to the results obtained using an original WIMS library (NEA 0329/10) it can be concluded that a significant improvement in predicting global lattice parameters has been achieved, even though the new multigroup constants library is generated from first principles.

Benchmark results for the TRX, BAPL and TCA lattices as calculated by WIMS-D/4.1 and SRAC95 (both with JENDL-3.2 based data) agree reasonably well. But for lattices with higher plutonium concentrations (NEA high conversion LWR and Plutonium Recycling lattices), WIMS-D/4.1 systematically underestimates the k_{inf} compared to SRAC95 and some other previous calculations. This underestimation, as we thought, is due to the unshielded absorption resonance of ^{242}Pu at 2.67 eV, which is found to be true, improving the results to a considerable extent by introducing a 16-resonance-groups (Thermal cutoff at 2.1 eV) WIMS library. Whereas, this 16-resonance-groups library do have no noticeable negative effect (due to lesser thermal upscattering) for the TRX, BAPL and TCA lattices. So, we recommend a 16-resonance-groups WIMS-D/4 library compared to the usual 13-resonance-groups (thermal cutoff at 4 eV) ones. A further improvement of the WIMS calculated parameters for different lattices is achieved by taking into account of the self-shielding due to elastic resonances in ^{238}U , ^{240}Pu and ^{242}Pu . Furthermore, it was shown that more detail study should be done for the mutual interference effect in higher Pu-content cores.

Acknowledgment

The first author gratefully acknowledge the support of the STA, the JAERI, Japan, the BAEC, Bangladesh and would like to thank Hiroshi Akie, Dept. of Reactor Engineering, JAERI and Kunio Kaneko, Integrated Technical Information Research Organization, Tokyo, for invaluable help and cooperation in the work. The author also wishes to express his thanks to A. Trkov, Institute 'Jozef Stefan' Slovenia for his helpful suggestions.

6. Conclusion

The JENDL-3.2 based 69-group WIMS-D library was prepared using NJOY91.108 data processing system. A number of thermal reactor benchmark lattices have been analyzed. The results show no significant differences with those from a similarly processed ENDF/B-VI based library. Almost all the parameters lie within or not too far from the uncertainty interval of the measurements which would further be improved by energy mesh refinement in the fast energy range. Compared to the results obtained using an original WIMS library (NEA 0329/10) it can be concluded that a significant improvement in predicting global lattice parameters has been achieved, even though the new multigroup constants library is generated from first principles.

Benchmark results for the TRX, BAPL and TCA lattices as calculated by WIMS-D/4.1 and SRAC95 (both with JENDL-3.2 based data) agree reasonably well. But for lattices with higher plutonium concentrations (NEA high conversion LWR and Plutonium Recycling lattices), WIMS-D/4.1 systematically underestimates the k_{inf} compared to SRAC95 and some other previous calculations. This underestimation, as we thought, is due to the unshielded absorption resonance of ^{242}Pu at 2.67 eV, which is found to be true, improving the results to a considerable extent by introducing a 16-resonance-groups (Thermal cutoff at 2.1 eV) WIMS library. Whereas, this 16-resonance-groups library do have no noticeable negative effect (due to lesser thermal upscattering) for the TRX, BAPL and TCA lattices. So, we recommend a 16-resonance-groups WIMS-D/4 library compared to the usual 13-resonance-groups (thermal cutoff at 4 eV) ones. A further improvement of the WIMS calculated parameters for different lattices is achieved by taking into account of the self-shielding due to elastic resonances in ^{238}U , ^{240}Pu and ^{242}Pu . Furthermore, it was shown that more detail study should be done for the mutual interference effect in higher Pu-content cores.

Acknowledgment

The first author gratefully acknowledge the support of the STA, the JAERI, Japan, the BAEC, Bangladesh and would like to thank Hiroshi Akie, Dept. of Reactor Engineering, JAERI and Kunio Kaneko, Integrated Technical Information Research Organization, Tokyo, for invaluable help and cooperation in the work. The author also wishes to express his thanks to A. Trkov, Institute 'Jozef Stefan' Slovenia for his helpful suggestions.

References

- 1) J.R.Askew, F.J. Fayers, P.B. Kemshell, A general Description of the Code WIMS, Jou.British Nucl. Energy Soc., p.564, Oct. 1966
- 2) ANSWERS Business, AEA Technology Centre, Dorchester Dorest DT2 8DH, England
- 3) C.J. Taubman, The WIMS 69-group Library Tape 166259, Winfrith, United Kingdom Atomic Energy Authority, AEEW-M-1324(1975)
- 4) M.J. Halsall, Recent Adjustments to the WIMS Nuclear Data Library, Winfrith, United Kingdom Atomic Energy Authority, AEEW-R-1492(1982)
- 5) S. Ganesan, Invitation letter to participate in the WIMS Library Update Project, International Atomic Energy Agency, Nov. 1990
- 6) M. Ravnik, A. Holubar, A. Trkov, WIMS Library Update Project, Final Report on Stage-1, Institute "Jozef Stefan", Ljubljana, Slovenia, IJS-DP-6245, Rev.1, Nov.1992
- 7) A. Trkov, M. Ravnik, WIMS Library Update Project, Final Report on Stage-2, Institute "Jozef Stefan", Ljubljana, Slovenia, IJS-DP-6726, May 1993
- 8) A. Trkov, T. Zidi, S. Ganasan, NJOY Verification for WIMS Library Preparation, Institute "Jozef Stefan", Ljubljana, Slovenia, IJS-DP-6622, May 1993
- 9) A.. Trkov, D.L. Aldama, Parametric study of the NJOY input options in the frame of the WIMS Library Update Project, Institute "Jozef Stefan", Ljubljana, Slovenia, IJS-DP-7049, Nov. 1994
- 10) Los Alamos National Laboratory (R.E. MacFarlane et.al., NJOY91.91 A Code System for Producing Pointwise and Multigroup Neutron and Photon Cross Sections form ENDF/B Evaluated Nuclear Data, LANL Contribution to RSIC Peripheral Shielding Routine Collection, PSR-171, 1994
- 11) A. Trkov, T. Zidi, S. Ganasan, NJOY Verification for WIMS Library Preparation (Apppendix A.1: Corrections applied to the WIMSR module of NJOY), Institute "Jozef Stefan", Ljubljana, Slovenia, IJS-DP-6622, May 1993
- 12) M. Ravnik, A. Holubar, A. Trkov, WIMS Library Update Project, Final Report on Stage-1 (4.4 Sensitivity of calculated benchmark parameters to various fission spectra), Institute "Jozef Stefan", Ljubljana, Slovenia, IJS-DP-6245, Rev.1, Nov.1992
- 13) M.L. Williams, R.Q. Wright, J. Barhen, W. Rothenstein, Benchmark of epithermal Methods in the Lattice Physics Code EPRI-CELL, Proceeding: Thermal Reactor Benchmark Calculations, Techniques, Results and applications, Upton, New York, May 17-18, 1982

- 14) CSEWG: Benchmark Specifications (With supplements), National Nuclear Data Center, BNL Upton, New York, USA, BNL-19302, Vol.II, ENDF-202)
- 15) H. Tsuruta et. Al. , Critical Sizes of Light-Water Moderated UO₂ and PuO₂-UO₂ Lattices, JAERI 1254, Feb. 1978
- 16) H. Akie, Y. Ishiguro, H. Takano, Summary Report on the International Compariso of NEACRP Burnup Benchmark Calculations for High Conversion Light Water Reactor Lattices, JAERI-M 88-200, NEACRP-L-309, Oct. 1988
- 17) OECD Documents: Physics of Plutonium Recycling Volume II (Plutonium Recycling in Pressurized-Water Reactors), Jan. 1995
- 18) WIMS-D/4.1, a new version of WIMS-D/4, Package -ID: NEA 0329/10, Nov, 1990
- 19) K. Okumura, K. Kaneko, K. Tsuchihashi, SRAC95; General Purpose Neutronics Code System, JAERI - Data/ Code 96-015, Mar. 1996
- 20) M. Ravnik, A. Holubar, A. Trkov, WIMS Library Update Project, Final Report on Stage-1 (Appendix B.2 Optimized input specifications for the benchmark lattices), Institute "Jozef Stefan", Ljubljana, Slovenia, IJS-DP-6245, Rev.1, Nov.1992
- 21) Takamasa Mori, Masayuki Nakagawa, MVP/ GMVP: General Purpose Monte Carlo Codes for Neutron and Photon Transport Calculations based on Continuous Energy and Multugroup Methods, JAERI - Data/Code 94-007, Aug. 1994

Table 1
Integral parameters for TRX and BAPL lattices calculated by WIMS-D/4.1

Lattices	Results from	k_{inf}	k_{eff}	p^{28}	δ^{25}	δ^{28}	C^*
TRX-1	EXPT.	----	1.00000 (0.3)	1.320 (1.6)	0.0987 (1.0)	0.0946 (4.3)	0.797 (1.0)
	NEA 0329 /10	1.1825 (C/E)	1.0021 (.2)	1.2784 (-3.2) 0.9685	0.0990 (.3) 1.0034	0.0965 (2) 1.0201	0.7797 (-2.2) 0.9783
	ENDF-VI	1.1767 (C/E)	0.9928 (-.72)	1.362 (3.2) 1.0318	0.0982 (-.51) 0.9949	0.0970 (2.5) 1.0254	0.8022 (.65) 1.0065
	JENDL-3.2	1.1771 (C/E)	0.9948 (-.52)	1.3612 (3.1) 1.0312	0.0982 (-.51) 0.9949	0.0959 (1.4) 1.0137	0.8016 (.58) 1.0058
TRX-2	EXPT.	----	1.00000 (.10)	0.837 (1.9)	0.0614 (1.3)	0.0693 (5.1)	0.647 (0.93)
	NEA 0329 /10	1.1633 (C/E)	0.9961 (-.39)	0.8071 (-3.6) 0.9643	0.0611 (-.49) 0.9951	0.0695 (.3) 1.0029	0.6357 (-1.7) 0.98253
	ENDF-VI	1.1624 (C/E)	0.9940 (-.6)	0.8532 (1.9) 1.0194	0.0604 (-1.6) 0.9837	0.0688 (-.7) 0.9928	0.6466 (-.06) 0.9994
	JENDL-3.2	1.1635 (C/E)	0.9958 (-.4)	0.8530 (1.9) 1.0191	0.0604 (-1.6) 0.9837	0.0684 (-1.3) 0.9870	0.6465 (-.08) 0.9992
BAPL-1	EXPT.	----	1.00000 (.10)	1.39 (.72)	0.0840 (2.4)	0.0780 (5.1)	0.00000
	NEA 0329 /10	1.1414 (C/E)	1.0027 (.27)	1.3584 (-2.3) 0.9773	0.0843 (.36) 1.0036	0.0755 (-3.2) 0.9680	0.8002
	ENDF-VI	1.1444 (C/E)	1.0024 (.24)	1.3824 (-.55) 0.9945	0.0827 (-1.5) 0.9845	0.0744 (-4.6) 0.9538	0.8029
	JENDL-3.2	1.1461 (C/E)	1.0047 (.5)	1.3833 (-.48) 0.9952	0.0826 (-1.7) 0.9833	0.0741 (-5) 0.9500	0.8032
BAPL-2	EXPT.	----	1.00000 (.10)	1.120(.89)	0.0680(1.5)	0.0700(5.7)	0.00000
	NEA 0329 /10	1.1441 (C/E)	1.0003 (.03)	1.1335 (1) 1.0121	0.0689 (1.3) 1.0132	0.0652 (-6.9) 0.9314	0.7318
	ENDF-VI	1.1478 (C/E)	1.0014 (.1)	1.1513 (2.8) 1.0280	0.0675 (-.7) 0.9926	0.0640 (-8.6) 0.9143	0.7328
	JENDL-3.2	1.1498 (C/E)	1.0036 (.4)	1.1519 (2.8) 1.0285	0.0675 (-.7) 0.9926	0.0639 (-8.7) 0.9129	0.7329
BAPL-3	EXPT.	----	1.00000(.10)	0.906(1.1)	0.0520(1.9)	0.0570(5.3)	0.00000
	NEA 0329 /10	1.1289 (C/E)	0.9980 (-.2)	0.8935 (-1.4) 0.9862	0.0530 (1.9) 1.0192	0.0538(-5.6) 0.9439	0.6568
	ENDF-VI	1.1333 (C/E)	1.0008 (.08)	0.9057 (-.03) 0.9997	0.0519 (-.2) 0.9981	0.0525 (-7.9) 0.9211	0.6552
	JENDL-3.2	1.1354 (C/E)	1.0030 (.3)	0.9061 (.01) 1.0001	0.0519 (-.2) 0.9981	0.0525(-7.9) 0.9211	0.6563

Note : Measurements and their % uncertainty (in parenthesis) are given in row-1. Other values in parenthesis are the % differences from measurements.

Table 2
Integral parameters for TRX and BAPL lattices calculated by SRAC95 and WIMS-D/4.1

Lattices	Results from	k_{inf}	k_{eff}	p^{28}	δ^{25}	δ^{28}	C^*
TRX-1	EXPT.	----	1.00000(0.3)	1.320(1.6)	0.0987(1.0)	0.0946(4.3)	0.797(1.0)
	SRAC95 (JENDL-3.2)	1.1808 (C/E)	0.999 (-.1)	1.3362 (1.2) 1.0123	0.0975 (-1.2) 0.9878	0.0958 (1.3) 1.0127	0.7951(-.23) 0.9976
	WIMS-D/4.1 (JENDL-3.2)	1.1771 (C/E)	0.9948 (-.52)	1.3612 (3.1) 1.0312	0.0982 (-.51) 0.9949	0.0959 (1.4) 1.0137	0.8016 (.58) 1.0058
TRX-2	EXPT.	----	1.00000(.10)	0.837(1.9)	0.0614(1.3)	0.0693(5.1)	0.647(0.93)
	SRAC95 (JENDL-3.2)	1.1681 (C/E)	1.0010 (.1)	0.8340 (-.36) 0.9964	0.0598(-2.6) 0.9739	0.0686(-1) 0.9899	0.6411(-.9) 0.9909
	WIMS-D/4.1 (JENDL-3.2)	1.1635 (C/E)	0.9958 (-.4)	0.8530 (1.9) 1.0191	0.0604 (-1.6) 0.9837	0.0684 (-1.3) 0.9870	0.6465 (-.08) 0.9992
BAPL-1	EXPT.	----	1.00000(.10)	1.39(.72)	0.0840(2.4)	0.0780(5.1)	0.0000
	SRAC95 (JENDL-3.2)	1.1425 (C/E)	1.0019 (.2)	1.3973 (.5) 1.0053	0.0825 (-1.8) 0.9821	0.0741 (-5) 0.9500	0.08979
	WIMS-D/4.1 (JENDL-3.2)	1.1461 (C/E)	1.0047 (.5)	1.3833 (-.48) 0.9952	0.0826 (-1.7) 0.9833	0.0741 (-5) 0.9500	0.8032
BAPL-2	EXPT.	----	1.00000 (.10)	1.120 (0.89)	0.0680 (1.5)	0.0700 (5.7)	0.0000
	SRAC95 (JENDL-3.2)	1.1472 (C/E)	1.0022 (.22)	1.1604 (3.6) 1.0361	0.0673 (-1) 0.9897	0.0639 (-8.7) 0.9129	0.73754
	WIMS-D/4.1 (JENDL-3.2)	1.1498 (C/E)	1.0036 (.4)	1.1519 (2.8) 1.0285	0.0675 (-.7) 0.9926	0.0639 (-8.7) 0.9129	0.7329
BAPL-3	EXPT.	----	1.00000(.10)	0.906(1.1)	0.0520(1.9)	0.0570(5.3)	0.0000
	SRAC95 (JENDL-3.2)	1.13411 (C/E)	1.0029 (.3)	0.9090 (.33) 1.0033	0.0517 (-.58) 0.9942	0.0526 (-7.7) 0.9228	0.6588
	WIMS-D/4.1 (JENDL-3.2)	1.1354 (C/E)	1.0030 (.3)	0.9061 (.01) 1.0001	0.0519 (-.2) 0.9981	0.0525(-7.9) 0.9211	0.6563

Note : Measurements and their % uncertainty (in parenthesis) are given in row-1. Other values in parenthesis are the % differences from measurements.

Table 3
Multiplication factors for TCA UO_2 lattices (MVP, SRAC95 and WIMS-D/4.1)

Lattice name	Code name	k_{inf}	k_{eff}
1.50U	MVP	-----	1.0026 (± 0.00052)
	SRAC95	1.3595	1.0060
	WIMS (NEA0329/10)	1.3543	1.0055
	WIMS (ENDF/B-VI)	1.3582	1.0012
	WIMS (JENDL-3.2)	1.3604	1.0042

Table 3 (Continued)

1.83U	MVP	-----	1.0031 (± 0.00057)
	SRAC95	1.3712	1.0075
	WIMS (NEA0329/10)	1.3650	1.0052
	WIMS (ENDF/B-VI)	1.3698	1.0027
	WIMS (JENDL-3.2)	1.3721	1.0055
2.48U	MVP	-----	1.0038 (± 0.00055)
	SRAC95	1.3681	1.0046
	WIMS (NEA0329/10)	1.3612	0.9998
	WIMS (ENDF/B-VI)	1.3668	1.0001
	WIMS (JENDL-3.2)	1.3693	1.0026
3.00U	MVP	-----	1.0021 (± 0.00052)
	SRAC95	1.3518	1.0061
	WIMS (NEA0329/10)	1.3454	1.0056
	WIMS (ENDF/B-VI)	1.3512	1.0025
	WIMS (JENDL-3.2)	1.3538	1.0048

Table 4
Multiplication factors for TCA PuO₂-UO₂ (MOX) lattices (MVP, SRAC95 and WIMS-D/4.1)

Lattices	Code name	k _{inf}	k _{eff}
2.42PU	MVP	-----	0.9961 (± 0.0007)
	SRAC95	1.3443	0.9907
	WIMS-D/4.1	1.3365	0.9881
2.98PU	MVP	-----	0.9994 (± 0.00059)
	SRAC95	1.3393	0.9944
	WIMS-D/4.1	1.3337	0.9928
4.24PU	MVP	-----	0.9995 (± 0.00054)
	SRAC95	1.3001	0.9978
	WIMS-D/4.1	1.2971	0.9974
5.55PU	MVP	-----	1.0001 (± 0.00074)
	SRAC95	1.2421	0.9996
	WIMS-D/4.1	1.2410	1.0006

Table 5
Multiplication factors for NEA HCLWR and Plutonium Recycling lattices (SRAC95 and WIMS-D/4.1)

Lattices	Code name	k _{inf}
NEA-V6E8	SRAC95	1.0982
	WIMS-D /4.1	13 res grps
		16 res grps
NEA-V1E7	SRAC95	1.1277
	WIMS-D /4.1	13 res grps
		16 res grps
NEA-A (poor Pu-vector)	SRAC95	1.1333
	WIMS-D /4.1	13 res grps
		16 res grps
NEA-B (better Pu-vector)	SRAC95	1.1911
	WIMS-D /4.1	13 res grps
		16 res grps

Table 6

Integral parameters for TRX and BAPL lattices from WIMS-D/4.1 with 16 resonance groups
 (The values in parenthesis are the % differences from 13 resonance group calculation)

Lattices	Results from	k_{inf}	k_{eff}	ρ^{28}	δ^{25}	δ^{28}	C^*
TRX-1	EXPT.	---	1.00000	1.320	0.0987	0.0946	0.797
	WIMS-D/4.1 (JENDL-3.2)	1.1775 (0.03)	0.9952 (0.04)	1.3580 (-0.235)	0.09752 (-0.69)	0.09579 (-0.115)	0.8008 (-0.175)
TRX-2	EXPT.	---	1.00000	0.837	0.0614	0.0693	0.647
	WIMS-D/4.1 (JENDL-3.2)	1.1637 (0.017)	0.9961 (0.03)	0.8514 (-0.19)	0.0600 (-0.66)	0.0684 (0.0)	0.646 (-0.077)
BAPL-1	EXPT.	----	1.00000	1.39	0.0840	0.0780	0.0000
	WIMS-D/4.1 (JENDL-3.2)	1.1465 (0.035)	1.0050 (0.03)	1.3809 (-0.17)	0.0822 (-0.48)	0.0741 (0.0)	0.8025 (-0.087)
BAPL-2	EXPT.	----	1.00000	1.120	0.0680	0.0700	0.0000
	WIMS-D/4.1 (JENDL-3.2)	1.1501 (0.026)	1.0040 (0.04)	1.1500 (-0.165)	0.0671 (-0.59)	0.0639 (0.0)	0.7324 (-0.068)
BAPL-3	EXPT.	----	1.00000	0.906	0.0520	0.0570	0.0000
	WIMS-D/4.1 (JENDL-3.2)	1.1357 (0.026)	1.0033 (0.03)	0.9046 (-0.166)	0.0516 (-0.578)	0.0525 (0.0)	0.6559 (-0.107)

Table 7

Multiplication factors for TCA UO₂ lattices from WIMS-D/4.1 with 16 resonance groups
 (The values in parenthesis are the % differences from 13 resonance group calculation)

Lattice name	Code name	k_{inf}	k_{eff}
1.50U	WIMS-D/4.1	1.3608 (0.029)	1.0046 (0.04)
1.83U	WIMS-D/4.1	1.3725 (0.029)	1.0058 (0.03)
2.48U	WIMS-D/4.1	1.3696 (0.022)	1.0028 (0.02)
3.00U	WIMS-D/4.1	1.3540 (0.015)	1.0050 (0.02)

Table 8

Multiplication factors for TCA PuO₂-UO₂ lattices from WIMS-D/4.1 with 16 resonance groups
 (The values in parenthesis are the % differences from 13 resonance group calculation)

Lattice name	Code name	k_{inf}	k_{eff}
2.42PU	WIMS-D/4.1	1.3382 (0.05)	0.9886 (0.05)
2.98PU	WIMS-D/4.1	1.3351 (0.04)	0.9932 (0.04)
4.24PU	WIMS-D/4.1	1.2980 (0.02)	0.9976 (0.02)
5.55PU	WIMS-D/4.1	1.2417 (0.02)	1.0007 (0.01)

Table 9
Influence of self-shielding due to U-238 elastic resonances on TRX and BAPL lattices
 (The values in the table are % differences relative to no elastic resonance shielding)

Lattice name	Results from	k_{inf}	k_{eff}	ρ^{28}	δ^{25}	δ^{28}	C^*
TRX-1	WIMS-D/4.1 (JENDL-3.2 : 16 res. grps.)	0.035	0.039	0.007	1.241	-0.03	-0.112
TRX-2	"	0.021	0.059	-0.012	1.133	0.015	-0.062
BAPL-1	"	0.026	0.029	-0.029	0.925	0.027	-0.087
BAPL-2	"	0.02	0.022	-0.009	0.88	0.016	-0.055
BAPL-3	"	0.015	0.017	0.0	0.853	0.019	-0.045

Table 10
Influence of self-shielding due to elastic resonances on TCA and NEACRP lattices
 (The values in the table are % differences relative to no elastic resonance shielding)

Lattice name	Total Pu content : number densities ($10^{24}/\text{cm}^3$)	Results from	U-238		Pu-240		Pu-242	
			k_{inf}	k_{eff}	k_{inf}	k_{eff}	k_{inf}	k_{eff}
TCA 1.50U	0.0	WIMS-D/4.1 (JENDL-3.2 : 16 res. grps.)	0.024	0.033	-----	-----	-----	-----
TCA 1.83U	0.0	"	0.019	0.027	-----	-----	-----	-----
TCA 2.48U	0.0	"	0.014	0.02	-----	-----	-----	-----
TCA 3.00U	0.0	"	0.012	0.017	-----	-----	-----	-----
TCA 2.42Pu	4.0364×10^{-4}	"	-0.004	-0.0003	0.007	0.010	-0.003	-0.003
TCA 2.98Pu	"	"	-0.003	-0.0004	0.007	0.006	-0.003	-0.002
TCA 4.24Pu	"	"	-0.001	0.0003	0.005	0.005	-0.002	-0.001
TCA 5.55Pu	"	"	-0.001	0.0002	0.004	0.004	-0.001	-0.001
NEA V6E8	2.7375×10^{-3}	"	0.4	-----	-0.01	-----	0.123	-----
NEA V1E7	2.3941×10^{-3}	"	0.2	-----	0.02	-----	0.03	-----
NEA-A	2.2927×10^{-2}	"	0.08	-----	0.02	-----	0.09	-----
NEA-B	2.155×10^{-2}	"	0.04	-----	0.06	-----	0.03	-----

Table 11

Influence of NJOY treatment of resonance interference on TCA and NEACRP lattices
 (The values in the table are % differences relative to WIMS interference treatment)

Lattices Name	Total Pu content : number densities ($10^{24}/\text{cm}^3$)	Results from	Pu-239		Pu-240		Pu-242	
			k_{inf}	k_{eff}	k_{inf}	k_{ff}	k_{inf}	k_{eff}
TCA 2.42Pu	4.0364×10^{-4}	WIMS-D/4.1 (JENDL-3.2 : 16 res. grps.)	0.012	0.024	-0.012	-0.011	-0.005	-0.004
TCA 2.98Pu	4.0364×10^{-4}	"	0.012	0.021	-0.009	-0.008	-0.004	-0.003
TCA 4.24Pu	4.0364×10^{-4}	"	0.013	0.02	-0.006	-0.006	-0.003	-0.002
TCA 5.55Pu	4.0364×10^{-4}	"	0.016	0.02	-0.004	-0.004	-0.002	-0.002
NEA V6E8	2.7375×10^{-3}	"	1.55	-----	-0.48	-----	-0.186	-----
NEA V1E7	2.3941×10^{-3}	"	0.9	-----	-0.31	-----	-0.187	-----
NEA-A	2.2927×10^{-2}	"	0.533	-----	-0.24	-----	-0.252	-----
NEA-B	2.155×10^{-2}	"	0.208	-----	-0.054	-----	-0.057	-----

Table 12
Infinite multiplication factor for NEA HCLWR Lattices
(WIMS-D/4.1, SRAC95 and Others¹⁶)

Lattice	WIMS-D /4.1	SRAC95	WIMS-E	HELIOS.HX	WIMS-ATR	BOXER	DANDE	CASMO
V6E8	1.105	1.098	1.096	1.105	1.107	1.093	1.091	1.103
V1E7	1.130	1.128	1.119	1.122	1.130	1.116	1.120	1.120

Table 13
Infinite multiplication factor for NEA Plutonium Recycling Lattices
(WIMS-D/4.1, SRAC95 and Others¹⁷)

Lattice	WIMS-D /4.1	SRAC95	ANL	BNFL	CEA	ECN	EDF	HIT
Benchmark-A	1.1310	1.1333	1.1324	1.1043	1.1334	1.1313	1.1217	1.1396
Benchmark-B	1.1822	1.1911	1.1785	1.1805	1.1896	1.1838	1.1744	1.1926

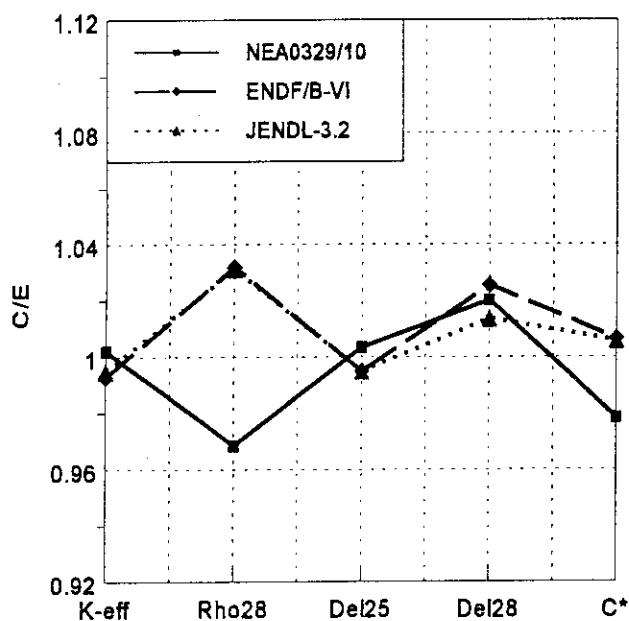


Fig. 1: C / E value of lattice parameters of TRX-1 from WIMS-D/4.1 with three different data libraries

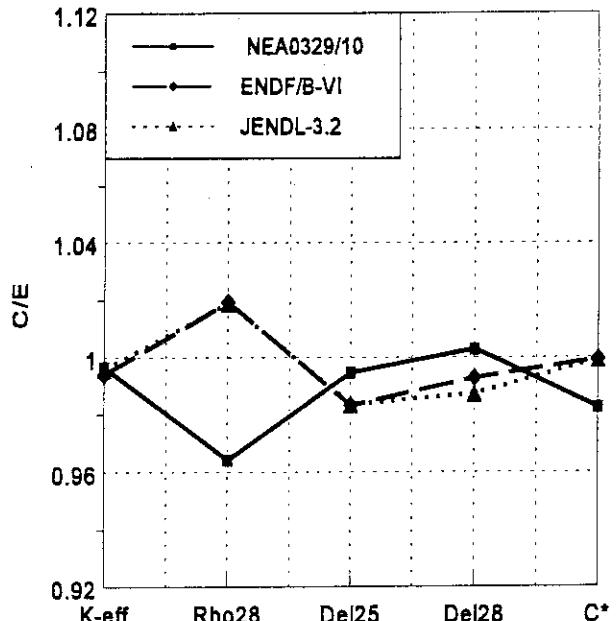


Fig. 2: C / E value of lattice parameters of TRX-2 from WIMS-D/4.1 with three different data libraries

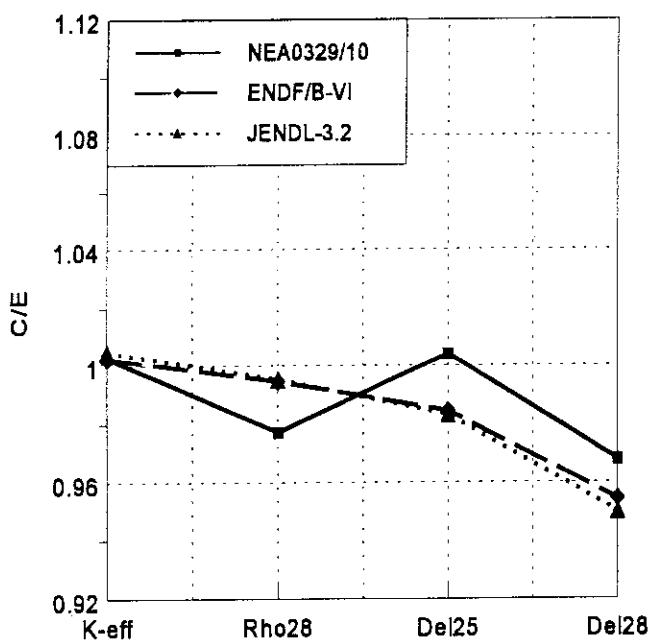


Fig. 3: C / E value of lattice parameters of BAPL-1 from WIMS-D/4.1 with three different data libraries

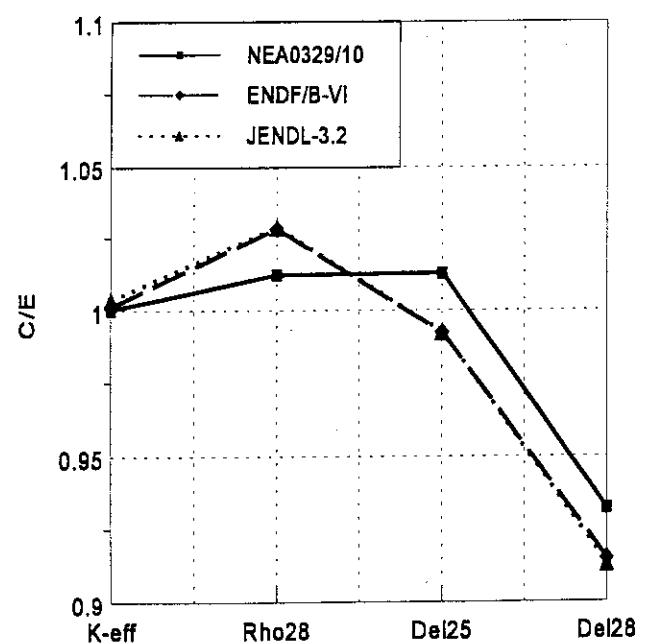


Fig. 4: C / E value of lattice parameters of BAPL-2 from WIMS-D/4.1 with three different data libraries

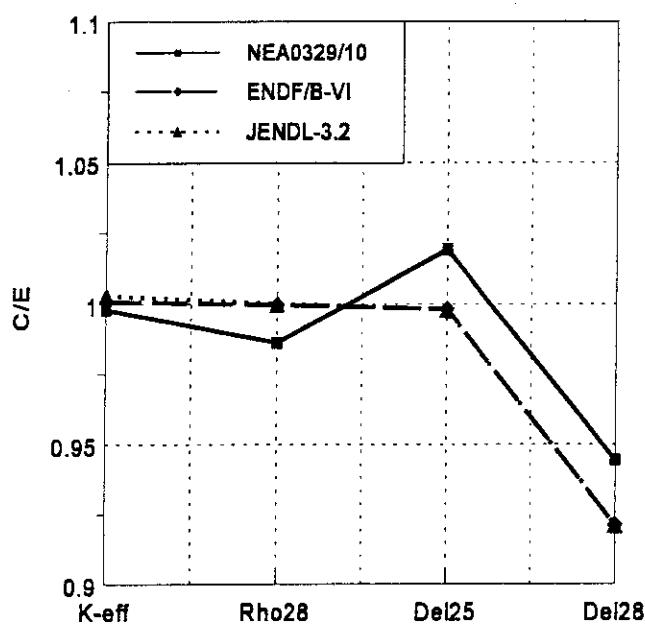


Fig. 5: C / E value of lattice parameters of BAPL-3 from WIMS-D/4.1 with three different data libraries

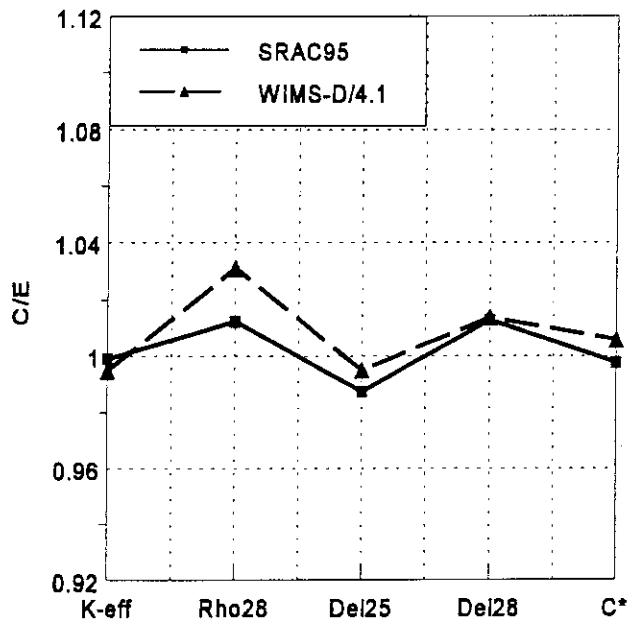


Fig. 6: C / E value of lattice parameters of TRX-1 from SRAC95 and WIMS-D/4.1

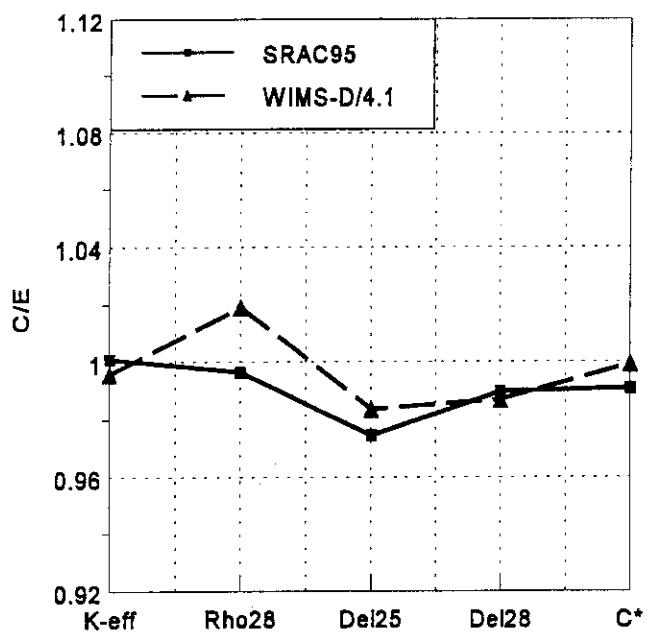


Fig. 7: C / E value of lattice parameters of TRX-2 from SRAC95 and WIMS-D/4.1

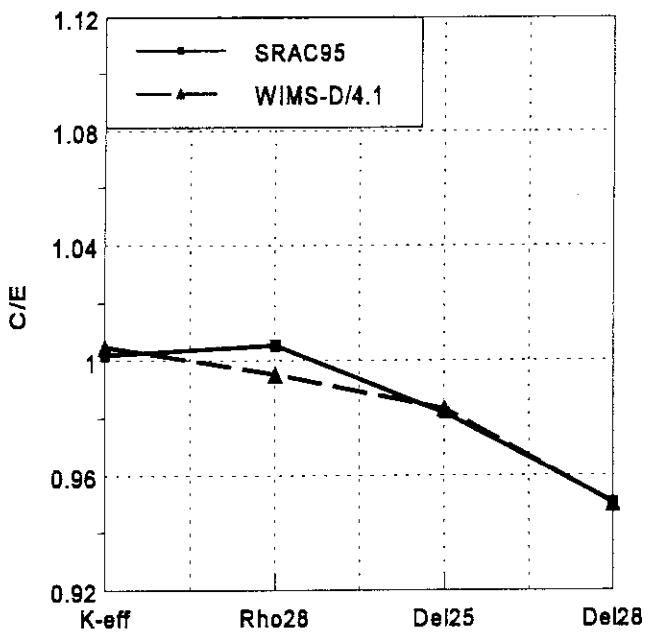


Fig. 8: C / E value of lattice parameters of BAPL-1 from SRAC95 and WIMS-D/4.1

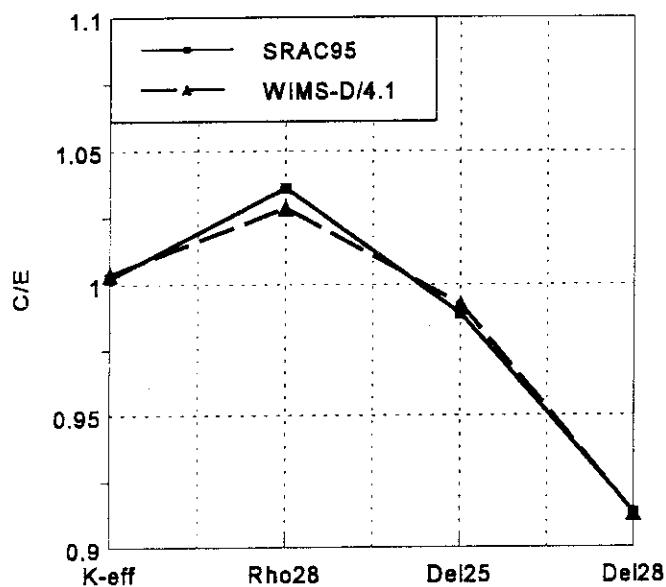


Fig. 9: C / E value of lattice parameters of BAPL-2 from SRAC95 and WIMS-D/4.1

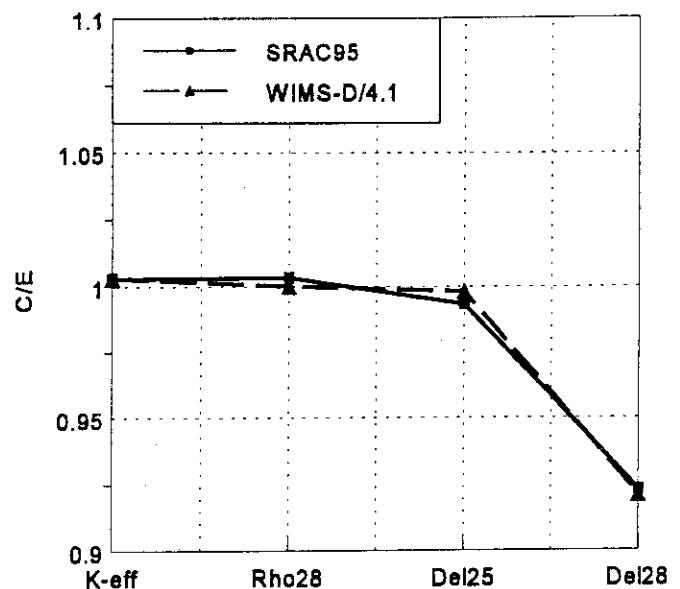


Fig. 10: C / E value of lattice parameters of BAPL-3 from SRAC95 and WIMS-D/4.1

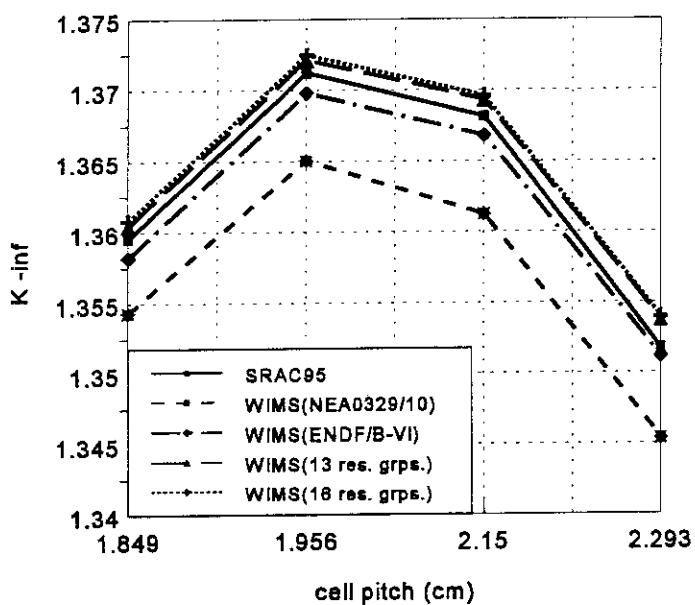


Fig. 11: k_{inf} as function of cell pitch of TCA UO_2 lattice from SRAC95 and WIMS-D/4.1 (with four data libraries)

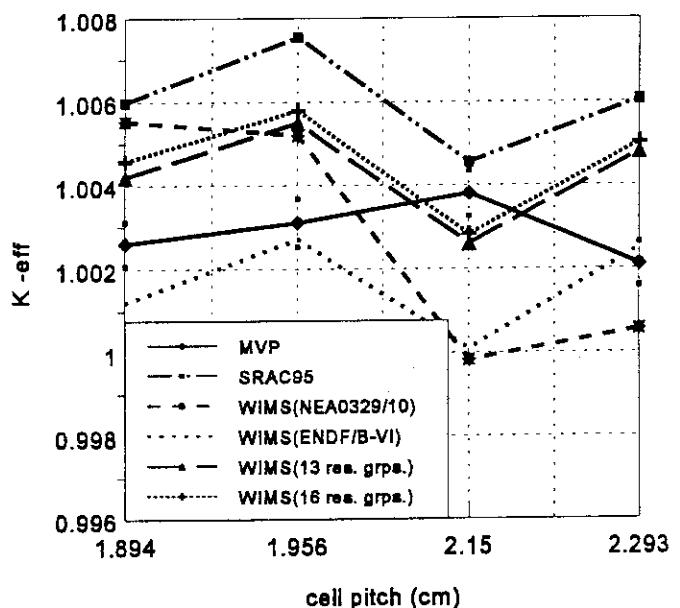


Fig. 12: k_{eff} as function of cell pitch of TCA UO_2 lattice from SRAC95 and WIMS-D/4.1 (with four data libraries)

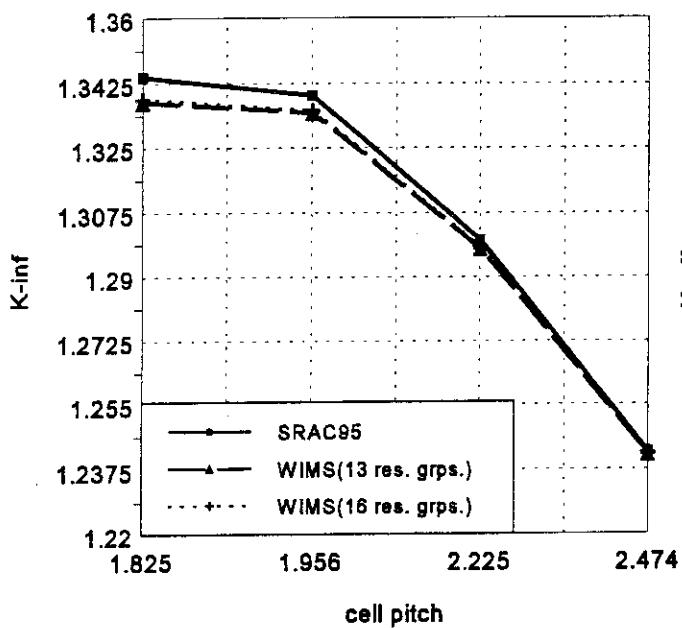


Fig. 13: k_{inf} as function of cell pitch of TCA $\text{PuO}_2\text{-UO}_2$ (MOX) lattice from SRAC95 and WIMS-D/4.1

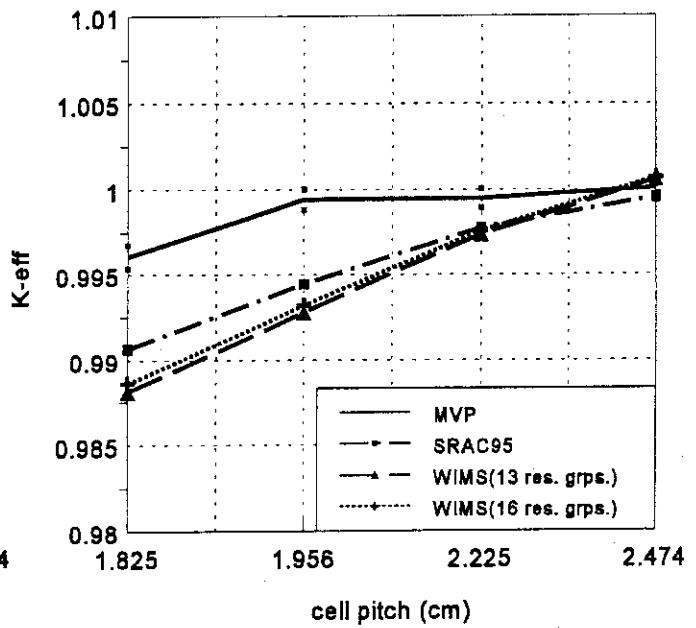


Fig. 14: k_{eff} as function of cell pitch of TCA $\text{PuO}_2\text{-UO}_2$ (MOX) lattice from SRAC95 and WIMS-D/4.1

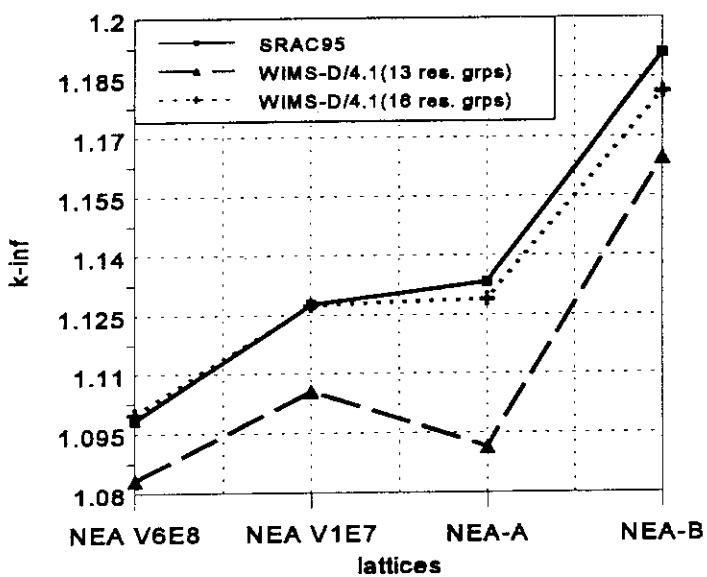


Fig. 15: k_{inf} of four NEA lattices from SRAC95 and WIMS-D/4.1

Appendix- A
List of nuclides in the library with details of data tabulation

Material	WIMS ID	Self-Shielding Model , Reference Sigma Zero	Resonance Tabulation, Sigma Zero Grid	Thermal Scattering Law, Temperatures
¹ H	125	Narrow Resonance Approx. (NRA), Infinity	No resonance tabulation	H-1 in H ₂ O, (293, 400, 500, 600)
¹ H in ZrH	126	"	"	H-1 in ZRH, (293, 400, 600, 1200)
² D	128	"	"	D-2 in D ₂ O, (93, 400, 500, 600)
¹⁰ B	525	"	"	Free gas, (293, 500, 700, 1600)
¹¹ B	528	"	"	Free gas, (293, 600)
¹² C	625	"	"	Graphite, (293, 500, 700,1000,1600)
¹⁶ O	825	"	"	Free gas, (293, 600, 900, 1500)
²⁷ Al	1325	"	"	Free gas, (293)
Cr (nat.)	2400	NJOY Flux Calculator upto 3E5eV & NRA above, Infinity	"	Free gas, (293, 600)
⁵⁵ Mn	2525	NJOY Flux Calculator upto 1E5eV & NRA above, Infinity	"	Free gas, (293, 600)
Fe (nat.)	2600	NJOY Flux Calculator upto 3.5E5eV & NRA above, Infinity	"	Free gas, (293, 600)
Ni (nat.)	2800	NJOY Flux Calculator upto 5.53E5eV & NRA above, Infinity	"	Free gas, (300, 600)
Zr (nat.)	4000	NJOY Flux Calculator upto 3.016E4eV & NRA above, Infinity	"	Free gas, (293, 600)
Zr in ZrH	4001	NJOY Flux Calculator upto 3.016E4eV & NRA above, Infinity	"	Free gas, (293, 400, 600, 1200)
²³⁴ U	9225	NJOY Flux Calculator upto 1.5E3eV & NRA above, 1E6	Abs . Integral only, (5E1,1E2,1E3,1E4,1E5, 1E6,1.5E6, 2E6,8E6, 1E10)	Free gas, (293, 600, 1500)

Appendix- A (continued)

^{235}U	9228	NJOY Flux Calculator upto 5E2eV & NRA above, 8E2	Abs. & Fiss. Integral, (5E2,8E2,1.2E3,1.8E3, 2.8E3,4.5E3,8E3,1E4, 3E4,1E10)	Free gas, (293, 600, 900, 1500)
^{236}U	9231	NJOY Flux Calculator upto 1.53E3eV & NRA above, 1E6	Abs. Integral only, (5E1,1E2,1E3,1E4,1E5, 1E6,1.5E6, 2E6,8E6, 1E10)	Free gas, (293, 600, 1500)
^{238}U	9237	NJOY Flux Calculator upto 1E4eV & NRA above, 28	Abs. & Elst. Integral, (1.2E1,2.8E1,5.E1,1E2, 1.4E2,2.6E2,1E3,1E4, 1E5,1E10)	Free gas, (293, 600, 900, 1500)
^{238}Pu	9434	NJOY Flux Calculator upto 5E2eV & NRA above, 1E5	No resonance tabulation	Free gas, (293, 600, 1500)
^{239}Pu	9437	NJOY Flux Calculator upto 2.5E3eV & NRA above, 1E3	Abs. & Fiss. Integral, (1E1,5E1,1E2,1E3,2E3,1E 4,1E5,1E6,1.5E6, 1E10)	Free gas, (293, 600, 900, 1500)
^{240}Pu	9440	NJOY Flux Calculator upto 4E3eV & NRA above, 1E4	Abs. & Elst. Integral, (5E1,1E2,1E3,1E4,1E5, 1E6,1.5E6, 2E6,8E6, 1E10)	Free gas (293, 600, 900, 1500)
^{241}Pu	9443	NJOY Flux Calculator upto 3E2eV & NRA above, 2E3	Abs. & Fiss. Integral, (1E1,5E1,1E2,1E3,2E3,1E 4,1E5,1E6,1.5E6, 1E10)	Free gas, (293, 600, 900, 1500)
^{242}Pu	9446	NJOY Flux Calculator upto 1.15E3eV & NRA above, 1E5	Abs. & Elst. Integral, (5E1,1E2,1E3,1E4,1E5, 1E6,1.5E6, 2E6,8E6, 1E10)	Free gas, (293, 600, 900, 1500)
^{241}Am	9543	NJOY Flux Calculator upto 1.5E2eV & NRA above, 1E6	Abs . Integral only, (5E1,1E2,1E3,1E4,1E5, 1E6,1.5E6, 2E6,8E6, 1E10)	Free gas, (293, 600, 1500)

APPENDIX -B: NJOY INPUT INSTRUCTIONS

6 2222 /
 0 /
 3 / 600K
 3 222 /
 3 252 /
 6 /
 6 222 /
 0 /
 WIMSR
 0 /
 MODER
 20 -21
 MODER
 30 -31 / CONVERT SCATTERING LAW DATA TO BINARY UNIT-31
 RECONR
 -21 -22
 PENDF TAPE FOR H-1(H2O) FROM JENDL-3.2(F6) TAPE 301/
 125 2/
 .002 0 6 .002 1 E-10
 1-H-1(H2O) FROM JENDL-3.2(F6) TAPE T301/
 *PROCESSED BY THE NJOY91-V108 */
 0/
 BROADR
 -22 -23
 125 4 0 0 0 /
 002 1 E06 .002 1 E-10/
 293. 400. 500. 600.
 0/
 UNRESR
 -21 -23 -24
 125 4 1 1
 293. 400. 500. 600.
 1.E10
 0/
 THERMR* / ADD THERMAL SCATTERING DATA TO UNIT-26
 -31 -24 -26
 1.125 8 4 4 0 2 222 1
 293. 400. 500. 600.
 0.805 2.1
 1.E10
 GROUFR
 -21 -26 0 -25
 125 9 0 5 1 4 1 1
 1-H-1(H2O) FROM JENDL-3.2(F6)/
 293. 400. 500. 600.
 0.805 2.1
 3 / 293K
 3 222 /
 3 252 /
 6 /
 6 222 /
 0 /
 3 / 400K
 3 222 /
 3 252 /
 6 /
 6 222 /
 0 /
 3 / 500K
 3 222 /
 3 252 /
 6 /

JAERI-Research 96-056

B-2: HYDROGEN BOUND H₂O

0 /
 6 /
 MODER
 20 -21
 MODER
 30 -31 / CONVERT SCATTERING LAW DATA TO BINARY UNIT-31
 RECONR
 -21 -22
 PENDF TAPE FOR H-1(H2O) FROM JENDL-3.2(F6) TAPE 301/
 125 2/
 1 0 0 1
 125
 0 0 1.E10 0 0 222 9 0 1 0 0 /
 1 0.0 1.00 1.00 1.00 1.00 .990 .987 .987 .963 .966 1. 941
 .941 .941 .941
 STOP

B-2: HYDROGEN BOUND IN ZRH

0 /
 6 /
 MODER
 20 -21
 MODER
 30 -31 / CONVERT SCATTERING LAW DATA TO BINARY UNIT-31
 RECONR
 -21 -22
 PENDF TAPE FOR H-1(ZRH) FROM JENDL-3.2(F6) TAPE 301/
 125 2/
 002 0 6 .002 1 E-10
 1-H-1(ZRH) FROM JENDL-3.2(F6) TAPE T301/
 *PROCESSED BY THE NJOY91-V108 */
 0/
 BROADR
 -22 -23
 125 4 0 0 0 /
 002 1 E06 .002 1 E-10
 293. 400. 600.
 0/
 UNRESR
 -21 -23 -24
 125 4 1 1
 293. 400. 600.
 1.E10
 0/
 THERMR / ADD THERMAL SCATTERING DATA TO UNIT-26
 -31 -24 -26
 7.125 8 4 4 12 1 225 1
 293. 400. 600. 1200.
 0.005 2.1
 GROUPR
 -21 -26 0 -25
 125 4 1 1
 293. 400. 600. 1200.
 1.E10
 0 /
 THERMR / ADD THERMAL SCATTERING DATA TO UNIT-26
 -31 -24 -26
 7.125 8 4 4 12 1 225 1
 293. 400. 600. 1200.

1-H-1(ZRH) FROM JENDL-3.2(F6)/
 1-H-1(ZRH) FROM JENDL-3.2(F6)/

293. 400. 600. 1200.
 1.E10
 3 / 293K
 3 225 /
 3 226 /
 3 252 /
 6 /
 6 225 /
 6 226 /
 0 /
 3 / 400K
 3 225 /
 3 226 /
 3 252 /
 6 /
 6 225 /
 6 226 /
 0 /
 3 / 600K
 3 225 /
 3 226 /
 3 252 /
 6 /
 6 225 /
 6 226 /
 0 /
 3 / 1200K
 3 225 /
 3 226 /
 3 252 /
 6 /
 6 225 /
 6 226 /
 0 /
 0 /
 WIMSR
 -25 27
 24
 1.001
 125
 1
 0.01.E10 0. 225 226 1 1.00 / NO PI SCATT.
 1.00 1.00 1.00 1.00 1.00 .987 .987 .963 .966 1.0 .941
 .941 .941 .941
 STOP

128 2/
 002. 0 6 .002 1.E-10
 2-D-1(D₂O) FROM JENDL-3.2(F6) TAPE T301/
 *PROCESSED BY THE NJOY91-V108 */
 0/
 BROADR
 -22 -23
 128 4 0 0 0 /
 002 1 E06 .002 1.E-10/
 293. 400. 500. 600.
 0/
 UNRESR
 -21 -23 -24
 128 4 1 1
 293. 400. 500. 600.
 1.E10
 0/
 THERMR / ADD THERMAL SCATTERING DATA TO UNIT-26
 -31 -24 -26
 11 128 8 4 4 0 2 228 1
 293. 400. 500. 600.
 0.005 2.1
 GROUPR
 -21 -26 0 -25
 128 9 0 5 1 4 1 1
 2-D-1(D₂O) FROM JENDL-3.2(F6)/
 293. 400. 500. 600.
 1.E10
 3 / 293K
 3 228 /
 3 252 /
 6 /
 6 228 /
 0 /
 3 / 400K
 3 228 /
 3 252 /
 6 /
 6 228 /
 0 /
 3 / 500K
 3 228 /
 3 252 /
 6 /
 6 228 /
 0 /
 3 / 600K
 3 228 /
 3 252 /
 6 /
 6 228 /
 0 /
 6 /
 6 228 /
 0 /
 0 /
 MODER
 20 -21
 MODER
 30 -31 / CONVERT SCATTERING LAW DATA TO BINARY UNIT-31
 RECONR
 -21 -22
 PENDF TAPE FOR2-D-1(D₂O) FROM JENDL-3.2(F6) TAPE 301/
 128

B-3 : DEUTERIUM BOUND IN D₂O

0 /
 6 /
 MODER
 20 -21
 MODER
 30 -31 / RECONR*
 -21 -22
 PENDF TAPE FOR2-D-1(D₂O) FROM JENDL-3.2(F6) TAPE 301/

1 3 252 /
 0.0 1.E1000. 228 00 1.00 /
 1.00 1.00 1.00 1.00 .990 .987 .963 .966 1. .941
 941 .941 .941
 STOP

B-4 : BORON -10
 0 /
 6 / *MODER*
 0 / *RECONR*
 20 -21 *PENDF TAPE FOR B-10 FROM JENDL-3.2(F6) TAPE 301*/
 -21 -22 *5-B-10 FROM JENDL-3.2(F6) TAPE T301 */
 525 2/ *PROCESSED BY THE NJOY91-V108 */
 0/ *BROADR*
 -22 -23
 525 4 0 0 0 /
 .002 1.E06 .002 1.E-10 /
 293. 500. 700. 1600.
 0/ *UNRESR*
 21 -23 -24
 525 4 .2 1
 293. 500. 700. 1600.
 1.E10 1.E04
 0/ *THERMR*
 0 -24 -26
 0 525 12 4 1 0 1 221 1
 293. 500. 700. 1600.
 0.005 2.1
 GROUPR
 21 -26 0 -25
 525 9 0 5 1 4 2 1
 5-B-10 FROM JENDL-3.2(F6)/
 293. 500. 700. 1600.
 1.E10 1.E04
 3 / 293K
 3 221 /
 3 252 /
 6 / 6 221 /
 0 / *THERMR*
 3 / 500K
 3 221 /
 3 252 /
 6 / 6 221 /
 0 / *GROUPR*
 -21 -26 0 -25
 528 9 0 5 1 2 2 1
 5-B-11 FROM JENDL-3.2(F6)/

B-5 : BORON-11
 0 /
 6 / *MODER*
 0 / *RECONR*
 20 -21 *PENDF TAPE FOR B-11 FROM JENDL-3.2(F6) TAPE 301*/
 -21 -22 *PENDF TAPE FOR B-11 FROM JENDL-3.2(F6) TAPE 301*/
 528 2/ 0/ *BROADR*
 -22 -23
 .002 0 .6 .002 1.E-10 /
 *5-B-11 FROM JENDL-3.2(F6) TAPE T301 */
 *PROCESSED BY THE NJOY91-V108 */
 0/ *UNRESR*
 528 2 0 0 0 /
 .002 1.E06 .002 1.E-10 /
 293. 600.
 1.E10 1.E04
 0/ *THERMR*
 -21 -23 -24
 528 2 2 1
 293. 600.
 0.005 2.1
 GROUPR
 -21 -26 0 -25
 528 9 0 5 1 2 2 1
 5-B-11 FROM JENDL-3.2(F6)/

B-7: OXYGEN-16

6/
6221/
0/
0/
6/
MODER
20-21
RECONR
-21-22
PENDF TAPE FOR O-16 FROM JENDL-3.2(F6) TAPE 301/
825 2/
.002 .0 .6 .002 1.E-10
*8-O-16 FROM JENDL-3.2(F6) TAPE T301 */
*PROCESSED BY THE NJOY91-V108 */
0/
BROADR

-22-23
825 4 0 0 0/
.002 1.E06 .002 1.E-10/
293. 600. 900. 1500.
0/
UNRESR
-21-23-24
825 4 5 1
293. 600. 900. 1500.
1.E10 1.E04 1.E03 40. 28.
0/
THERMR
0-24-26
0.825 12. 4. 1. 0. 1. 221. 1
293. 600. 900. 1500.
0.005 2.1
GROUPR

GROUPR
-21-26 0-25
825 9 0 5 1 4 5 1
8-O-16 FROM JENDL-3.2(F6)/
293. 600. 900. 1500.
1.E10 1.E04 1.E03 40. 28.
3/
3.221 /
3.252 /
6/
6.221 /
3.221 /
600K
3.252 /
6/
6.221 /
0/
3/
900K
3.221 /
3.252 /
6/
6.221 /
0/
3/
1500K
3.221 /
3.252 /

B-8: ALUMINIUM-27

0 /
6 /
MODER
20-21
RECONR
-21-22
PENDF TAPE FOR AL-27 FROM JENDL-3.2(F6) TAPE 301/
1325 2/
.002 0. 6 .002 1.E-10
*13-AL-27 FROM JENDL-3.2(F6) TAPE T301 */
*PROCESSED BY THE NJOY91-V108 */
0/
BROADR

-22-23
1325 1 0 0 0/
.002 1.E06 .002 1.E-10/
293./
0/
UNRESR
-21-23-24
1325 1 2 1
293.0/
1.E10 5.E02
0/
THERMR
0-24-26
0.1325 12. 1. 1. 0. 1. 221. 1
293.0
0.005 2.1
GROUPR
-21-26 0-25
1325 9 0 5 1 1 2 1
13-AL-27 FROM JENDL-3.2(F6)/
293.
1.E10 5.E02
3 / 293K
3.221 /
3.252 /
6/
6.221 /

-21 -26 0 -25
 2800 9 -5 1 2 2 1
 N(NAT) FROM JENDL-3.2(F6) /
 300. 600.
 1.E10 1.E4
 5.53E05 17.859 30000 / CHECK SPECTRUM AVOVE UPPER END OF RR RANGE
 3 / 300K
 3 221 /
 3 222 /
 6 /
 6 221 /
 0 /
 3 / 600K
 3 221 /
 3 222 /
 6 /
 6 221 /
 0 /
 0 /
 WIMSR
 -25 27
 2.4
 1 0 0 1
 2800
 28
 0 0 1.E10 0 0.0 221 0 1 100 /
 0.63 0.63 0.63 0.63 0.63 0.63 0.63 0.63 0.63 0.63 0.63 0.63
 STOP
 B-13: ZIRCONIUM (NATURAL)
 0 /
 6 /
 MODER
 20 -21
 RECONR
 -21 -22
 PENDF TAPE FOR ZR(NAT) FROM JENDL-3.2(F6) TAPE T306/
 4000 2 /
 .002 293. 6 .002 1.E-10/
 ZR(NAT) FROM JENDL-3.2(F6) TAPE T306/
 *PROCESSED BY THE NJOY91-V108 */
 0 /
 BROADR
 -22 -23
 4000 4 0.0 /
 .003 1.E06 .06 1.E-07/
 293. 600.
 0 /
 THERMR /
 0 -23 -26
 0 4000 12 2 1 0 1 221 1
 .003 1.E06 .06 1.E-07/
 293. 600.
 0.005 2.1
 GROUPR
 -21 -26 0 -25
 4000 9 0 -5 1 2 2 1
 ZR(NAT) FROM JENDL-3.2(F6) /
 293. 600. /
 1.E10 1.E4
 3.016E04 6.4084 30000 / CHECK SPECTRUM AVOVE UPPER END OF RR RANGE
 3 / 293K
 3 221 /
 3 222 /
 6 /
 6 221 /
 0 /
 3 / 600K
 3 221 /
 3 222 /
 6 /
 6 221 /
 0 /
 0 /
 WIMSR
 -25 27
 2.4
 1 0 0 1
 4000
 40
 0 0 1.E10 0 0.0 221 0 1 100 /
 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22
 0.22 0.22 0.22
 STOP
 B-14: ZIRCONIUM (IN ZRH)
 0 /
 6 /
 MODER
 20 -21
 MODER
 30 -31 / CONVERT SCATTERING LAW DATA TO BINARY UNIT-31
 RECONR
 -21 -22
 PENDF TAPE FOR ZR(ZRH) FROM JENDL-3.2(F6) TAPE T306/
 4000 2 /
 .002 .0 6 .04 1.E-10/
 ZR(ZRH) FROM JENDL-3.2(F6) TAPE T306/
 *PROCESSED BY THE NJOY91-V108 */
 0 /
 BROADR
 -22 -23
 4000 4 0.0 /
 .003 1.E06 .06 1.E-10/
 293. 400. 600. 1200.
 0 /
 THERMR /
 -31 -23 -26
 58 4000 12 4 4 13 1 235 1
 293. 400. 600. 1200.
 0.005 2.1
 GROUPR
 -21 -26 0 -25
 4000 9 0 -5 1 4 3 1
 ZR(ZRH) FROM JENDL-3.2(F6) /

BROADR
 1.E10 1.E3 5.E2
 3.016E04 6.4084 9000 / CHECK SPECTRUM ABOVE UPPER END OF RR RANGE
 3/ 293K
 3.235 /
 3.236 /
 3.232 /
 6/
 6.235 /
 0/
 3/ 400K
 3.235 /
 6/
 3.236 /
 3.232 /
 6/
 6.235 /
 0/
 3/ 600K
 3.235 /
 3.236 /
 3.232 /
 6/
 6.235 /
 0/
 3/ 1200K
 3.235 /
 3.236 /
 3.232 /
 6/
 6.235 /
 0/
 0/
 WMSR
 -25.27
 2.4
 1.001
 4000
 40
 0.01.E09 0.0 235 236 1.100 /
 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22 0.22
 0.22 0.22 0.22
 STOP
 B-15: URANIUM-234
 0 /
 6 /
 MODER
 20.-21
 RECONR
 -21.-22
 PENDF TAPE FOR U-234 FROM JENDL-3.2(F6) TAPE T313 /
 9225.2 /
 .002 0.6 .04 1.E-0 /
 92.U-234 FROM JENDL-3.2(F6) TAPE T313 /
 *PROCESSED BY THE NJOY91-V108 * /
 0/
 WTMSR
 -25.27
 2.4
 1.001
 9225
 92
 92

0.0 1.E06 1.1941 .221 0.1 1.00 /
 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965
 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965 0.1965
 STOP

B-16: URANIUM-235

0 /
 6 /
 MODER
 20 -21
 RECONR
 -21 -22

PENDF TAPE FOR U-235 FROM JENDL-3.2(F6) TAPE T313/
 9228 2/
 .002 0. 6 .04 1.E-07 /
 92-U-235 FROM JENDL-3.2(F6) TAPE T313/
 *PROCESSED BY THE NIOY91-V108 */
 0/
 BROADDR
 -22 -23

9228 4 0 1 0 /
 .002 1.E06 .04 1.E-07 /
 293. 600. 900. 1500. /

UNRESR
 -21 -23 -24

9228 4 10 1
 1.E10 3.E04 1.E04 8.E03 4.5E03 2.8E03 1.8E03 1.2E03 8.E02 5.E02
 9228 12 4 1 0 1 221 1
 293. 600. 900. 1500.
 0.005 2.1

GROUPR
 -21 -26 0 -25

9228 9 0 .5 1.4 10 1
 92-U-235 FROM JENDL-3.2(F6)/
 293. 600. 900. 1500.
 1.E10 3.E04 1.E04 8.E03 4.5E03 2.8E03 1.8E03 1.2E03 8.E02 5.E02
 5.E02 11.5 30000/
 3 / 293K

5 18 /
 6 /
 6 221 /
 0 / 600K
 3 / 3.221 /
 3.252 /
 3.452 /
 3.455 /
 5.455 /
 5.18 /
 6 /
 6 221 /
 0 / 1.500K
 3 / 3.221 /
 3.252 /
 3.452 /
 3.455 /
 5.455 /
 5.18 /
 6 /
 6 221 /
 0 /
 0 /
 WIMSR
 -25 27

24
 9228 1 0 0 1
 92 0 0 8.E02 1 11.5 221 0 1 000 /
 0.990 0.980 0.971 0.962 0.933 0.933 0.577 0.320 0.075 0.191
 STOP

B-17: URANIUM-236

0 /
 6 /
 MODER
 20 -21
 RECONR
 -21 -22

PENDF TAPE FOR U-236 FROM JENDL-3.2(F6) TAPE T313/
 9231 2/
 .002 0. 6 .04 1.E-07 /
 92-U-236 FROM JENDL-3.2(F6) TAPE T313/
 *PROCESSED BY THE NIOY91-V108 */
 0/
 BROADDR
 -22 -23

9231 3 0 1 0 /
 .003 1.E06 .06 1.E-07 /
 293. 600. 1500.

6 18/
 6 221 /
 0 /
 3 / 900.K
 3 221 /
 3 252 /
 3 452 /
 3 455 /
 5 18/
 5 455 /
 6 /
 6 18/
 6 221 /
 0 /
 3 / 1500.K
 3 221 /
 3 252 /
 3 452 /
 3 455 /
 5 18/
 5 455 /
 6 /
 6 18/
 6 221 /
 0 /
 0 /
 WIMSR
 -25.27
 2.4
 1.0 1
 9237
 92
 0.028 1.10.599 221.0 1.1.0 /
 0.990 0.980 0.971 0.962 0.953 0.933 0.577 0.320 0.320 0.075 0.191
 0.800 0.414 0.414 0.414 0.414 0.414 /
 STOP

 B-19: PLUTONIUM-238
 0 /
 6 /
 MODER
 20 -21
 RECONR
 -21 -22
 PENDF-TAPE FOR PU-238 FROM JENDL-3.2(F6)TAPE 314/
 9434.2/
 .002 0.6.04 1.E-07/
 94-PU-238 FROM JENDL-3.2(F6) TAPE T314/
 *PROCESSED BY THE NIJOY91-V108 */
 0 /
 BROADR
 -22 -23
 9434.3 0.0 0 /
 .003 1.E06 .06 1.E-07/
 293. 600. 1500.
 0 /

UNRESR
 -21 -23 -24
 9434.3 10 1
 293. 600. 1500.
 1.E10 8.E06 2.E06 1.5E06 1.E06 1.E05 1.E04 1.E03 1.E02 5.E01
 0 /
 THERMR
 0 -24 -26
 0.9434.12 3 1 0 1 221 1
 293. 600. 1500.
 0.005 2.1
 GROUFR
 -21 -26 0 -25
 9434.9 0 -5 1 3 10 1
 94-PU-238 FROM JENDL-3.2(F6)/
 293. 600. 1500.
 1.E10 8.E06 2.E06 1.5E06 1.E06 1.E05 1.E04 1.E03 1.E02 5.E01
 5.E02 28.53 30000/
 3 f 293K
 3 221 /
 3 252 /
 3 452 /
 5 18 /
 6 /
 6 221 /
 6 221 /
 0 /
 3 / 600K
 3 221 /
 3 252 /
 3 452 /
 6 /
 6 221 /
 6 221 /
 0 /
 WIMSR
 -25.27
 2.4
 1.0 1
 9434.4/
 0 /
 6 /
 MODER
 20 -21
 RECONR
 -21 -22
 PENDF-TAPE FOR PU-238 FROM JENDL-3.2(F6)TAPE 314/
 94
 0 0 1.E05 0 28.53 221.0 1.100 /
 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2
 STOP

 B-20: PLUTONIUM-239
 0 /
 6 /
 MODER

20-21 0 / 1500K
 RECONR 3 / 1500K
 -21-22 3 / 221 / 3 221 /
 PENDF TAPE FOR PU-239 FROM JENDL-3.2(F6)TAPE 314/
 9437 2/
 .002 0 .6 .04 1.E-07/
 94-PU-239 FROM JENDL-3.2(F6) TAPE T314/
 *PROCESSED BY THE NILOY91-V108 */
 0/
 BROADR 0 / 0 /
 -22 -23 0 / 0 /
 9437 4 0 0 0/
 .003 1.E06 .06 1.E-07/
 293. 600. 900. 1500.
 0/
 UNRESR 0 / 0 /
 -21 -23 -24 0 / 0 /
 9437 4 10 1/
 293. 600. 900. 1500.
 1.E10 1.5E06 1.E06 1.E05 1.E03 2.E03 1.E04 2.E02 5.E01 1.E01
 0/
 THERMR 0 -24 -26 0 /
 0 / 221 1 0 / 221 1
 0 / 900. 1500.
 0.005 2.1
 GROUFR 0 / 0 /
 -21 -26 0 -25 0 / 0 /
 9437 9 0 -5 14 10 1 0 / 0 /
 94-PU-239 FROM JENDL-3.2(F6)/
 293. 600. 900. 1500.
 1.E10 1.5E06 1.E06 1.E05 1.E04 2.E03 1.E03 1.E02 5.E01 1.E01
 2.5E03 7.97 30000/
 3 / 293K
 3 221 / 3 221 /
 3 252 / 3 252 /
 3 452 / 3 452 /
 3 455 / 3 455 /
 5 455 / 5 455 /
 5 18 / 5 18 /
 6 / 6 /
 6 221 / 6 221 /
 0 / 0 /
 3 / 3 /
 3 221 / 3 221 /
 3 252 / 3 252 /
 3 452 / 3 452 /
 3 455 / 3 455 /
 6 / 6 /
 6 221 / 6 221 /
 0 / 0 /
 3 / 3 /
 900K
 0 / 0 /
 3 / 221 / 3 221 /
 3 252 / 3 252 /
 3 452 / 3 452 /
 3 455 / 3 455 /
 6 / 6 /
 6 221 / 6 221 /
 0 / 0 /
 3 / 3 /
 1500K
 3 221 / 3 221 /
 3 252 / 3 252 /
 3 452 / 3 452 /
 3 455 / 3 455 /
 6 221 / 6 221 /
 0 / 0 /
 3 / 221 1 3 221 1
 293. 600. 900. 1500.
 0.005 2.1
 GROUPR 0 / 0 /
 -21 -26 0 -25 0 / 0 /
 9440 9 0 0 1.4 10 1 0 / 0 /
 94-PU-240 FROM JENDL-3.2(F6)TAPE T314/
 9440 2/
 .002 0 .6 .04 1.E-07 /
 94-PU-240 FROM JENDL-3.2(F6)TAPE T314/
 *PROCESSED BY THE NILOY91-V108 */
 0/
 UNRESR 0 / 0 /
 -22 -23 0 / 0 /
 9440 4 0 0 0 / 0 / 0 /
 .003 1.E06 .06 1.E-07 /
 293. 600. 900. 1500.
 0 / 0 /
 THERMR 0 -24 -26 0 /
 0 / 10 1 0 / 10 1
 0 / 900. 1500.
 0.005 2.1
 GROUPR 0 / 0 /
 -21 -26 0 -25 0 / 0 /
 9440 9 0 0 1.4 10 1 0 / 0 /
 94-PU-240 FROM JENDL-3.2(F6)/
 293. 600. 900. 1500.

1.E10 8.E06 2.E06 1.5E06 1.E05 1.E06 1.E03 1.E02 5.E01
 4.E03 1.6E4 30000/-28
 3 / 293K
 3.221 /
 3.222 /
 3.432 /
 3.455 /
 5.455 /
 5.18 /
 6/
 6.221 /
 0/
 3 / 600K
 3.221 /
 3.222 /
 3.452 /
 3.455 /
 6/
 6.221 /
 0/
 3 / 900K
 3.221 /
 3.252 /
 3.452 /
 3.455 /
 6/
 6.221 /
 0/
 3 / 1500K
 3.221 /
 3.252 /
 3.452 /
 3.455 /
 6/
 6.221 /
 0/
 0/
 WTMSR
 -25.27
 24.
 10.01
 9440
 94
 00.1.E04 1 1.644 22101100/
 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2
 0.2 0.2 0.2
 STOP

9443 2/
 .002 0.6 0.4 1.E-07/
 94-PU-241 FROM JENDL-3.2(F6) TAPE T314/
 0/
 BROADR
 -22.-23
 9443 4 0 0 0/
 .003 1.E06 .96 1.E-07/
 293. 600. 900. 1500.
 0/
 UNRESR
 -21.-23.-24
 9443 4 10 1/
 293. 600. 900. 1500.
 1.E10 1.5E06 1.E06 1.E05 1.E04 2.E03 1.E03 1.E02 5.E01 1.E01
 0/
 THERMR
 0-24.-26
 0 9443 12 4 1 0 1 221 1
 293. 600. 900. 1500.
 0.005 2.1
 GROUPR
 -21.-26 0.-25
 9443 9 0.5 14 10 1
 94-PU-241 FROM JENDL-3.2(F6)/
 293. 600. 900. 1500.
 1.E10 1.5E06 1.E06 1.E05 1.E04 2.E03 1.E03 1.E02 5.E01 1.E01
 3.E02 11.35 30000/
 3 / 293K
 3.221 /
 3.252 /
 3.452 /
 3.455 /
 3.455 /
 5.455 /
 5.18 /
 6/
 6.221 /
 0/
 3 / 600K
 3.221 /
 3.252 /
 3.452 /
 3.455 /
 6/
 6.221 /
 0/
 0/
 WTMSR
 -25.27
 24.
 10.01
 9440
 94
 00.1.E04 1 1.644 22101100/
 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2
 0.2 0.2 0.2
 STOP

B.22: PLUTONIUM-241
 0 /
 6 /
 MODER
 20.-21.
 RECONR
 -21.-22
 PENDF TAPE FOR PU-241 FROM JENDL-3.2(F6) TAPE T314/
 3 / 900K
 3.221 /
 3.252 /
 3.452 /
 3.455 /
 6/
 6.221 /
 0/
 3 / 1500K
 3.221 /
 3.252 /

3.452 /
 3.455 /
 6 /
 6.221 /
 0 /
 WIMSR
 .25.27
 2.4
 1.0.0 1
 9443
 94
 0.02.E03 1 11.35 221.01000 /
 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2
 0.2 0.2 0.2
 STOP

B-23: PLUTONIUM-242

0 /
 6 /
 MODER
 RECONR
 -21.-22.
 PENDFTAPE FOR PU-242 FROM JENDL-3.2(F6)TAPE 314/
 9446.2/
 002 0. 6. 04 1.E-07/
 94-PU-242 FROM JENDL-3.2(F6) TAPE T314/
 *PROCESSED BY THE NIJOY91-V108 */
 0/
 BROADR
 -22.-23
 9446.4 0 0 0 /
 .003 1.E06 .06 1.E-07/
 293. 600. 900. 1500.
 0/
 UNRESR
 -21.-23.-24
 9446.4 10 1
 293. 600. 900. 1500.
 1.E10 8.E06 2.E06 1.5E06 1.E06 1.E05 1.E04 1.E03 1.E02 5.E01
 0/
 THERMR
 0.-24.-26
 0.9446.12. 4 1.0 1.221 1
 293. 600. 900. 1500.
 0.005 2.1
 GROUPR
 -21.-26. 0.-25
 9446.9 0 0 1.4 10 1
 94-PU-242 FROM JENDL-3.2(F6)/
 293. 600. 900. 1500.
 1.E10 8.E06 2.E06 1.5E06 1.E06 1.E05 1.E04 1.E03 1.E02 5.E01
 1.15E3 8.32 300000 /-28
 3 / 293K
 3.221 /

B-24: AMERICIUM-241

0 /
 3 / 1500K
 3.221 /
 3.252 /
 3.452 /
 6 /
 6.221 /
 0 /
 WIMSR
 -25.27
 2.4
 1.0 0 1
 9446
 94
 0.0 1.E05 1 8.32 221.01100 /
 .19 .19 .19 .19 .19 .19 .19 .19 .19 .19 .19 .19 .19 .19 .19 .19
 19. 19. 19
 STOP
 0/
 PENDFTAPE FOR AM-241 FROM JENDL-3.2(F6) TAPE 313/
 9543.2/
 .002 0. 6. 04 1.E-07/
 PENDFTAPE FOR AM-241 FROM JENDL-3.2(F6) TAPE T313/
 *PROCESSED BY THE NIJOY91-V108 */
 0/
 BROADR
 -22.-23
 9543.3 0 0 0 /
 .003 1.E06 .06 1.E-07/

APPENDIX -C: INPUT SPECIFICATIONS OF WIMS-D/4.1 AND SRAC95
FOR THE DIFFERENT BENCHMARK LATTICE

```

293. 600. 1500.
0/
*UNRELR*
-21.-23.-24
9543.3 10.1
293. 600. 1500.
1.E10. 8.E06. 2.E06. 1.5E06. 1.E06. 1.E05. 1.E04. 1.E03. 1.E03. 1.E02. 5.E01
0/
*THERMR*
0.-24.-26 /ADD THERMAL SCATTERING DATA TO UNIT-26
95.-AM-24 /FROM JENDL-3.2(F6)*/
```

293. 600. 1500.

0.005 2.1

GROUPR

-21.-26 0.-25

9543.9 0.-5 1 3 10 1

1.E10. 8.E06. 2.E06. 1.5E06. 1.E06. 1.E05. 1.E04. 1.E03. 1.E03. 1.E02. 5.E01

1.5E02. 11.13 30000/

3./ 293.K

3.221./

3.252./

3.432./

5 18./

6./

6 18./

6 221./

0/
3./ 600.K

3.221./

3.252./

3.432./

6./

6 18./

6 221./

0/
0/
3./ 1500.K

3.221./

3.252./

3.432./

6./

6 18./

6 221./

0/
0/
WIMSR

-25.27

24

1 0 0 1

9543

95

0.0 1.E06 1 11.13 221 0 1 100/

0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2 0.2

STOP

C-1: WIMS INPUT FOR TRX-1

```
*****  
* TRX-1 SEQUENCE 1 S12 REGULAR 1 6  
* STAND INPUT NMETERIAL 3  
*****
```

CELL 6

SEQUENCE 1

NGROUP 69 2

NMESH 10

NREGION 4 0 4

NMATERIAL 3

NREACT 2

PREFCUT

INITIATE

ANNULUS 1 0.4915 1

ANNULUS 2 0.5042 0

ANNULUS 3 0.5753 2

ANNULUS 4 0.94822 3

FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 \$

19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 \$

37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 \$

55 56 57 58 59 60 61 62 63 64 65 66 67 68 69

MESH 4 1 1 4

MATERIAL 1.-1 293.0 1 9228.1 0.0006253 9237.1 0.047205

MATERIAL 2.-1 293.0 2 1325.0 0.06025

MATERIAL 3.-1 293.0 3 125.0 0.06676 825.0 0.03338

REGULAR 1 6

S 12

BEGINC

THERMAL 24

BEFORE 1

DNB 1 0. 0. 0.

DNB 2 0. 0. 0.

DNB 3 0.06676 0. 0.03338 0.

BUCKLING 0.005174 0.000526

DIFFUSION 1 3 1

LEAKAGE 5

REACTION (9228.293.0)(9237.293.0)

PARTITION 45 69

BEGINC

C-2: WIMS INPUT FOR TRX-2

```
*****  
* TRX-2 SEQUENCE 1 S12 REGULAR 1 6  
* STAND INPUT NMETERIAL 3  
*****
```

CELL 6

SEQUENCE 1

NGROUP 69 2

NMESH 14

NREGION 4 0 4

```

NMATERIAL 3
NREACT 2
PREOUT
INITIATE
  ANNULUS 1 0.4915 1
  ANNULUS 2 0.5042 0
  ANNULUS 3 0.5753 2
ANNULUS 4 1.1414 3
FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 $
19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 $
37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 $
55 56 57 58 59 60 61 62 63 64 65 66 67 68 69
MESH 4 1 1 8
MATERIAL 1 -1 293.0 1 9228.1 0.0006253 9237.1 0.047205
MATERIAL 2 -1 293.0 2 1325 0.06025
MATERIAL 3 -1 293.0 3 125 0.06676 825 0.03338
REGULAR 1 6
S 12
BEGINC
THERMAL 24
BEEONE 1
DNB 1.0. 0.0. 0.
DNB 2.0. 0.0. 0.
DNB 3.0.06676 0. 0.03338 0.
BUCKLING 0.0002734 0.000525
DIFFUSION 1 3 1
LEAKAGE 5
REACTION (9228.293.0) (9237.293.0)
PARTITION 45 69
BEGINC

C-4: WIMS INPUT FOR BAPl-2

***** * *****
* BAPl-UO2-2 SEQUENCE 1 S 12 REGULAR 1 6
* 09.05.96 STAND.INPUT MMETEIRAL 3
***** * *****
CELL 6
SEQUENCE 1
NGROUP 69 2
NMESH 10
NREGION 4 0 4
NMATERIAL 3
NREACT 2
PREOUT
INITIATE
  ANNULUS 1 0.4864 1
  ANNULUS 2 0.5042 0
  ANNULUS 3 0.5753 2
ANNULUS 4 0.8179 3
FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 $
19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 $
37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 $
55 56 57 58 59 60 61 62 63 64 65 66 67 68 69
MESH 4 1 1 4
MATERIAL 1 -1 293.0 1 9228.1 0.0003112 9237.1 0.023127 825 0.046946
MATERIAL 2 -1 293.0 2 1325 0.06025
REGULAR 1 6
S 12
BEGINC
THERMAL 24
BEEONE 1
DNB 1.0. 0.0. 0.
DNB 2.0. 0.0. 0.
DNB 3.0.06676 0. 0.03338 0.
BUCKLING 0.0003018 0.000529
DIFFUSION 1 3 1
LEAKAGE 5
REACTION (9228.293.0) (9237.293.0)
PARTITION 45 69
BEGINC

```

C-5: WIMS INPUT FOR BAPl-3

```
*****  
* BAPl-JC2-3 SEQUENCE 1 S12 REGULAR 16  
* 03.04.96 STAND. INPUT MMETERIAL 3  
*****  
CELL 6  
SEQUENCE 1  
NGROUP 69 2  
NMESH 10  
NREGION 4 0 4  
NMATERIAL 3  
NREACT 2  
PREOUT  
INITIATE  
ANNULUS 1 0.4864 1  
ANNULUS 2 0.5042 0  
ANNULUS 3 0.5753 2  
ANNULUS 4 0.94806 3  
FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 $  
19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 $  
37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 $  
55 56 57 58 59 60 61 62 63 64 65 66 67 68 69  
MESH 8 1 3 8  
MATERIAL 1 -1 300.0 1 9228.1 6.08E-04 $  
9237.1 2.25E-02 825 4.725E-02  
MATERIAL 2 -1 300.0 2 1325 5.587E-02  
MATERIAL 3 -1 300.0 3 125 6.676E-02 825 3.338E-02  
REGULAR 1  
S 12  
BEGINC  
THERMAL 24  
BEEONE 1  
DNB 1.0. 0.0. 0.  
DNB 2.0. 0.0. 0.  
DNB 3.0.06676.0.0.03338.0.  
BUCKLING 0.0079139 0.000786096  
DIFFUSION 1 3 1  
LEAKAGE 5  
BEGINC  
C-7: WIMS INPUT FOR TCA 183U  
*****  
* TCA183U SEQUENCE 1 S12 REGULAR 16  
* 10.06.96 STAND. INPUT MMETERIAL 3  
*****  
CELL 6  
SEQUENCE 1  
NGROUP 69 2  
NMESH 20  
NREGION 4 0 4  
NMATERIAL 3  
PREOUT  
INITIATE  
ANNULUS 1 0.625 1  
ANNULUS 2 0.6325 0  
ANNULUS 3 0.7085 2  
ANNULUS 4 1.103583 3  
FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 $  
19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 $  
37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 $  
55 56 57 58 59 60 61 62 63 64 65 66 67 68 69  
MESH 8 1 3 8  
MATERIAL 1 -1 300.0 1 9228.1 6.08E-04 $  
9237.1 2.25E-02 825 4.725E-02  
MATERIAL 2 -1 300.0 2 1325 5.587E-02  
MATERIAL 3 -1 300.0 3 125 6.676E-02 825 3.338E-02  
REGULAR 1  
S 12  
BEGINC  
THERMAL 24
```

C-6: WIMS INPUT FOR TCA 150U

```
*****  
* TCA150U SEQUENCE 1 S12 REGULAR 16  
* 10.06.96 STAND. INPUT MMETERIAL 3  
*****  
CELL 6  
SEQUENCE 1  
NGROUP 69 2  
NMESH 20  
NREGION 4 0 4  
NMATERIAL 3  
PREOUT  
INITIATE
```

BEEONE 1
 DNB 1.0. 0.0. 0.
 DNB 2.0. 0.0. 0.
 DNB 3.0.06676.0. 0.03338.0.
 BUCKLINGS 0.00885495 0.00047504
 DIFFUSION 1 3 1
 LEAKAGE 5
 BEGINC

CELL 6
 SEQUENCE 1
 NGROUP 69 2
 NMESH 20
 NREGION 4 0 4
 NMATERIAL 3
 PREOUT
 INITIATE
 ANNULUS 1.0.625 1
 ANNULUS 2.0.6325 0
 ANNULUS 3.0.7085 2
 ANNULUS 4.1.2937197 3
 FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 \$
 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 \$
 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 \$
 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69

MESH 8 1 3 8
 MATERIAL 1 -1 300.0.1 9228.1 6.086E-04 \$
 - 9237.1 2.255E-02 825.4.725E-02
 MATERIAL 2 -1 300.0.2 1325 5.587E-02
 MATERIAL 3 -1 300.0.3 125 6.676E-02 825 3.338E-02
 REGULAR 1
 S 12

BEGINC
 THERMAL 24
 BEEONE 1
 DNB 1.0. 0.0. 0.
 DNB 2.0. 0.0. 0.
 DNB 3.0.06676.0. 0.03338.0.
 BUCKLINGS 0.00872856 0.000951431
 DIFFUSION 1 3 1
 LEAKAGE 5
 BEGINC

 C-8: WIMS INPUT FOR TCA 248U

 * TCA248U SEQUENCE 1 S12 REGULAR 1 6
 * 10.06.96 STAND. INPUT MMETERIAL 3

 CELL 6
 SEQUENCE 1
 NGROUP 69 2
 NMESH 20
 NREGION 4 0 4
 NMATERIAL 3
 PREOUT
 INITIATE
 ANNULUS 1.0.625 1
 ANNULUS 2.0.6325 0
 ANNULUS 3.0.7085 2
 ANNULUS 4.1.2130386 3
 FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 \$
 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 \$
 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 \$
 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69

MESH 8 1 3 8
 MATERIAL 1 -1 300.0.1 9228.1 6.086E-04 \$
 - 9237.1 2.255E-02 825.4.725E-02
 MATERIAL 2 -1 300.0.2 1325 5.587E-02
 MATERIAL 3 -1 300.0.3 125 6.676E-02 825 3.338E-02
 REGULAR 1
 S 12

BEGINC
 THERMAL 24
 BEEONE 1
 DNB 1.0. 0.0. 0.
 DNB 2.0. 0.0. 0.
 DNB 3.0.06676.0. 0.03338.0.
 BUCKLINGS 0.00867071 0.00121928
 DIFFUSION 1 3 1
 LEAKAGE 5
 BEGINC

 C-10: WIMS INPUT FOR TCA 242PU

 * TCA242 SEQUENCE 1 S12 REGULAR 1
 * 30.05.96 STAND. INPUT MMETERIAL 3

 CELL 6
 SEQUENCE 1
 NGROUP 69 2
 NMESH 20
 NREGION 4 0 4
 NMATERIAL 3
 PREOUT
 INITIATE
 ANNULUS 1.0.5325 1
 ANNULUS 2.0.5415 0
 ANNULUS 3.0.6115 2
 ANNULUS 4.1.0296723 3
 FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 \$
 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 \$
 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 \$
 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69

MESH 8 1 3 8

 C-9: WIMS INPUT FOR TCA 300U

 * TCA300U SEQUENCE 1 S12 REGULAR 1 6
 * 10.06.96 STAND. INPUT MMETERIAL 3

MATERIAL 1 -1 300.0 1 9225.1 7.436E-07 9228.1 9.393E-05 \$
 9237.1 1.295E-02 9434.1 2.0E-06 9437.1 2.749E-04 \$
 9440.1 8.843E-05 9443.1 2.819E-06 9446.1 8.124E-06 \$
 9543.1 1.059E-06 825 2.784E-02
 MATERIAL 2 -1 300.0 2 4000 3.84E-02
 MATERIAL 3 -1 300.0 3 125 6.676E-02 825 3.338E-02
 REGULAR 1
 S 12
 BEGINC
 THERMAL 24
 BEFORNE 1
 DNB 1.0. 0.0. 0.
 DNB 2.0. 0.0. 0.
 DNB 3.0.6676.0. 0.03338.0.
 BUCKLINGGS 0.00660895 0.00147104
 DIFFUSION 1 3 1
 LEAKAGE 5
 BEGINC

C-11: WIMS INPUT FOR TCA 298PU

 * TCA2.98PU SEQUENCE 1 S12 REGULAR 1 6
 * 07.05.96 STAND. INPUT MMETERIAL 3

CELL 6
 PREOUT
 INITIATE
 ANNULUS 1 0.5325 1
 ANNULUS 2 0.5415 0
 ANNULUS 3 0.6115 2
 ANNULUS 4 1.2553539 3
 FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 \$
 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 5
 * 07.05.96 STAND. INPUT MMETERIAL 3

SEQUENCE 1
 NGROUP 69 2
 NMESH 20
 NREGION 4 0 4
 NMATERIAL 3
 PREOUT
 MESH 8 1 3 8
 MATERIAL 1 -1 300.0 1 9225.1 7.436E-07 9228.1 9.393E-05 \$
 9237.1 1.295E-02 9434.1 2.0E-06 9437.1 2.749E-04 \$
 9440.1 8.843E-05 9443.1 2.819E-05 9446.1 8.124E-06 \$
 9543.1 1.059E-06 825 2.784E-02
 MATERIAL 2 -1 300.0 2 4000 3.84E-02
 MATERIAL 3 -1 300.0 3 125 6.676E-02 825 3.338E-02
 REGULAR 1
 S 12
 BEGINC
 THERMAL 24
 BEFORNE 1
 DNB 1.0. 0.0. 0.
 DNB 2.0. 0.0. 0.
 DNB 3.0.6676.0. 0.03338.0.
 BUCKLINGGS 0.00612493 0.00166506
 DIFFUSION 1 3 1
 LEAKAGE 5
 BEGINC

C-12: WIMS INPUT FOR TCA 424PU

 * TCA4.24PU SEQUENCE 1 S12 REGULAR 1 6
 * 07.05.96 STAND. INPUT MMETERIAL 3

CELL 6
 SEQUENCE 1
 NGROUP 69 2
 NMESH 20
 NREGION 4 0 4
 NMATERIAL 3
 PREOUT
 INITIATE
 ANNULUS 1 0.5325 1
 ANNULUS 2 0.5415 0
 ANNULUS 3 0.6115 2
 ANNULUS 4 1.2553539 3
 FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 \$
 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 5
 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 \$
 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69
 MESH 8 1 3 8
 MATERIAL 1 -1 300.0 1 9225.1 7.436E-07 9228.1 9.393E-05 \$
 9237.1 1.295E-02 9434.1 2.0E-06 9437.1 2.749E-04 \$
 9440.1 8.843E-05 9443.1 2.819E-05 9446.1 8.124E-06 \$
 9543.1 1.059E-06 825 2.784E-02
 MATERIAL 2 -1 300.0 2 4000 3.84E-02
 MATERIAL 3 -1 300.0 3 125 6.676E-02 825 3.338E-02
 REGULAR 1
 S 12
 BEGINC
 THERMAL 24
 BEFORNE 1
 DNB 1.0. 0.0. 0.
 DNB 2.0. 0.0. 0.
 DNB 3.0.6676.0. 0.03338.0.
 BUCKLINGGS 0.00670258 0.00157741

C-13: WIMS INPUT FOR TCA 555PU

 * TCA5.55PU SEQUENCE 1 S12 REGULAR 1 6
 * 07.05.96 STAND. INPUT MMETERIAL 3

CELL 6
 SEQUENCE 1
 NGROUP 69 2
 NMESH 20

```

NREGION 4 0 4
NMATERIAL 3
PREOUT
INITATE
  ANNULUS 1.0.5325 1
  ANNULUS 2.0.5415 0
  ANNULUS 3.0.6115 2
  ANNULUS 4.1.39584.06 3
  FEWGROUP 1.2.3.4.5.6.7.8.9.10.11.12.13.14.15.16.17.18 $  

  19.20.21.22.23.24.25.26.27.28.29.30.31.32.33.34.35.36 $  

  37.38.39.40.41.42.43.44.45.46.47.48.49.50.51.52.53.54 $  

  55.56.57.58.59.60.61.62.63.64.65.66.67.68.69
MESH .8 1 3 8
MATERIAL 1.-1 300.0 1 9225.1 7.436E-07 9228.1 9.393E-05 $  

  9237.1 1.295E-02 9434.1 2.0E-06 9437.1 2.749E-04 $  

  9440.1 8.843E-05 9443.1 2.8119E-05 9446.1 8.124E-06 $  

  9543.1 1.059E-06 825.2.784E-02
MATERIAL 2.-1 300.0 2 4000 3.84E-02
MATERIAL 3.-1 300.0 3 125 6.676E-02 825 3.338E-02
REGULAR 1
S 12
BEGINC
THERMAL 24
BEEONE 1
  DNB 1.0. 0.0. 0.
  DNB 2.0. 0.0. 0.
  DNB 3.0.6676.0. 0.03338.0.
BUCKLINGS 0.00486039 0.0016496
DIFFUSION 1 3 1
LEAKAGE 5
BEGINC

```

C-14: WIMS INPUT FOR NEA-V6E8

```

INITATE
  ANNULUS 1.0.41 1
  ANNULUS 2.0.475 2
  ANNULUS 3.0.5713996.3
  FEWGROUP 1.2.3.4.5.6.7.8.9.10.11.12.13.14.15.16.17.18 $  

  19.20.21.22.23.24.25.26.27.28.29.30.31.32.33.34.35.36 $  

  37.38.39.40.41.42.43.44.45.46.47.48.49.50.51.52.53.54 $  

  55.56.57.58.59.60.61.62.63.64.65.66.67.68.69
MESH .8 4 8
MATERIAL 1.-1 900.0 1 9228.1 6.194E-05 $
  9237.1 2.022E-02 9437.1 1.367E-03 $
  9440.1 6.009E-04 9443.1 2.418E-04 9446.1 1.844E-04 $
  825 4.608E-02
MATERIAL 2.-1 600.0 2 4000 3.702E-02
MATERIAL 3.-1 600.0 3 125 4.744E-02 825 2.372E-02
REGULAR 16
S 12
BEGINC
BEGINC

```

C-15: WIMS INPUT FOR NEA-V1E7

```

*****  

* NEA-V1E7 SEQUENCE 1 S12 REGULAR 16  

* 30.05.96 STAND. INPUT MMATERIAL 3  

*****  

CELL 6
SEQUENCE 1
NGROUP 69 2
NMESH 20
NREGION 3 0 3
NMATERIAL 3
PREOUT
INITATE
  ANNULUS 1.0.41 1
  ANNULUS 2.0.475 2
  ANNULUS 3.0.6407572.3
  FEWGROUP 1.2.3.4.5.6.7.8.9.10.11.12.13.14.15.16.17.18 $  

  19.20.21.22.23.24.25.26.27.28.29.30.31.32.33.34.35.36 $  

  37.38.39.40.41.42.43.44.45.46.47.48.49.50.51.52.53.54 $  

  55.56.57.58.59.60.61.62.63.64.65.66.67.68.69
MESH .8 4 8
MATERIAL 1.-1 900.0 1 9228.1 6.194E-05 $
  9237.1 2.038E-02 9437.1 1.367E-03 $
  9440.1 6.009E-04 9443.1 2.418E-04 9446.1 1.844E-04 $
  825 4.608E-02
MATERIAL 2.-1 600.0 2 4000 3.702E-02
MATERIAL 3.-1 600.0 3 125 4.744E-02 825 2.372E-02
REGULAR 16
S 12
BEGINC
BEGINC

```

C-16: WIMS INPUT FOR NEA-A

```

*****  

* NEAA SEQUENCE 1 S12 REGULAR 16  

* 30.05.96 STAND. INPUT MMATERIAL 3  

*****  

CELL 6
SEQUENCE 1
NGROUP 69
NMESH 20
NREGION 3 0 3
NMATERIAL 3
PREOUT
INITATE
  ANNULUS 1.0.41 1
  ANNULUS 2.0.475 2
  ANNULUS 3.0.5713996.3
  FEWGROUP 1.2.3.4.5.6.7.8.9.10.11.12.13.14.15.16.17.18 $  

  19.20.21.22.23.24.25.26.27.28.29.30.31.32.33.34.35.36 $  

  37.38.39.40.41.42.43.44.45.46.47.48.49.50.51.52.53.54 $  

  55.56.57.58.59.60.61.62.63.64.65.66.67.68.69
MESH .8 4 8
MATERIAL 1.-1 900.0 1 9228.1 6.094E-05 $
  9237.1 2.022E-02 9437.1 1.563E-03 $
  9440.1 6.872E-04 9443.1 2.765E-04 $
INITATE

```

C-18: SRAC95 INPUT FOR TRX-1

```

ANNULUS 1.04095 1
ANNULUS 2.04750 2
ANNULUS 3.074096 3
FEWGROUP 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 $ 06951000
19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 $ 00953000
37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 $ 00954000
55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 00955000
MESH 8 4 8 00956000
MATERIAL 1-1 933.0 1 9228.1 1.1445E-04 $ 00957000
9227.1 1.9939E-02 9434.1 1.467E-04 9437.1 1.0285E-03 $ 00958000
9440.1 7.9657E-04 9443.1 3.3997E-04 $ 00959000
9446.1 5.6388E-04 825.4 5.6354E-02 00959100
MATERIAL 2-1 579.3 2 4000 4.7716E-02 825.2 3.858E-02 $ 00959200
MATERIAL 3-1 579.3 3 125 4.7716E-02 825.1 6.6226E-05 00959300
$25 3.6346E-06 528 1.6226E-05 00959400
REGULAR 1 6 00959500
S 12 00959600
BEGINC / 00959700
BEGINC / 00959800
46(1) / 00959900
6663.1 16000 5010290 0600 / 00960000
0100.5055.1 0.00001 0.0001 1.00 100.0.1 / 00960100
11123.3 / T.R 00960200
1 1/R.X 00960300
1 2.3/R.M 00960400
0.0 0.284.40.04915 0.5753 0.75 0.9030 / RX 00960500
&31 3 1 / PLOT FOR PU 00960600
3 / NMAT 00960700
FUELXMX 0 2 293.0 0.9830 0.0 / 1 00960800
XU050001 2 1 6.2530E-04 00960900
XU080001 2 1 4.7205E-02 00961000
CLADMX2X 0 1 293.0 0.1676 0.0 / 2 00961100
XAL70001 2 1 5.1727E-02 00961200
COOLXMX 0 2 293.0 2.000 0.0 / 3 00961300
XH01H001 0 1 6.6760E-02 00961400
XC060001 0 1 3.3380E-02 00961500
0 0 1 0 / OPT(1:3),MREC <-> REACTION RATE >> + 00961600
1 1 2 1 1 1 70(1.0) 0.7047902 36(0.0) / MPOS(1,235,L238,IX,LY,IX,FGS 00961700
0 / PLOT FOR PEACO

```

C-19: SRAC95 INPUT FOR TRX-2

```

TRX2
TRX-2 BENCHMARK PROBLEM.
1 1 1 2 1 4 3 -2 0 0 0 0 2 0 1 1 0 0
0.005459
J9347.PFASTJ32.DATA OLD FILE 01110000
J9347.PTHMLJ32.DATA OLD FILE 01120000
J9347.PMCRSJ32.DATA OLD FILE 01130000
J9062.FASTJDATA SCR CORE 01140000
J9062.THERMALJDATA SCR CORE 01150000
J9062.UMCROSS.DATA SCR CORE 01160000
J9062.MACROWRK.DATA SCR CORE 01170000
J9062.MACRO.DATA SCR CORE 01180000
J9062.FLUX.DATA SCR CORE 01190000
J9062.MICREF.DATA SCR CORE 01200000
61 46 0 0 /

```

```

61(1)   /          0 0 3.3380E-02      000000550
46(1)   /          0 1 0 / IOPT(1,3),MREC ----- << REACTION RATE >> -----
1 1 2 1 1 7(1,0) 0.7047902 36(0,0) / MPOSIL235,L238,IY,IY,FGS
0/ PEACO

```

C-21: SRAC95 INPUT FOR BAPl-2

```

3          00000010
BAPl2          BAPl-2 BENCHMARK PROBLEM. THERMAL CUT IS 2.3824 EV :
1 1 1 2 1 4 3 -2 0 0 0 0 2 2 1 1 0 0      00000020
0.003547      00000030
J9347.PFASTJ32.DATA    OLD FILE      01110000
J9347.PTHMLJ32.DATA    OLD FILE      01120000
J9347.PMCRSJ32.DATA    OLD FILE      01130000
J9062.FASTUDATA        SCR CORE      01140000
J9062.THERMALUDATA    SCR CORE      01150000
J9062.UMCROSS.DATA    SCR CORE      01160000
J9062.MACROWK.DATA    SCR CORE      01170000
J9062.MACRODATA       SCR CORE      01180000
J9062.FLUX.DATA        SCR CORE      01190000
J9062.MICREF.DATA     SCR CORE      01200000
61 46 0 0 /
61(1)   /
46(1)   /

```

C-20: SRAC95 INPUT FOR BAPl-1

```

BAPl1          00000010
BAPl-1 BENCHMARK PROBLEM. THERMAL CUT IS 2.3824 EV :
1 1 1 2 1 4 3 -2 0 0 0 0 2 2 1 1 0 0      00000020
0.003259      00000030
0.003259      00000040
J9347.PFASTJ32.DATA    OLD FILE      01110000
J9347.PTHMLJ32.DATA    OLD FILE      01120000
J9347.PMCRSJ32.DATA    OLD FILE      01130000
J9062.FASTUDATA        SCR CORE      01140000
J9062.THERMALUDATA    SCR CORE      01150000
J9062.UMCROSS.DATA    SCR CORE      01160000
J9062.MACROWK.DATA    SCR CORE      01170000
J9062.MACRODATA       SCR CORE      01180000
J9062.FLUX.DATA        SCR CORE      01190000
J9062.MICREF.DATA     SCR CORE      01200000
61 46 0 0 /
61(1)   /
46(1)   /

```

C-22: SRAC95 INPUT FOR BAPl-3

```

3          00000010
BAPl3          BAPl-3 BENCHMARK PROBLEM. THERMAL CUT IS 2.3824 EV :
1 1 1 2 1 4 3 -2 0 0 0 0 2 2 1 1 0 0      00000020
0.003472      00000030
J9347.PFASTJ32.DATA    OLD FILE      01110000
J9347.PTHMLJ32.DATA    OLD FILE      01120000
J9347.PMCRSJ32.DATA    OLD FILE      01130000
J9062.FASTUDATA        SCR CORE      01140000
J9062.THERMALUDATA    SCR CORE      01150000
J9062.UMCROSS.DATA    SCR CORE      01160000
J9062.MACROWK.DATA    SCR CORE      01170000
J9062.MACRODATA       SCR CORE      01180000
J9062.FLUX.DATA        SCR CORE      01190000
J9062.MICREF.DATA     SCR CORE      01200000
0 0 1 0 / IOPT(1,3),MREC ----- << REACTION RATE >> -----
1 1 2 1 1 7(1,0) 0.7047902 36(0,0) / MPOSIL235,L238,IY,IY,FGS
0/ PEACO

```

J9062.UMCROSS.DATA SCR CORE 01160000
J9062.MACROWRK.DATA SCR CORE 01170000
J9062.MACRO.DATA SCR CORE 01180000
J9062.FLUX.DATA SCR CORE 01190000
J9062.MICREF.DATA SCR CORE 01200000
61 46 0 /
61(1) /
66 64 1 16000 5010290 050 0
0100505550 0.00001 0.0001 0.001 1.0 1000 0.0 1
00000240
00000230
00000240
00000250

T18U
1 1 1 1 / R-X
1 2 3 3 / R-M
0.0 0.284 0.40 0.4864 0.5733 0.77 0.90285 / RX
3 / NMAT
U023X31X 0 3 293 0 0.9728 0.0 / 1
XU050001 2 1 3.1120E-04
XU080001 2 1 2.3127E-02
X0060001 0 0 4.6946E-02
CLD3X32X 0 1 293.0 0.1778 0.0 / 3
XAL70001 0 0 4.89943E-02
MOD3X33X 0 2 293.0 2.000 0.0 / 4
XH01H001 0 0 6.6760E-02
X0060001 0 0 3.3380E-02
0 0 1 0 / IOPT(1,3),MREC ----- << REACTION RATE >> ----->
1 1 2 1 1 1 70(1.0) 0.7047902 36(0.0) / MPOS(1,235,L,238,1X,TX,FX,FGS
0/PEACO
00000650

C-23: SRAC95 INPUT FOR TCA 150U

T15U
TCA 1.50U BENCHMARK PROBLEM : PICTH 1.849 CM : 24X24 : B2 = 0.008700 00013600
1 1 1 1 2 1 4 3 -2 0 0 0 0 0 2 2 1 0 0 0
8.70000E-3
J9347.PFAST132..DATA OLD FILE
J9347.PTHMLJ32..DATA OLD FILE
J9062.FAST132..DATA SCR CORE
J9062.THERMAL132..DATA SCR CORE
J9062.UMCROSS..DATA SCR CORE
J9062.MACROWRK..DATA SCR CORE
J9062.MACRO..DATA SCR CORE
J9062.FLUX..DATA SCR CORE
J9062.MICREF..DATA SCR CORE
61 46 0 /
61(1) /
46(1) /

4.8 8 1 18000 506230 045 0
0100505550 0.00001 0.0001 0.001 1.0 1000 0.0 1
00011300
00011260
00011100
00011100

C-24: SRAC95 INPUT FOR TCA 183U

T18U
TCA 1.83U BENCHMARK PROBLEM : PICTH 1.956 CM : 22X22 : B2 = 0.009330 00013600
1 1 1 1 2 1 4 3 -2 0 0 0 0 0 2 2 1 0 0 0
9.33000E-3
J9347.PFAST132..DATA OLD FILE
J9347.PTHMLJ32..DATA OLD FILE
J9347.PMCRS132..DATA OLD FILE
J9062.FAST132..DATA SCR CORE
J9062.THERMAL132..DATA SCR CORE
J9062.UMCROSS..DATA SCR CORE
J9062.MACROWRK..DATA SCR CORE
J9062.MACRO..DATA SCR CORE
J9062.FLUX..DATA SCR CORE
J9062.MICREF..DATA SCR CORE
61 46 0 /
61(1) /
46(1) /

4.9 9 9 1 19000 506230 045 0
0100505550 0.00001 0.0001 1.0 1000 0.0 1
00011300
00011260
00011100

C-25: SRAC95 INPUT FOR TCA 248U

T24U
TCA 2.48U BENCHMARK PROBLEM : PICTH 2.150 CM : 20X20 : B2 = 0.009890 00013600
1 1 1 1 2 1 4 3 -2 0 0 0 0 0 2 2 1 0 0 0
9.89000E-3
J9347.PFAST132..DATA OLD FILE
J9347.PTHMLJ32..DATA OLD FILE
PIN2XP2X 0 3 300.0 1.2500 0.0 / 1
XU050001 2 0 6.08600E-04
XU080001 2 0 2.25500E-02
X0060001 0 0 4.72500E-02
CLD2X2CX 0 1 300.0 0.1670 0.0 / 2
XAL70001 2 0 5.58700E-02
MOD2XM2X 0 2 300.0 2.000 0.0 / 3
XH01H001 0 0 6.6760E-02
X0060001 0 0 3.3380E-02
0/PEACO
00011300

C-28: SRAC95 INPUT FOR TCA 298PU

T298
TCA 2.98PU BENCHMARK PROBLEM. USING CRITICAL BUCKLING = 0.00828
1 1 1 2 1 4 3 -2 0 0 0 0 2 0 1 0 0 0
0.0082800

J9347.PFASTJ32.DATA OLD FILE 01110000
J9347.PTHMLJ32.DATA OLD FILE 01120000
J9347.PMCRSJ32.DATA OLD FILE 01130000
J9062.FASTU.DATA SCR CORE 01140000
J9062.THERMALU.DATA SCR CORE 01150000
J9062.UMCROSS.DATA SCR CORE 01160000
J9062.MACROWRK.DATA SCR CORE 01170000
J9062.MACRO.DATA SCR CORE 01180000
J9062.FLUX.DATA SCR CORE 01190000
J9062.MICREF.DATA SCR CORE 01200000
61 46 0 /
6(1) /
46(1) /

4 9 9 1 1 9 0 0 0 5 0 6 2 3 0 0 4 5 0
0 1 0 0 5 0 5 5 0 0 0 0 0 1 0 0 0 0 0 0 1
9(1) /R-X
1 1 1 2 3 3 3 3 /R-M
0.0 0.3074/0.4348 0.5325 0.6115 0.78378 0.9245 1.0465 1.1556 1.1125/RX
3

PIN2XP2X 0 10 300.0 1.0650 0.0 / 1
XU040001 2 0 7.4360E-07
XU050001 2 0 9.3920E-05
XU080001 2 0 1.2950E-02
XPUS0001 2 0 2.0000E-06
XPU9001 2 0 2.7490E-04
XPUS0001 2 0 8.8430E-05
XPU10001 2 0 2.8190E-05
XPUS0001 2 0 8.1240E-06
XAM1001 2 0 1.0590E-06
XO060001 0 0 2.7840E-02
CLD2XC2X 0 1 300.0 0.1380 0.0 / 3
XZRN0001 2 0 3.8400E-02
MOD2XM2X 0 2 300.0 2.000 0.0 / 4
XH01H001 0 0 6.6760E-02
XO060001 0 0 3.3380E-02
0/ PEACO

C-30: SRAC95 INPUT FOR TCA 553PU
T555
TCA 5.55PU BENCHMARK PROBLEM. USING CRITICAL BUCKLING = 0.00651
1 1 1 1 2 1 4 3 -2 0 0 0 0 2 0 1 0 0 0
0.0065100

J9347.PFASTJ32.DATA OLD FILE 01110000
J9347.PTHMLJ32.DATA OLD FILE 01120000
J9347.PMCRSJ32.DATA OLD FILE 01130000
J9062.FASTU.DATA SCR CORE 01140000
J9062.THERMALU.DATA SCR CORE 01150000
J9062.UMCROSS.DATA SCR CORE 01160000
J9062.MACROWRK.DATA SCR CORE 01170000
J9062.MACRO.DATA SCR CORE 01180000
J9062.FLUX.DATA SCR CORE 01190000
J9062.MICREF.DATA SCR CORE 01200000
61 46 0 /
6(1) /
46(1) /

4 1 1 1 1 1 1 1 0 0 0 5 0 6 2 3 0 0 4 5 0
0 1 0 0 5 0 5 5 0 0 0 0 0 1 0 0 0 0 0 1
11(1) /R-X
1 1 1 2 3 3 3 3 3 /R-M
0.0 0.3074/0.4348 0.5325 0.6115 0.77385 0.9076 1.0240 1.1285
1.2241/1.3200 1.2370 /RX

C-29: SRAC95 INPUT FOR TCA 424PU
T424
TCA 4.24PU BENCHMARK PROBLEM. USING CRITICAL BUCKLING = 0.00779
1 1 1 1 2 1 4 3 -2 0 0 0 0 2 0 1 0 0 0
0.0077900

3
PIN2XP2X 0 10 300.0 1.0650 0.0 /1
XU040001 2 0 7.4360E-07
XU050001 2 0 9.3930E-05
XU080001 2 0 1.2950E-02
XPU0001 2 0 2.0000E-06
XPU0001 2 0 2.7490E-04
XPU0001 2 0 8.8430E-05
XPU0001 2 0 2.8190E-05
XPU0001 2 0 8.1240E-06
XAM10001 2 0 1.0590E-06
XO060001 0 0 2.7840E-02
CLD2XC2X 0 1 300.0 0.1580 0.0 /3
XZRN0001 2 0 3.8400E-02
MOD2XCM2X 0 2 300.0 2.000 0.0 /4
XH01H001 0 0 6.6760E-02
XO060001 0 0 3.3380E-02
0 / PEACO

C-31: SRAC95 INPUT FOR NEA V6E8

V6E8
CELL (BURNUP) CALCULATION BY PU WITH COOLING OPTION : 65 FP CHAIN
1 1 1 2 1 4 3 -2 1 0 0 0 2 0 1 0 0 0 / SRAC CONTROL
1.00E-15 / GEOMETRICAL BUCKLING
J9347.PFAST132.DATA OLD FILE
J9347.PFAST132.DATA OLD FILE
J9347.PTMHLJ32.DATA OLD FILE
J9347.PMCRSJ32.DATA OLD FILE
J4244.FASTUDATA SCR CORE
J4244.THERMALU.DATA SCR CORE
J4244.UMCROSS.DATA SCR CORE
J4244.MACROWRK.DATA SCR CORE
J4244.MACRODATA NEW CORE
J4244.FLUX.DATA SCR CORE
J4244.MICREF.DATA SCR CORE
61 46 1 1 / 107 GROUP => 2 GROUP
61(1) /
46(1) /
61 /
46 /
65 5 3 1 1 5 0 0 0 5 0 6 3 0 0 0 6 0 0 / PATH
0 100 100 5 5 5 -1 0.0001 0.0001 1.0 10. 0.5 /
1 1 1 2 3 / T-R
3(1) / R-X
1 2 3 / M-R
61(1) /
46(1) /
61 /
46 /
65 5 3 1 1 5 0 0 0 5 0 6 15 0 0 3 0 0 / PATH
0 20 50 5 5 -1 0.0001 0.00001 0.001 1.0 10. 0.5 /
1 1 1 2 3 / T-R
3(1) / R-X
1 2 3 / M-R
0.0 0.236714 0.334764 0.41 0.475 0.610200 / D : 95 VM/VP : 1.1
3 / NMAT
FUELX01X 0 7.900 0.0 0.82 0.0 / MAT 1 : FUEL ROD UO2/PUO2
1 1 1 2 3 / T-R
XU050009 2 0 6.0940E-05 /
XU080009 2 0 2.0250E-02 /
XU080009 2 0 1.5630E-03 /
XPU090009 2 0 1.5630E-03 /
XPU090009 2 0 6.8720E-04 /
XPU100009 2 0 2.7650E-04 /
XPU120009 2 0 2.1080E-04 /
XO060009 0 0 4.6100E-02 /
MAT2XOBX 0 4.600 0.13 0.0 / MAT 2 : CLADDING
XPU090009 2 0 3.7020E-02 /
MAT3XLWX 0 2 600 1.0 0.0 / MAT 3 : LIGHT WATER
XH01H008 0 0 4.7440E-02 / VOID 0%
XO060008 0 0 2.3720E-02 /
0 / PEACO

3
XFEN0008 2 0 4.8310E-02 / 1
XCRN0008 2 0 1.5700E-02 / 2
XNIN0008 2 0 7.6480E-03 / 3
XMN50008 2 0 1.4860E-03 / 4
MAT3XLWX 0 2 600 1.0 / MAT 3 : LIGHT WATER
00013400
00013400
00013400
00013400
XH01H008 0 0 4.7440E-02 / 1 VVVVVVVVVVV
XO060008 0 0 2.3720E-02 / 2 VVVVVVVVVVV
0001440
0001440

C-32: SRAC95 INPUT FOR NEA V1E7

V1E7
CELL (BURNUP) CALCULATION BY PU WITH COOLING OPTION : 65 FP CHAIN
1 1 1 2 1 4 3 -2 1 0 0 0 2 0 1 0 0 0 / SRAC CONTROL
1.00E-15 / GEOMETRICAL BUCKLING
J9347.PFAST132.DATA OLD FILE
J9347.PTMHLJ32.DATA OLD FILE
J9347.PMCRSJ32.DATA OLD FILE
J4244.FASTUDATA SCR CORE
J4244.THERMALU.DATA SCR CORE
J4244.UMCROSS.DATA SCR CORE
J4244.MACROWRK.DATA SCR CORE
J4244.MACRODATA NEW CORE
J4244.FLUX.DATA SCR CORE
J4244.MICREF.DATA SCR CORE
61 46 1 1 / 107 GROUP => 2 GROUP
61(1) /
46(1) /
61 /
46 /
65 5 3 1 1 5 0 0 0 5 0 6 3 0 0 0 6 0 0 / PATH
0 100 100 5 5 5 -1 0.0001 0.0001 1.0 10. 0.5 /
1 1 1 2 3 / T-R
3(1) / R-X
1 2 3 / M-R
0.0 0.236714 0.334764 0.41 0.475 0.610200 / D : 95 VM/VP : 1.1
3 / NMAT
FUELX01X 0 7.900 0.0 0.82 0.0 / MAT 1 : FUEL ROD UO2/PUO2
XU050009 2 0 6.1940E-05 /
XU080009 2 0 2.0580E-02 /
XPU100009 2 0 1.3670E-03 /
XPU100009 2 0 6.0990E-04 /
XPU100009 2 0 2.4180E-04 /
XPU120009 2 0 1.8440E-04 /
XO060009 0 0 4.6080E-02 /
MAT2XOBX 0 600 0.13 0.0 / MAT 2 : CLADDING
XZRN0008 2 0 3.7020E-02 /
MAT3XLWX 0 2 600 1.0 0.0 / MAT 3 : LIGHT WATER
XH01H008 0 0 4.7440E-02 / 1 VOID 0%
XO060008 0 0 2.3720E-02 /
0 / PEACO

C-33: SRAC95 INPUT FOR NEA-A

```

NEAA
NEAA - BENCHMARK PROBLEM
1.1.1.2 143.2.1 0 0 0.2 0 1 0 0 / SRAC CONTROL
1.00E-30 / GEOMETRICAL BUCKLING
J9347.PFASTIJ32.DATA OLD FILE
J9347.PTHMLIJ32.DATA OLD FILE
J4244.FASTU DATA OLD FILE
J4244.THERMALU DATA SCR CORE
J4244.UMCROSS.DATA SCR CORE
J4244.MACROWRK.DATA SCR CORE
J4244.MACRO.DATA NEW CORE
J4244.FLUX.DATA SCR CORE
J4244.MICREF.DATA SCR CORE
61 46 1 1 / 107 GROUP => 2 GROUP
46(1) /
61(1) /
46 /
46 /
47 73 1 17 0 0 0 5 0 6 23 0 0 45 0 / PATH
0.50 50 5.5 1 0.0001 0.00001 0.001 1.0 10. 0.5 /
00012400
1.1 1 2 3 3 3 / T-R
3(1) / R-X
0.50 50 5.5 1 0.0001 0.00001 0.001 1.0 10. 0.5 /
00012500
1.2 3 / M-R
0.0 0.2364 0.3344 0.4095 0.475 0.5774 0.6642 0.656650 /
00000100
3 / NMAT
FUELX01X 0.9 933.0 0.819 0.0 / MAT 1 : FUEL.ROD UO2/PUO2
XU040009 2 0 2.4626E-07 /1
XU050009 2 0 5.1515E-05 /2
XU080009 2 0 2.0295E-02 /3
XPU80009 2 0 2.1800E-05 /4
XPU90009 2 0 7.1155E-04 /5
XPU00009 2 0 2.7623E-04 /6
XPU10009 2 0 1.4591E-04 /7
XPU20009 2 0 4.7643E-05 /8
X0060009 2 0 4.3100E-02 /9
MAT2X0BX 0 1.579.3 0.131 0.0 / MAT 2 : CLADDING
00000600
00013000
XZRN0008 2 0 4.3248E-02 /1
MAT3XLWX 0 4.579.3 1.0 0 / MAT 3 : MODERATOR
00013500
XH01H008 0 0.4.7716E-02 /1 VVVVVVVVVV
X0060008 0 0 2.3858E-03 /2 VVVVVVVVVV
XB000008 0 0 3.6346E-06 /2 VVVVVVVVVV
XB010008 0 0 1.6226E-05 /2 VVVVVVVVVV
0 / PEACO
00014400

FUELX01X 0.9 933.0 0.819 0.0 / MAT 1 : FUEL
00000300
XU050009 2 0 1.4456E-04 /1
XU080009 0 1.9939E-02 /2
XPU180009 2 0 1.1467E-04 /3
XPU190009 2 0 1.0283E-03 /4
XPU100009 2 0 7.9657E-04 /5
XPU130009 2 0 3.3997E-04 /6
XPU20009 2 0 5.6388E-04 /7
X0060009 2 0 4.5854E-02 /8
MAT2X0BX 0 1.579.3 0.131 0.0 / MAT 2 : CLADDING
00000600
00013000
XZRN0008 2 0 4.3248E-02 /1
MAT3XLWX 0 4.579.3 1.0 0 / MAT 3 : LIGHT WATER
00013500
XH01H008 0 0.4.7716E-02 /1 VVVVVVVVVV
X0060008 0 0 2.3858E-03 /2 VVVVVVVVVV
XB000008 0 0 3.6346E-06 /2 VVVVVVVVVV
XB010008 0 0 1.6226E-05 /2 VVVVVVVVVV
0 / PEACO
00014400

MAT2X0BX 0 1.579.3 0.131 0.0 / MAT 2 : CLADDING
00000600
00013000
XZRN0008 2 0 4.3248E-02 /1
MAT3XLWX 0 4.579.3 1.0 0 / MAT 3 : LIGHT WATER
00013500
XH01H008 0 0.4.7716E-02 /1 VVVVVVVVVV
X0060008 0 0 2.3858E-03 /2 VVVVVVVVVV
XB000008 0 0 3.6346E-06 /2 VVVVVVVVVV
XB010008 0 0 1.6226E-05 /2 VVVVVVVVVV
00013700
00013700
00013700
0 / PEACO
00014400

```

C-34: SRAC95 INPUT FOR NEA-B

```

NEAB
NEAB - BENCHMARK PROBLEM
1.1.1.2 143.2.1 0 0 0.2 0 1 0 0 / SRAC CONTROL
1.00E-30 / GEOMETRICAL BUCKLING
J9347.PFASTIJ32.DATA OLD FILE
J9347.PTHMLIJ32.DATA OLD FILE
J9347.PMCRSJ32.DATA OLD FILE
J4244.FASTU DATA SCR CORE

```